

December 12, 2001

LICENSEE: FirstEnergy Nuclear Operating Company

FACILITY: Davis-Besse Nuclear Power Station, Unit 1

SUBJECT: SUMMARY OF NOVEMBER 27, 2001, PUBLIC MEETING TO DISCUSS THE LICENSEE'S RESPONSE TO BULLETIN 2001-01

On November 27, 2001, Nuclear Regulatory Commission (NRC) management and staff met in Rockville, Maryland, with representatives of FirstEnergy Nuclear Operating Company and their contractors. The meeting was requested by the licensee in order to supplement their response to NRC Bulletin 2001-01, "Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles." The licensee has proposed that there is reasonable assurance that Davis-Besse is safe to operate until the start of their 13th refueling outage which is scheduled to begin in February 2002. The bulletin recommended that reactor pressure vessel head inspections be performed prior to December 31, 2001. The purpose of the meeting was to (1) discuss the licensee's revised probabilistic safety assessment based upon comments received from the staff during the meetings of November 9, and November 14, 2001, and (2) answer NRC questions concerning the licensee's responses to the staff's requests for additional information and other submittals. A listing of the meeting participants is included as Enclosure 1. The meeting handouts are included as Enclosure 2.

At the conclusion of the meeting, the staff requested the licensee to docket additional information that the staff considered necessary to complete their review. This information included commitments to:

- (1) shutdown by February 16, 2002, to perform vessel head penetration (VHP) nozzle inspections;
- (2) Implement any compensatory measures and schedule for implementation such as:
  - operating at a lower reactor coolant system hot leg temperature to reduce the head temperature effects on crack initiation and growth (598 °F vs. 605 °F),
  - maximizing the availability of the plant's redundant critical safety systems until shutdown,
  - providing increased human factors reliability through additional training and personnel (e.g., dedicated operator - emergency core cooling system switchgear from injection to recirculation and include small-break loss-of-coolant accident on the simulator);
- (3) perform inspection of 100 percent of VHP nozzles (e.g., qualified visual examination or other non-destructive examination technique(s));

- (4) perform the PSA for the upper (95 percent) Projection 1.5 Shape Parameter without any credit for the inspection completed in 1998 and 2000, and provide the results;
- (5) explain the basis for the reduction in core damage frequency presented at this meeting, by a factor of 30, from that which was presented to NRC in the meeting of November 14, 2001;
- (6) provide the inspection history of other plants' nozzles where no axial or circumferential leaks have been identified on the same material heats used in Davis-Besse nozzles;
- (7) arrange a future meeting prior to shutting down on February 16, 2001, to discuss Davis-Besse's inspection plans (e.g., scope, method, etc.); and
- (8) characterize identified flaws through destructive examination.

*/RA/*

Stephen P. Sands, Project Manager, Section 2  
Project Directorate III  
Division of Licensing Projects Management  
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosures: 1. List of Meeting Participants  
2. Meeting Handout

cc w/encls: See next page

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- (7) arrange a future meeting prior to shutting down on February 16, 2001, to discuss Davis-Besse's inspection plans (e.g., scope, method, etc.); and
- (8) characterize identified flaws through destructive examination.

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DMcCain	NRR Participants
TBergman	

**ADAMS ACCESSION NO: ML013340215**

OFFICE	PM:PDIII-2	LA:PDIII-2	BC:EMCB	SC:PDIII-2
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DATE	12/11/01	12/12/01	12/12/01	12/12/01

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NOVEMBER 28, 2001

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