



RESPONDING TO FUKUSHIMA-DAIICHI

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Overview

- Timeline
- Near-Term Task Force
- Enhancements
- Spent Fuel Safety
- Conclusions

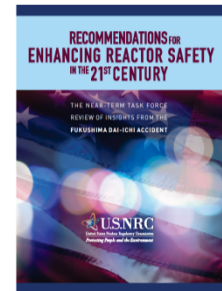
United States

NIC

Nuclear Infrastructure
Council



Timeline



Current U.S. Plant Safety

- Similar sequence of events in the U.S. is unlikely
- Existing mitigation measures could reduce the likelihood of core damage and radiological releases
- No imminent risk from continued operation and licensing activities



Enhancements without Delay

- Reevaluate external hazards, including seismic and flooding hazards
- Perform seismic and flooding hazard walkdowns
- Modify SBO rule to require enhanced capability to mitigate prolonged SBO



Enhancements without Delay (cont'd)

- Mitigation Strategies for Beyond Design Basis External Events
- Require reliable hardened vent designs in BWRs with Mark I and II containments
- Enhancement of spent fuel pool instrumentation



Enhancements without Delay (cont'd)

- Strengthen and integrate onsite emergency response capabilities
- Require staffing and communications equipment to respond to multiunit events and prolonged SBO



Recommendations for Other Actions

- Tier 2 Recommendations – Could not be initiated in the near term due to resource or critical skill set limitations
- Tier 3 Recommendations – Require further staff study to support a regulatory action



Additional Issues

- Filtration of Containment Vents
- Seismic Monitoring Instruments
- Emergency Planning Zone Size
- KI Beyond 10 Miles
- Dry Cask Storage
- Loss of Ultimate Heat Sink



Approach on Near Term Enhancements

- Commission approved implementation of specific recommendations
 - Issue orders, requests for information (50.54(f) letters), and new regulations
 - Seek stakeholder input in determining action on each recommendation



Schedule



- FY2012 Appropriations – Accelerate schedule
- NRC goal is to issue Tier 1 Orders and 50.54(f) letters by March 11th
- Planning to submit to the Commission by February 17; Commission will direct the staff
- Overall Goal – Complete enhancements in 5 years

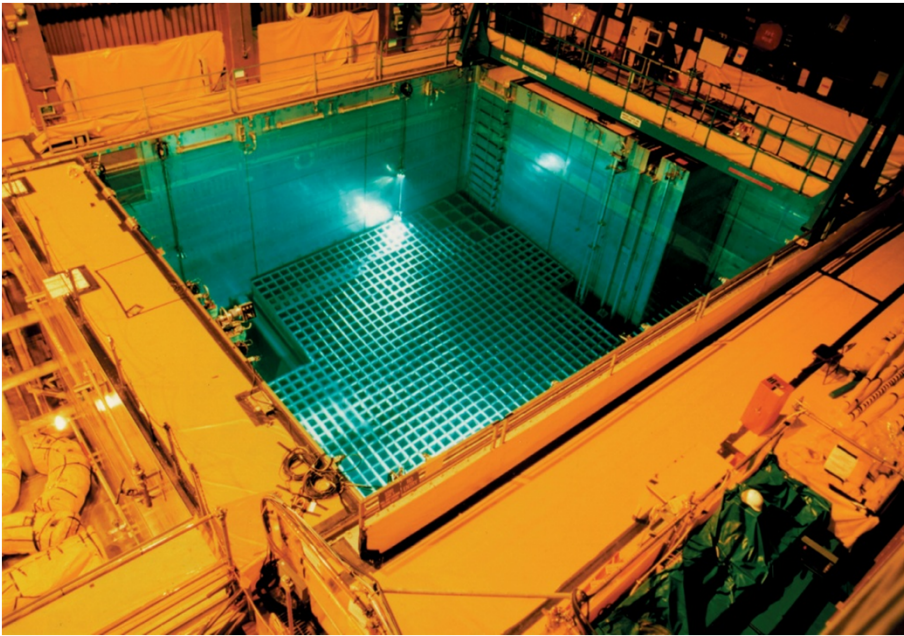


NRC Follow-up

- Review and assess licensee responses
- Establish necessary regulatory framework
- Inspect to ensure compliance with all new regulatory requirements
- Consider implications for other nuclear facilities



U.S. Spent Fuel Pools



- Spent fuel rods stored in spent fuel pools (SFPs) under at least 20 feet of water
- Typically $\sim 1/4$ to $1/3$ of fuel in reactor replaced with fresh fuel every 18 to 24 months
- Spent fuel stored in pools minimum of 5 years



Spent Fuel Safety

- Spent Fuel Pools (SFP) originally designed for limited storage of spent fuel until removed off-site
- Safety achieved primarily by maintaining water inventory, geometry, and soluble boron (PWRs)
- Drain down can lead to uncovered fuel, heat-up, and the release of radionuclides



Risk of Large Release

- SFP risk is low, due to the very low likelihood of events that could damage the thick reinforced pool walls
 - Likelihood of fuel uncover is low; $6E-7$ to $2E-6$ /yr (NUREG-1738)
 - Potential consequences may be large due to heatup of the fuel in the pool
 - Heatup of the fuel in the pool can lead to “zirconium fire” initiation and propagation
 - Large inventory of ^{137}Cs available for release



Spent Fuel Safety and Security

- NRC extensively reexamined pool safety and security after 9-11 attacks
 - Low vulnerability to attack
 - Significantly improved analysis of fuel coolability / heatup
 - Mitigation measures improved passive coolability of fuel
 - Improved fuel configuration within the pool achieves substantially greater passive cooling capability by natural convection



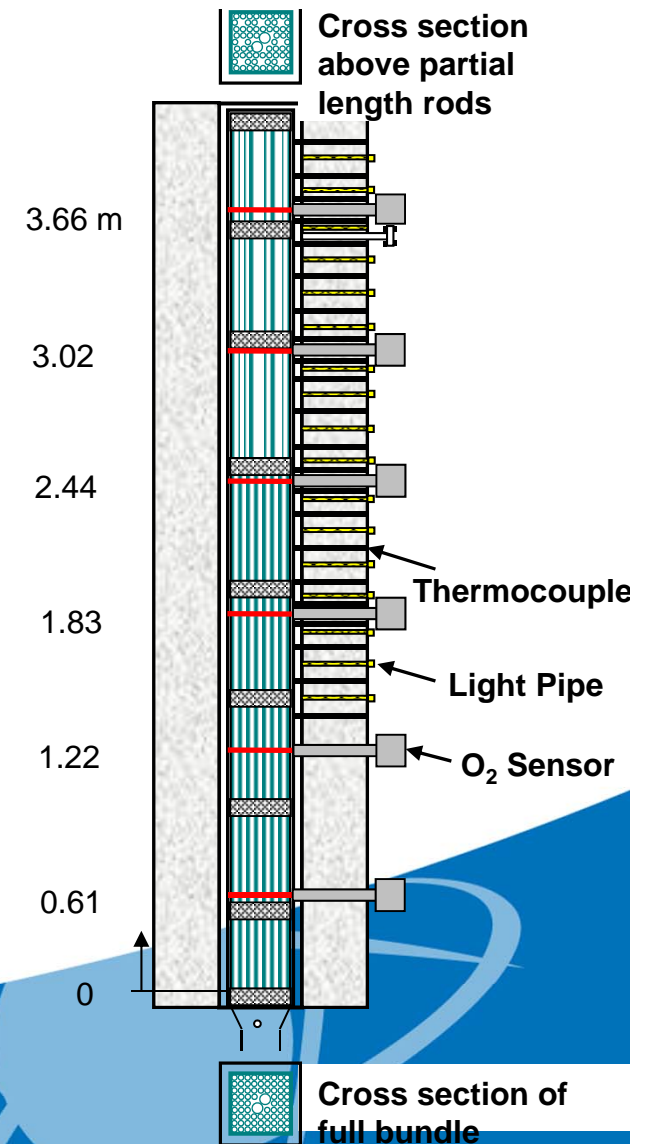
Spent Fuel Safety and Security

- NRC required spray capability for each site to improve active cooling capability
- Licensees performed site-specific assessments; NRC inspected
- Coolability of fuel within pools has been enhanced by measures identified and assessed as part of post-9/11 measures
- Conducting research to confirm understanding and validate analytical modeling



Zirconium Fire Investigations During SFP Loss of Coolant Accident (LOCA)

- Prototypic full length 9×9 BWR hardware
 - Single pool rack cell
 - Upper & lower tie plates with seven spacers
 - Water tubes and channel box
 - 74 electric heater rods with Zr-2 cladding (eight partial length)
 - 5000 W simulating a 100 day old assembly
- Measurements
 - Temp profiles: Axial and radial
 - Induced flow: Effect of ignition on flow
 - O₂ concentration: Determine depletion
 - Nature of fire: Initiation location & axial burn rate



Zirc Fire Investigations During SFP LOCA – Post-test



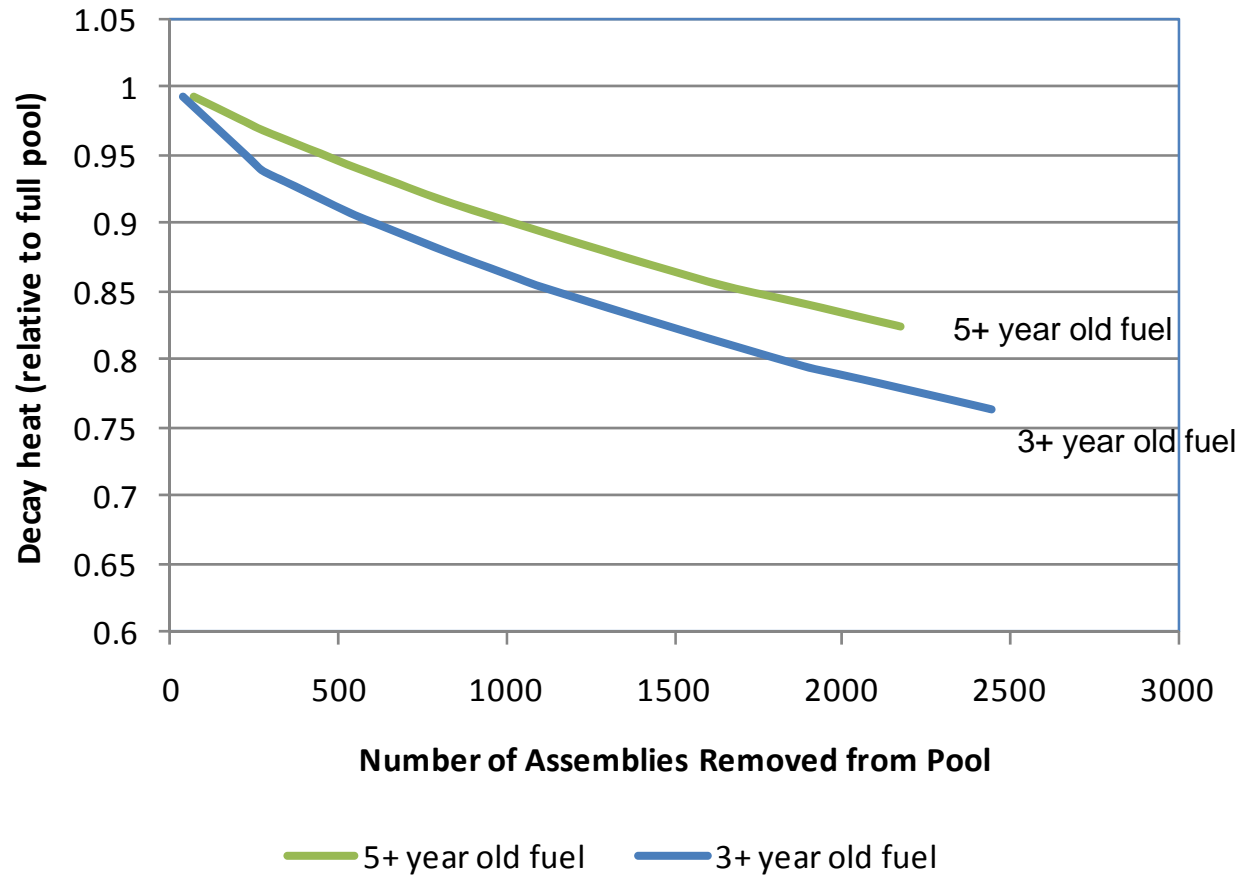
Removing Fuel from Pools

- NRC has considered benefits of removal of fuel from the pool and returning to a low density racking type configuration
- There are competing factors in such a consideration
 - Storage in dry casks must be consistent with cask design
 - Discharging of fuel increases the risk of cask drops and increases worker doses
 - Removal of fuel will decrease the inventory of Cesium-137
 - Removal of fuel does not appreciably reduce decay heat (most of the decay heat is from recently discharged fuel)
 - Reduction in potential land contamination and economic impacts, if a large release occurred



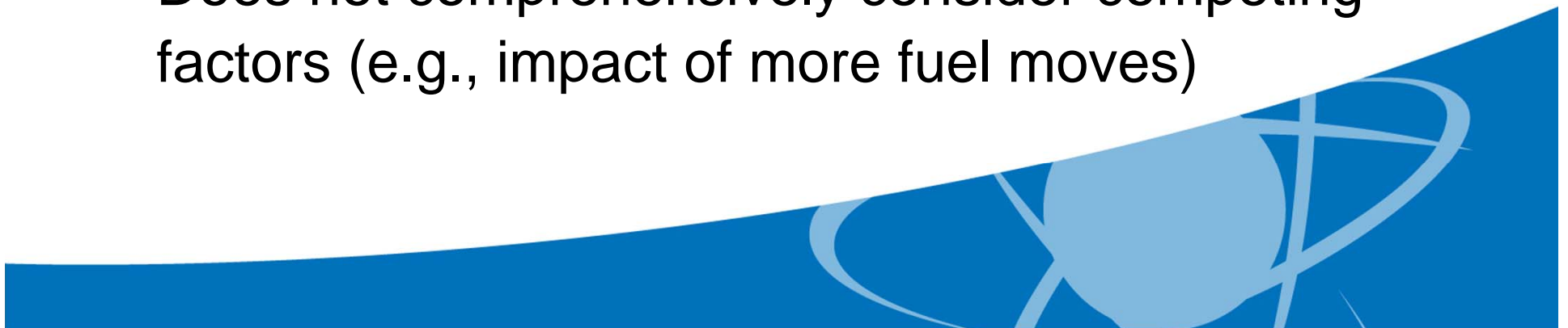
Impact of Removing Assemblies

Reduction of pool thermal heat load



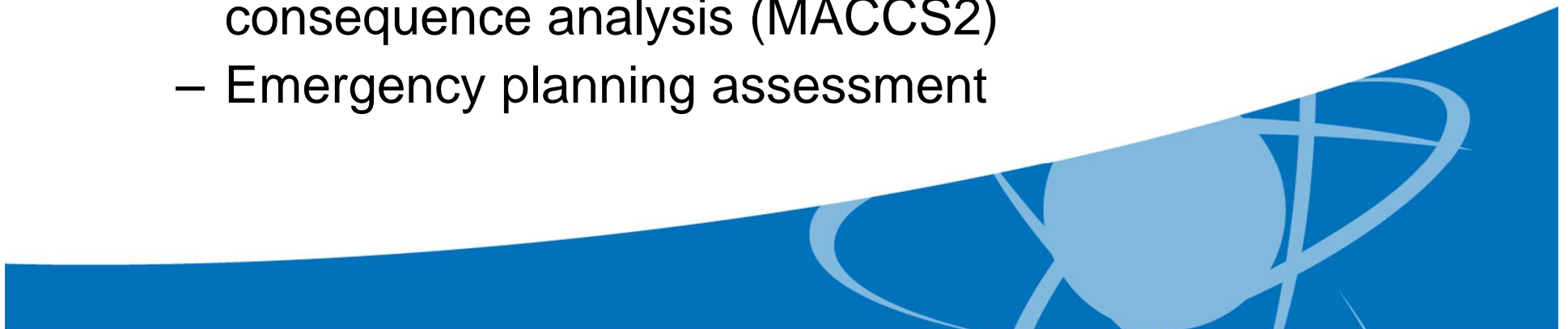
Spent Fuel Pool Scoping Study

- NRC has initiated an updated SFP study
- Estimate the change in accident consequences associated with removing older fuel from the SFP and placing it in dry storage
- Limited scope analysis (e.g., single SFP/operating cycle for low/high density racking)
- Does not comprehensively consider competing factors (e.g., impact of more fuel moves)



Comparative Consequences

- Technical approach relies on realistic analysis using expedient and technically-defensible deterministic methods and assumptions
- Elements of study include
 - Information gathering
 - Seismic and structural assessment
 - Accessibility, decay heat, and radionuclide inventory assessment
 - Accident progression (MELCOR) and offsite consequence analysis (MACCS2)
 - Emergency planning assessment



Conclusions

- No immediate safety concerns based on Fukushima nuclear emergency
- Confirmed the existing safety measures for nuclear power plants, including SFPs
- Moving forward with nuclear power plant enhancements
- Examining additional near-term and long-term reviews
- Spent fuel must be managed safely and securely

