UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
611 RYAN PLAZA DRIVE, SUITE 400
ARLINGTON, TEXAS 76011-4005

February 27, 2006

James J. Sheppard, President and
Chief Executive Officer
STP Nuclear Operating Company
P.O. Box 289
Wadsworth, Texas 77483

SUBJECT: SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION - NRC RADIATION SAFETY TEAM INSPECTION REPORT 05000498/2006008; 05000499/2006008

Dear Mr. Sheppard:

On January 26, 2006, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at your South Texas Project Electric Generating Station, Units 1 and 2, facility The enclosed Radiation Safety Team inspection report documents the inspection findings which were discussed with Mr. Gary Parkey, Executive Vice President of Generation and General Plant Manager, and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The team reviewed selected procedures and records, observed activities, and interviewed personnel. Specifically, the team evaluated the inspection areas within the Radiation Protection Strategic Performance Area that are scheduled for review every two years. These areas are:

- Radiation Monitoring Instrumentation
- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems
- Radioactive Material Processing and Transportation
- Radiological Environmental Monitoring Program and Radioactive Material Control Program

This inspection report documents one self-revealing, non-cited violation of very low safety significance (Green). Additionally, one licensee-identified violation, which was also determined to be of very low safety significance, is listed in Section 4OA7 of this report. If you contest these non-cited violations or their significance, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington DC 20555-0001; with copies to the Regional Administrator, U.S. Nuclear Regulatory Commission Region IV, 611 Ryan Plaza Drive, Suite 400, Arlington, Texas 76011-4005; the Director, Office of Enforcement, U.S. Nuclear Regulatory Commission, Washington DC 20555-001; and the NRC Resident Inspector at the South Texas, Units 1 and 2, facility.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).
Should you have any questions concerning this inspection, we will be pleased to discuss them with you.

Sincerely,

//RA//

Michael P. Shannon, Chief
Plant Support Branch
Division of Reactor Safety

Dockets: 50-498
50-499
Licenses: NPF-76
NPF-80

Enclosure:
NRC Inspection Report 05000498/2006008; 05000499/2006008
w/Attachment: Supplemental Information

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Senior Project Engineer, DRP/A **(TRF)**
Team Leader, DRP/TSS **(RLN1)**
RITS Coordinator **(KEG)**

Only inspection reports to the following:
DRS STA **(DAP)**
T. Bloomer, OEDO RIV Coordinator **(TEB)**
ROPreports
STP Site Secretary **(LAR)**

Sunsi Review Completed: Yes ADAMS: Yes ☐ No Initials: mps
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RIV:DRS/PSB	PSB	PSB	PSB	C:PSB
LCCarsonII	GLGuerra	LTRicketson	BDBaca	DLStearns
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C:PSB	C:DRP/A	C:PSB		
MPShannon	CEJohnson	MPShannon		
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02/ 27 /06	02/ 27 /06	02/ 27 /06		

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ENCLOSURE

U.S. NUCLEAR REGULATORY COMMISSION
REGION IV

Dockets:	50-498, 50-499
Licenses:	NPF-76 NPF-80
Report No:	05000498/2006008 05000499/2006008
Licensee:	STP Nuclear Operating Company
Facility:	South Texas Project Electric Generating Station, Units 1 and 2
Location:	FM 521 - 8 miles west of Wadsworth Wadsworth, Texas 77483
Dates:	January 23 - 26, 2006
Inspectors:	L. C. Carson II, Senior Health Physicist - Team Leader B. D. Baca, Health Physicist G. L. Guerra, Health Physicist L. T. Ricketson, P.E., Senior Health Physicist D. L. Stearns, Health Physicist
Approved By:	Michael P. Shannon, Chief, Plant Support Branch Division of Reactor Safety

SUMMARY OF FINDINGS

IR 05000498/2006008, 05000499/2006008; January 23-26 2006; South Texas Project Electric Generating Station; Radiation Monitoring Instrumentation and Protective Equipment; Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems; and Radiological Environmental Monitoring Program and Radioactive Material Control Program

The report covered a one week period of inspection on site by a team of five region-based health physics inspectors. Based upon the results of the inspection, the team reviewed one self-revealing violation of very low safety significance (Green). The significance of most findings is indicated by their color (Green, White, Yellow, Red) using IMC 0609, "Significance Determination Process," (SDP). Findings for which the SDP does not apply may be "Green" or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

A. NRC-Identified and Self-Revealing Findings

Cornerstone: Public Radiation Safety (PS)

- Green. The team reviewed two examples of a self-revealing non-cited violation of Technical Specification 6.8.1, resulting from the licensee's failure to prevent radioactive material from being unconditionally released from a radiologically controlled area. The first example involved a radiation detection instrument with fixed radioactive contamination. The second example involved a contaminated lifting sling that was used to remove equipment and containers from the containment building. In both examples, the radioactive material was identified after it was removed from a radiologically controlled area but before it left the protected area. Corrective actions for the first example involved counseling the responsible individual. Corrective actions for the second example are still being evaluated. Both examples were entered into the licensee's corrective action program as Condition Reports 04-4266 and 05-14345.

  This finding is greater than minor because it was associated with a Public Radiation Safety cornerstone attribute (material release) and it affected the associated cornerstone objective in that the failure to control radioactive material decreases the licensee's assurance that the public will not receive unnecessary dose. Using the Public Radiation Safety Significance Determination Process, the team determined that the finding had very low safety significance because: (1) the finding was a radioactive material control finding, (2) it was not a transportation finding, (3) it did not result in public dose greater than 0.005 rem, and (4) radioactive material was not released from the protected area more than five times. Additionally, this finding had cross-cutting aspects associated with human performance. In the first example, a radiation protection technician failed to maintain direct supervision of the contaminated instrument. In the second example, the procedural guidance allowed the licensee to use only portable GM instruments on large items despite the loss of detection sensitivity.

Licensee Identified Violations

One violation of very low safety significance, which was identified by the licensee, has been reviewed by the team. Corrective actions taken or planned by the licensee have been entered into the licensee's corrective action program. This violation and corrective action is listed in Section 4OA7 of this report.

Report Details

2. RADIATION SAFETY
Cornerstone: Occupational Radiation Safety [OS]

2OS3 Radiation Monitoring Instrumentation and Protective Equipment (71121.03)

a. Inspection Scope

This area was inspected to determine the accuracy and operability of radiation monitoring instruments that are used for the protection of occupational workers and the adequacy of the program to provide self-contained breathing apparatus (SCBA) to workers. The team used the requirements in 10 CFR Part 20 and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- Calibration of area radiation monitors associated with transient high and very high radiation areas and post-accident monitors used for remote emergency assessment
- Calibration of portable radiation detection instrumentation, electronic alarming dosimetry, and continuous air monitors used for job coverage
- Calibration of whole body counting equipment and radiation detection instruments utilized for personnel and material release from the radiologically controlled area
- Licensee Event Reports, audits and self-assessments
- Corrective action program reports since the last inspection
- Licensee action in cases of repetitive deficiencies or significant individual deficiencies
- Calibration expiration and source response check currency on radiation detection instruments staged for use
- The licensee's capability for refilling and transporting SCBA air bottles to and from the control room and operations support center during emergency conditions, status of SCBA staged and ready for use in the plant and associated surveillance records, and personnel qualification and training
- Qualification documentation for onsite personnel designated to perform maintenance on the vendor-designated vital components, and the vital component maintenance records for SCBA units.

The inspector completed 9 of the required 9 samples.

b. Findings

No findings of significance were identified.

2PS1 Radioactive Gaseous And Liquid Effluent Treatment And Monitoring Systems (71122.01)

a. Inspection Scope

This area was inspected to ensure that the gaseous and liquid effluent processing systems are maintained so that radiological releases are properly mitigated, monitored, and evaluated with respect to public exposure. The team used the requirements in 10 CFR Part 20, 10 CFR Part 50 Appendices A and I, the Offsite Dose Calculation Manual, and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- The most current radiological effluent release reports, changes to radiation monitor setpoint calculation methodology, anomalous sampling results, effluent radiological occurrence performance indicator incidents, self-assessments, audits, and licensee event reports
- Gaseous and liquid release system component configurations
- Routine processing, sample collection, sample analysis, and release of radioactive liquid and gaseous effluent; and Radioactive liquid and gaseous effluent release permits and dose projections to members of the public
- Abnormal releases
- Changes made by the licensee to the Offsite Dose Calculation Manual, the liquid or gaseous radioactive waste system design, procedures, or operation since the last inspection
- Monthly, quarterly, and annual dose calculations
- Surveillance test results involving air cleaning systems and stack or vent flow rates
- Instrument calibrations of discharge effluent radiation monitors and flow measurement devices, effluent monitoring system modifications, effluent radiation monitor alarm setpoint values, and counting room instrumentation calibration and quality control
- Interlaboratory comparison program results
- Licensee event reports, special reports, audits, self-assessments and corrective action reports performed since the last inspection

The inspector completed 10 of the required 10 samples.

b. Findings

No findings of significance were identified.

2PS2 Radioactive Material Processing and Transportation (71122.02)

a. Inspection Scope

This area was inspected to verify that the licensee's radioactive material processing and transportation program complies with the requirements of 10 CFR Parts 20, 61, and 71 and Department of Transportation regulations contained in 49 CFR Parts 171-180. The team interviewed licensee personnel and reviewed:

- The radioactive waste system description, recent radiological effluent release reports, and the scope of the licensee's audit program
- Liquid and solid radioactive waste processing systems configurations, the status and control of any radioactive waste process equipment that is not operational or is abandoned in place, changes made to the radioactive waste processing systems since the last inspection, and current processes for transferring radioactive waste resin and sludge discharges
- Radio-chemical sample analysis results for radioactive waste streams and use of scaling factors and calculations to account for difficult-to-measure radionuclides
- Shipping records for non-excepted package shipments
- Licensee event reports, special reports, audits, state agency reports, self-assessments and corrective action reports performed since the last inspection

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

- Shipment packaging, surveying, labeling, marking, placarding, vehicle checking, driver instructing, and disposal manifesting

The inspector completed 6 of the required 6 samples.

2PS3 Radiological Environmental Monitoring Program (REMP) And Radioactive Material Control Program (71122.03)

a. Inspection Scope

This area was inspected to ensure that the REMP verifies the impact of radioactive effluent releases to the environment and sufficiently validates the integrity of the radioactive gaseous and liquid effluent release program; and that the licensee's surveys and controls are adequate to prevent the inadvertent release of licensed materials into the public domain. The team used the requirements in 10 CFR Part 20, Appendix I of 10 CFR Part 50, the Offsite Dose Calculation Manual, and the licensee's procedures

required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed

- Annual environmental monitoring reports and licensee event reports
- Selected air sampling and thermoluminescence dosimeter monitoring stations
- Collection and preparation of environmental samples
- Operability, calibration, and maintenance of meteorological instruments
- Each event documented in the Annual Environmental Monitoring Report which involved a missed sample, inoperable sampler, lost thermoluminescence dosimeter, or anomalous measurement
- Significant changes made by the licensee to the Offsite Dose Calculation Manual as the result of changes to the land census or sampler station modifications since the last inspection
- Calibration and maintenance records for air samplers, composite water samplers, and environmental sample radiation measurement instrumentation, quality control program, interlaboratory comparison program results, and vendor audits
- Locations where the licensee monitors potentially contaminated material leaving the radiological controlled area [or controlled access area] and the methods used for control, survey, and release from these areas
- Type of radiation monitoring instrumentation used to monitor items released, survey and release criteria of potentially contaminated material, radiation detection sensitivities, procedural guidance, and material release records
- Licensee event reports, special reports, audits, self-assessments and corrective action reports performed since the last inspection

The inspector completed 10 of the required 10 samples.

b. Findings

Introduction. The team reviewed two examples of a Green, self-revealing non-cited violation of Technical Specification 6.8.1, resulting from the licensee's failure to prevent radioactive material from being unconditionally released from the radiologically controlled area.

Description. On March 31, 2004, a radiation protection technician observed a radiation detection instrument outside a radiologically controlled area behind the Unit 2 equipment hatch lay down area. Upon further examination, the radiation protection technician noted a "Radioactive Material" label on the instrument. This alerted the radiation protection technician to the fact that radioactive material had been unconditionally released from a radiologically controlled area. Surveys of the instrument indicated no loose radioactive contamination, but did identify 3.7 to 3.9 nanocuries of fixed

radioactive contamination. The licensee concluded that another radiation protection technician assigned to frisk items in the area left the instrument unattended and failed to return it to the radiologically controlled area at the end of the work shift.

On October 29, 2005, an individual carrying a lifting sling from the maintenance operations facility caused a contamination monitor in the east gatehouse to alarm. Radiation protection personnel responded to the alarm and determined the sling was contaminated with radioactive material (approximately 149 nanocuries). The licensee concluded that the radioactive material originated from equipment and containers rigged out of the reactor containment building during outages. Prior to being released from the containment building, the all items were surveyed using portable Geiger-Muller (GM) friskers. However, the portable GM friskers were not capable of detecting the all quantities of radioactive material present on the items. Even though it was not in the containment building, the lifting sling captured and concentrated small amounts of radioactive material in the webbing. The lifting sling was also surveyed with portable GM friskers and again the radioactive material was not detected. A more sensitive measurement device, such as a tool monitor with scintillation detectors was not required by the licensee's procedure to supplement the portable GM friskers and therefore was not used.

In both of these examples, although the radioactive material was allowed outside radiologically controlled areas, it was not removed from the licensee's protected area.

Analysis. NRC regulations provide no minimum level of licensed radioactive material that can be disposed of in a manner other than as radioactive waste or transferred to a licensed recipient, with one exception in 10 CFR 20.2005.

In the first example, the licensee intended the instrument with fixed radioactive contamination be "conditionally released" from the radiologically controlled area under the control of a radiation protection technician. However, when the radiation protection technician using the instrument failed to maintain oversight of it, it was then unconditionally released. The failure of the radiation protection technician to control radioactive material outside the radiologically controlled area was a performance deficiency.

In the second example, the licensee failed to detect the radioactive material present in the lifting sling before it was released from the radiological controlled area. Information Notice No. 85-92, "Surveys of Waste Before Disposal from Nuclear Reactor Facilities," and Health Physics Position 073 provide guidance applicable to situations in which small amounts of radioactive material are accumulated. The guidance states that, "In order to preclude the unintentional release of radioactive material, a good monitoring program likely would include careful surveys using equipment and techniques for detecting very low levels of radioactivity. Surveys conducted with portable survey instruments using pancake GM probes are generally more appropriate for small items and small areas because of the loss of detection sensitivity created by moving the probe and the difficulties in completely scanning large areas. This does not preclude the use of such instruments for larger items and areas, if supplemented by other survey equipment or techniques." The accumulation of small amounts of contamination that escaped pancake probe detection may be detected using detectors sensitive to gamma radiation, such as by using a sensitive scintillation detector in a low-background area. The licensee did not supplement its surveys made with portable pancake GM probes with

surveys made with a sensitive scintillation detector. Therefore, the accumulated radioactive material was not discovered until an individual attempted to take the lifting sling from the protected area and entered a more sensitive radiation detection device located outside the radiologically controlled area. The failure to confine radioactive material to the radiologically controlled area is a performance deficiency.

This finding is greater than minor because it was associated with a Public Radiation Safety cornerstone attribute (material release) and it affected the associated cornerstone objective in that the failure to control radioactive material decreases the licensee's assurance that the public will not receive unnecessary dose. Using the Public Radiation Safety Significance Determination Process, the team determined that the finding had very low safety significance because: (1) the finding was a radioactive material control finding, (2) it was not a transportation finding, (3) it did not result in public dose greater than 0.005 rem, and (4) radioactive material was not released from the protected area more than five times. Additionally, this finding had cross-cutting aspects associated with human performance. In the first example, a radiation protection technician failed to maintain direct supervision of the contaminated instrument. In the second example, the procedural guidance allowed the licensee to use only portable GM instruments on large items despite the loss of detection sensitivity.

Enforcement. Technical Specification 6.8.1.a requires written procedures to be established, implemented, and maintained covering the applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978. Section 7 of Appendix B includes, "Procedures for Control of Radioactivity (For limiting materials released to environment and limiting personnel exposure)." Procedure 0PGP03-ZR-0053, Radioactive Material Control Program, Revision 11, addresses this requirement and states in Section 7.5.2, "Items and materials which contain licensed radioactivity, as determined by Radiation Protection procedures, SHALL NOT be unconditionally released from the radiologically controlled area." The licensee violated this requirement when it allowed the contaminated survey instrument and lifting sling to be unconditionally released from the radiologically controlled area.

Because the failure to confine radioactive material to the radiologically controlled area was determined to be of very low safety significance and the examples were entered into the licensee's corrective action program as Condition Reports 04-4266 and 05-14345, this violation is being treated as a non-cited violation, consistent with Section VI.A of the NRC Enforcement Policy: NCV 05000498/2006008-01; 05000499/2006008-01 - Failure to confine radioactive material to the radiologically controlled area.

4. OTHER ACTIVITIES

4OA2 Problem Identification and Resolution

Annual Sample Review

a. Inspection Scope

The team evaluated the effectiveness of the licensee's problem identification and resolution process with respect to the following inspection areas:

- Radiation Monitoring Instrumentation (Section 2OS3)
- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems (Section 2PS1)
- Radioactive Material Processing and Transportation (Section 2PS2)
- Radiological Environmental Monitoring Program and Radioactive Material Control Program (Section 2PS3)

b. Findings and Observations

No findings of significance were identified.

4OA6 Management Meetings

Exit Meeting Summary

On January 26, 2006, the team presented the inspection results to Mr. G. Parkey, Executive Vice President of Generation and General Plant Manager, and other members of his staff who acknowledged the findings. The team confirmed that proprietary information was not provided or examined during the inspection.

4OA7 Licensee-Identified Violations

The following violation of very low safety significance (Green) was identified by the licensee and is a violation of NRC requirements which meet the criteria of Section VI of NUREG-1600, "NRC Enforcement Policy," for being dispositioned as an NCV.

Department of Transportation Regulation 49 CFR 172.504(a) requires exclusive use shipments of low specific activity material to be placarded with a "RADIOACTIVE" placard on each side and each end of the vehicle. The licensee violated this requirement on October 20, 2004, when a shipment of low specific activity material was shipped to a processor in Tennessee via an exclusive use vehicle without the required placards. During a review of the documentation and regulations the next morning, the licensee identified the error, contacted the vehicle driver and instructed him to display the correct placards on the vehicle. This violation is of very low safety significance because the issue is a violation of transportation requirements, did not exceed radiation limits, did not result in a breach of the package during transit, did not involve the requirements of the Certificate of Compliance, was not a nonconformance with low level burial ground requirements, and was not a failure to make notifications or to provide emergency information. This event was documented in the licensee's corrective action program as CR-04-14179.

ATTACHMENT

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee personnel

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W. Curry, Senior Metrology Laboratory Technician, Metrology Laboratory
L. Earls, Consulting Engineer, Radiation Protection
E. Hardcastle, Laboratory Analyst, Health Physics
S. Head, Manager, Licensing
J. Houston, Senior Radwaste Specialist, Radiation Protection
R. Jones, Staff Lead Metrology Specialist, Metrology and Radiological Laboratories
G. Parkey, Executive Vice President of Generation and General Plant Manager
A. Passafuma, Radiological Environment Monitoring Program Technician, Health Physics
T. Riccio, Engineer, Instrumentation/Monitoring Systems
R. Savage, Senior Staff Specialist, Licensing
J. Sepulveda, Supervisor, Radiation Protection
D. Sherwood, Radiological Services Supervisor, Health Physics
J. Sheppard, President and Chief Executive Officer
D. Scoggins, Supervisor, Metrology Laboratory
M. Tomek, Radiation Protection Supervisor
G. Williams, REMP / Dosimetry Health Physicist, Radiation Protection

NRC

J. Cruz, Senior Resident Inspector

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

Opened and Closed During this Inspection

50-499/2006008-01	NCV	Failure to confine radioactive material to a radiologically controlled area. (Section 2PS3)

Previous Items Closed

NONE

Previous Items Discussed

NONE

LIST OF DOCUMENTS REVIEWED

Section 2OS3: Radiation Monitoring Instrumentation and Protective Equipment

Audits and Self Assessments

CR 04-08739 Instrument Setpoints for Contamination Control
CR 04-11611 2004 Thermo Electron User's Group Meeting for Radiation Protection Instrumentation and Dosimetry Assessment
CR 05-06702 Electronic Personal Dosimetry Failures during 1RE12
National Voluntary Laboratory Accreditation Program Assessment

Calibration Forms and Work Authorization Numbers

EQ ID: 400-00041-013, 400-00041-072, 400-00044-015, 400-00044-022, 400-00044-024, 400-00044-026, 400-00061-013, 400-00061-021, 400-00061-026, 400-00061-029, 400-00097-002, 400-00097-011, 400-00097-015, 400-00097-023, 400-00097-042, 400-00099-005, 400-00099-008, 400-00099-009, 400-00099-012, 400-00099-023, 400-00124-121
Work Authorization Numbers: 206549, 212970, 213495, 227239, 249372, 249373, 252362, 256798, 263831, 267729
Whole Body Counter System: Subject ID 999-98-1009 and 999-99-1009 for August 2005

Condition Records

04-10246, 04-12930, 04-13152, 04-13491, 04-14041, 04-14948, 04-16047, 04-16053, 04-16437, 05-00546, 05-02559, 05-03331, 05-04640, 05-04937, 05-06052, 05-06702, 05-07734, 05-08467, 05-09024, 05-09025, 05-09074, 05-09965, 05-10519, 05-11210

Procedures

0PGP03-ZA-0076	Accident Monitoring Instrumentation Maintenance, Revision 3
0PGP03-ZA-0078	Administration of the Radiation Monitoring System, Revision 8
0PGP03-ZA-0128	Medical Examination, Revision 5
0PGP03-ZC-0001	Radiological Instrumentation Control, Revision 8
0PGP03-ZR-0048	Personnel Dosimetry Program, Revision 12
0PGP03-ZR-0050	Radiation Protection Program, Revision 8
0PGP03-ZR-0051	Radiological Access and Work Controls, Revision 21
0PRP04-ZR-0004	Release of Materials From Radiologically Controlled Areas, Revision 13
0PRP02-ZR-0006	TLD Issue and Collection, Revision 7
0PGP03-ZR-0054	Respiratory Protection Program, Revision 12
0PRP06-ZR-0002	Respiratory Protection Equipment Issue and Return, Revision 16
0PRP06-ZR-0005	Maintenance, Inspection, and Storage of Respiratory Protection Equipment Revision 10
0PRP06-ZR-0008	Air Quality Evaluation for Compressors or Pressurized Gas Cylinders, Revision 3
0PRP02-ZR-0007	Evaluation of Intakes, Revision 9
0PRP02-ZR-0010	Personnel Exposure Investigation, Revision 7
0PRP02-ZR-0011	Calibration of WBC System, Revision 3
0PRP07-ZR-0011	Radiological Work ALARA Reviews, Revision 7
0PTP04-ZC-0013	Calibration of Air Samplers/Sampling Pumps, Revision 11

0PTP04-ZC-0002	Calibration of the Eberline 6112 (Series) Teledetector and Fag Kugelfischer Fh40f (Series) Radiameter, Revision 4
0PTP04-ZC-0018	Calibration of Self Reading Dosimeters, Revision 2
0PTP04-ZC-0022	Calibration of Area Alarm Monitors / Underwater Meters, Revision 8
0PTP04-ZC-0036	Calibration of the Eberline AMS-4 Air Monitoring System, Revision 5
0PTP04-ZC-0043	Calibration of Radiological Meters and Ion Chambers, Revision 12
0PTP04-ZC-0044	Calibration of Counting Instruments, Revision 7
0PTP04-ZC-0047	Calibration of the Siemens Electronic Personal Dosemeter, Revision 5
0PTP04-ZC-0049	Calibration of Geiger Counters and Micro R/Hr Meters, Revision 1
	Conduct of Operations for Radiation Protection, Chapter 9, Revision 6

Miscellaneous

2003 and 2004 Radioactive Effluent Release Reports
Licensee Event Report 2005-06
Positive Whole Body Count Logs
South Texas Project - System Health Report, 4th Quarter 2005, Radiation Monitoring (RA)
Walkdown Checklist for Area Monitors
Whole Body Counter Radioactive Source Certificate
Listing of SCBA Certified and Qualified Personnel
Air Quality Records for breathing air compressors
General Employee Training - 003: Respiratory Protection

Section 2PS1: Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems

Documents Reviewed

Procedures

0POP02-WL-0005	Waste Monitor Tank Operations, Revision 14
0POP02-WL-0100	Liquid Waste Release, Revision 11
0PCP09-ZR-0004	Determination of Radionuclides by Gamma Spectroscopy, Revision 17
0PCP09-ZR-0017	Liquid Permit Generation, Revision 13
0PCP07-ZS-0010	Waste Monitor Tank Sampling, Revision 2
0PSP05-RA-8010A	Unit Vent Particulate and Iodine Effluent Monitor Calibration, Revision 5
0PSP05-RA-8010B	MAB Unit Vent Wide Range Gas Monitor Calibration, Revision 6
0PSP05-RA-8038,	Liquid Waste Processing System No.1 Monitor Calibration, Revision 8
0PSP05-WL-4078,	Plant Liquid Waste Discharge Flow Calibration, Revision 3
0PSP07-WL-LDP1,	Liquid Effluent Permit, Revision 11
0PSP11-HF-0001,	FHB Exhaust Filter Airflow Capacity Test, Revision 8
0PSP11-ZH-0008,	CRE and FHB HVAC In-Place HEPA Filter Leak Test, Revision 13
0PSP11-ZH-0009,	EAB and FHB HVAC In-Place Adsorbent Leak Test, Revision 19

Condition Reports

04-10578, 04-13075. 04-13150, 04-13339, 04-14566, 04-14663, 04-10678, 04-11655, 04-11733, 04-15311, 04-15880, 04-16389, 05-01197, 05-03214, 05-05001, 05-05037, 05-05215, 05-05218, 05-06218, 05-06294, 05-06295, 05-06880, 05-07956, 05-08027, 05-010417, 05-11164, 05-12157, 05-14769, 05-15266, 05-15431, 05-15946, 06-00940, 06-01271

Audits and Surveillance

Chemistry and Offsite Dose Calculation Manual June 2004 to June 2005 Annual Evaluation
Quality Audit 04-08(OD), Offsite Dose Calculation Manual

Quality Monitoring Reports

MN-05-0-6647	MN-05-1-6701	MN-05-0-7835	MN-05-0-9263
MN-05-0-11447	MN-05-0-11538		

Monitor Calibrations and Surveillance

Unit Vent Radiation Monitors Source Check (31784799)
Unit Vent Radiation Monitors Source Check (31900981)
Unit Vent Radiation Monitors Source Check (31910718)
Unit Vent Particulate and Iodine Effluent Monitor Calibration (31855854)
Unit Vent Particulate and Iodine Effluent Monitor Calibration (31855891)
MAB Unit Vent Wide Range Gas Monitor Calibration (31780321)
Plant Liquid Waste Discharge Flow Calibration (31757618)
Liquid Waste Processing System No. 1 Monitor Calibration (31885865)

Other Documents

4th Quarter Measurement Assurance Program Sample Results 2004
2nd Quarter Measurement Assurance Program Sample Results 2005
3rd Quarter Measurement Assurance Program Sample Results 2006

System Health Report - Radwaste Systems, 4th quarter 2005
Release permits with liquid effluent monitor inoperable: 2WLDP845 and 2WLDP846

Section 2PS2: Radioactive Material Processing and Transportation

Corrective Action Documents (Condition Reports)

04-14179
05-09411
05-13645
05-14669
05-16331

Procedures

0POP03-ZA-0017	Radioactive Waste Process Control Program, Revision 5
0POP02-WS-0002	High Integrity Container (HIC) Dewatering, Revision 9
0PRP03-ZR-0001	Determination of Radioactive Material Curie Content, Revision 8
0PRP03-ZR-0002	Radioactive Waste Shipments, Revision 17
0PRP03-ZR-0009	10CFR61 Sampling and Analysis Program, Revision 6
0PRP03-ZR-0011	Shipment of Radioactive Material, Revision 12

Records

Waste Shipments of Type A, Type B, and LSA Materials.

Section 2PS3: Radiological Environmental Monitoring Program and Radioactive Material Control Program

Corrective Action Documents (Condition Reports)

04-04266, 05-30040, 05-03164, 05-03214, 05-04898, 05-14065, 05-14345

Audits and Self-Assessments

Self-Assessment/Benchmarking Report - RETS/REMP Workshop 2005
Audit Frequency Extension - Radiological Environmental Monitoring Program (File No.: Q54)

Procedures

0PGP03-ZR-0039, Radiological Environmental Monitoring Program, Revision 12
0PGP03-ZR-0053, Radioactive Material Control Program, Revision 11
0PRP04-ZR-0004, Release of Materials From Radiologically Controlled Areas, Revision 13
0PRP10-ZL-0023, REMP Interlaboratory Comparison Program, Revision 10
0PRP10-ZA-0003, Operation and Calibration of the Radiological Laboratory's Gamma Counting Systems, Revision 12
0PRP10-ZU-0001, REMP Sample Collection, Revision 5
0PRP10-ZU-0007, Environmental TLD Monitoring, Revision 10
0PSP05-EM-0001, Primary Meteorological System Calibration (60 Meter Tower)
0PSP05-EM-0002, Secondary Meteorological System Calibration (10 Meter Tower)

Standing Orders

No. 27 Maintenance Calibration of SAM Series Small Article Monitors, Revision 1
No. 30 Maintenance Calibration of Eberline PM-7 Portal Monitors, Revision 0
No. 31 Maintenance Calibration of Eberline PCM-1C Personnel Contamination Monitors, Revision 0

Miscellaneous

2004 Environmental Operating Report
Offsite Dose Calculation Manual, Revision 12