June 28, 2000

Mr. M. Reddemann
Site Vice President
Point Beach Nuclear Plant
Wisconsin Electric Power Company
6610 Nuclear Road
Two Rivers, WI 54241

SUBJECT: POINT BEACH - NRC INSPECTION REPORT 50-266/2000008(DRS); 50-301/2000008(DRS)

Dear Mr. Reddemann:

On June 9, 2000, the NRC completed a routine inspection at your Point Beach Nuclear Plant. The results of this inspection were discussed on June 8, 2000, with yourself and other members of your staff. The enclosed report presents the results of this inspection.

The inspection was an examination of activities conducted under your license as they relate to radiation safety and to compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection consisted of a selective examination of representative records, tours of your facility and interviews with personnel. Specifically, this inspection focused on occupational radiation safety.

Based on the results of this inspection, no findings were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

We will gladly discuss any question you have concerning this inspection.

Sincerely,

/RA/ by James E. Foster Acting for

Gary L. Shear, Chief
Plant Support Branch
Division of Reactor Safety

Docket Nos. 50-266; 50-301
License Nos. DPR-24; DPR-27

Enclosure: Inspection Report 50-266/2000008(DRS);
50-301/2000008(DRS)

cc w/encl: R. Grigg, President and Chief
Operating Officer, WEPCo
M. Sellman, Senior Vice President,
Chief Nuclear Officer
R. Mende, Plant Manager
J. O'Neill, Jr., Shaw, Pittman,
Potts & Trowbridge
K. Duveneck, Town Chairman
Town of Two Creeks
B. Burks, P.E., Director
Bureau of Field Operations
J. Mettner, Chairman, Wisconsin
Public Service Commission
S. Jenkins, Electric Division
Wisconsin Public Service Commission
State Liaison Officer

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos:	50-266; 50-301
License Nos:	DPR-24; DPR-27
Report No:	50-266/2000008(DRS); 50-301/2000008(DRS)
Licensee:	Wisconsin Electric Power Company
Facility:	Point Beach Nuclear Plant, Units 1 and 2
Location:	6610 Nuclear Road Two Rivers, WI 54241
Dates:	June 5-9, 2000
Inspector:	D. Nelson, Radiation Specialist
Approved by:	Gary L. Shear, Chief, Plant Support Branch Division of Reactor Safety

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas) reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety	Radiation Safety	Safeguards
● Initiating Events	● Occupational	● Physical Protection
● Mitigating Systems	● Public	
● Barrier Integrity		
● Emergency Preparedness		

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: http://www.nrc.gov/NRR/OVERSIGHT/index.html.

SUMMARY OF FINDINGS

Point Beach Nuclear Plant, Units 1 & 2
NRC Inspection Report 50-266/2000008(DRS); 50-301/2000008(DRS)

The report covers a one week period of announced inspection by a regional radiation specialist. This inspection focused on occupational radiation safety and included a review of the access control program, and as-low-as-is-reasonably-achievable (ALARA) planning and controls in conjunction with the October 16, 1999 - December 10, 1999, Unit 1 Refueling Outage (U1R25). In addition, the inspector reviewed the licensee's performance indicator data collecting and reporting process.

The significance of issues is indicated by their color (green, white, yellow, red) and was determined by the Significance Determination Process in Inspection Manual Chapter 0609.

RADIATION SAFETY

Cornerstone: Occupational Radiation Safety

- No findings were identified during this inspection.

Report Details

2. RADIATION SAFETY

Cornerstone: Occupational Radiation Safety

2OS1 Access Control

.1 Plant Walkdowns and Radiological Boundary Verifications

a. Inspection Scope

The inspector performed walkdowns of the radiologically controlled area (RCA) to verify the adequacy of radiological boundaries and postings. Specifically, the inspector performed confirmatory radiation measurements in the Unit 1 and Unit 2 Containment Building as well as the Auxiliary, Turbine, and Radwaste Buildings to verify that radiologically significant work areas (high radiation areas [HRAs] and radiation areas) were properly posted and controlled in accordance with 10 CFR Part 20 and the licensee's procedures.

b. Issues and Findings

No findings were identified during this inspection.

.2 Reviews of Radiation Work Permits

a. Inspection Scope

The inspector reviewed selected routine radiation work permits (RWPs) and electronic dosimeter (ED) alarm set points for both dose rate and accumulated dose for access to high radiation areas. The inspector verified that adequate work controls were in place to maintain worker exposures as-low-as-is-reasonably-achievable (ALARA).

b. Issues and Findings

No findings were identified during this inspection.

.3 Reviews of Licensee's Programmatic Controls for Highly Activated/Contaminated Materials

a. Inspection Scope

The inspector toured the Unit 1/ Unit 2 spent fuel pool (SFP) to verify that all highly activated/contaminated materials are stored in a box at the bottom of the SFP. The inspector also discussed the licensee's programmatic controls over the storage box with the Radiation Protection Manager.

b. Issues and Findings

No findings were identified during this inspection.

.4 Reviews of Radiologically Significant Work

a. Inspection Scope

The inspector reviewed the conduct of work activities in the RCA and Unit 1 containment during Refueling Outage U1R25 that were identified as having higher job dose estimates. Specifically, the inspector verified the adequacy of radiological controls (e.g. radiation work permits and surveys, and ALARA pre-job briefings for the following work activities):

- Steam Generator Platform Upgrades;
- Conoseal Removal/Installation;
- Reactor Coolant Pump Seal Maintenance;
- Replace Proximity Switches in Unit 1 Lower Cavity;
- Replace Defective Top Nozzles
- Miscellaneous Hot Valve Work;
- Steam Generator Sludge Lancing;
- Inservice Inspection Activities.

b. Issues and Findings

No findings were identified during this inspection.

.5 High Risk Significant, High Radiation Worker Performance

a. Inspection Scope

There were no work evolutions being performed during the inspection that were estimated to result in collective worker exposures greater than one person-rem. The inspector did observe the radiological control practices of personnel working within posted radiation areas.

b. Issues and Findings

No findings were identified during this inspection.

2OS2 As-Low-As-Is-Reasonably-Achievable (ALARA) Planning and Controls

.1 Review of Radiologically Significant Work Activities

a. Inspection Scope

The inspector reviewed the formal ALARA reviews for work activities in the RCA and containment during Refueling Outage U1R25 that were expected to result in significant radiological exposures to workers for the following work activities:

- Steam Generator Platform Upgrades;
- Conoseal Removal/Installation;
- Reactor Coolant Pump Seal Maintenance;
- Replace Proximity Switches in Unit 1 Lower Cavity;
- Replace Defective Top Nozzles;
- Miscellaneous Hot Valve Work;
- Steam Generator Sludge Lancing;
- Inservice Inspection Activities.

b. Issues and Findings

No findings were identified during this inspection.

.2 Radiation Dose Controls and Trending

a. Inspection Scope

The inspector reviewed the licensee's dose goals and dose trending for U1R25, to determine whether the licensee maintained occupational radiation exposure ALARA. The inspector also reviewed dose records to determine that the station's outage compared favorably with other pressurized water reactors.

b. Issues and Findings

No findings were identified during this inspection.

4 OTHER ACTIVITIES

4OA1 Performance Indicator Verification

a. Inspection Scope

The inspector verified the licensee's assessment of its performance indicator (PI) for occupational radiation safety. Specifically, the inspector reviewed historical Condition Reports (CRs) for the first and second quarters of 2000 concerning locked HRA control incidents, to ensure that greater than one rem/hr HRA Technical Specification or 10 CFR 20 non-conformances were properly classified as PIs.

b. Issues and Findings

No findings were identified during this inspection.

4OA2 Identification and Resolution of Problems

a. Inspection Scope

The inspector reviewed Condition Reports associated with technician performance, radiation worker practices, and control of HRAs, initiated since February 2000.

b. Issues and Findings

No findings were identified during this inspection.

4OA5 Performance Indicator Data Collecting and Reporting Process Review (TI 2515/144)

a. Inspection Scope

The inspector evaluated the licensee's performance indicator (PI) data collection and reporting process to determine whether the licensee was appropriately implementing the NRC/Industry guidance, as documented in Nuclear Energy Institute (NEI) 99-02, Revision 0, "Regulatory Assessment Performance Indicator Guide." Since the PI verification program procedure was in draft form, the evaluation consisted solely of interviews with licensee staff members responsible for data acquisition, verification and reporting.

b. Issues and Findings

No findings were identified during this inspection.

4OA6 Management Meetings

.1 Exit Meeting Summary

The inspector presented the inspection results to Mr. Reddemann, and other members of licensee management and staff, in an exit meeting on June 8, 2000. The licensee acknowledged the information and findings presented. No proprietary information was identified by the licensee.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

S. Baker, General Supervisor, Radiation Protection
A. Cayia, Manager, Site Services & Assessment
R. Farrell, Manager, Radiation Protection
F. Flentje, Senior Regulatory Compliance Specialist
J. Knorr, Manager, Regulation & Compliance
J. Lindsay, General Supervisor, Radiation Protection
R. Mende, Plant Manager
M. Reddemann, Site Vice President

NRC

F. Brown, Senior Resident Inspector

LIST OF ITEMS OPENED, CLOSED AND DISCUSSED

Opened

None

Closed

None

LIST OF BASELINE INSPECTION PROCEDURES PERFORMED

The following inspectable-area procedures were used to perform inspections during the report period. Documented findings are contained in the body of the report.

Inspection Procedure		Report Section
Number	Title	
71121-01	Access Control to Radiologically Significant Areas	2OS1
71121-02	ALARA Planning and Controls	2OS2
71151	Performance Indicator Verification	4OA1
2515/144	Performance Indicator Data Collecting and Reporting Process Review	4OA5

LIST OF ACRONYMS USED

ALARA	As-Low-As-Is-Reasonably-Achievable
CR	Condition Report
ED	Electronic Dosimeter
HRA	High Radiation Area
NEI	Nuclear Energy Institute
PI	Performance Indicator
RCA	Radiologically Controlled Area
RWP	Radiation Work Permit
SFP	Spent Fuel Pool

LIST OF DOCUMENTS REVIEWED

The following is a list of licensee documents reviewed during the inspection. Inclusion on this list does not imply that NRC inspectors reviewed the documents in their entirety, but, rather that selected sections or portions of the documents were evaluated as part of the overall inspection effort.

Condition Reports

Condition Reports: 00-0641, 00-0996, 00-1736, 00-1076, 00-1024, 00-1075, 00-1117, 00-1440, 00-1449, 00-1450, 00-1564, 00-1737.

Miscellaneous

1999 Dose 1/1/99 - 12/31/99 Sorted by RWP number ascending;
U1R25 ALARA Performance Summary;
Table 1, "U1R25 Refueling Outage Exposure Summary," 10/16/99 to 12/10/99;
Table 2, "U1R25 Daily Exposure (10/16/99 to 12/10/99;)"
ALARA Review Log.

Radiation Work Permits

RWP No. 99-1-1046, "Conoseal Removal/Installation;"
RWP No. 99-1-1066, "U1 RCP Seal Maintenance;"
RWP No. 99-1-1072, "Replace Defective Top Nozzles;"
RWP No. 99-1-1041, "U1R25 - Radiography;"
RWP No. 99-1-1061, "Outage Activites in HRA/HCAs;"
RWP No. 99-1-1062, "Outage Activities in HRA/HCAs in the PAB and Facade;"
RWP No. 99-1-1065, "S/G Sludge Lancing."

ALARA Reviews

AR No. 99-28, "Steam Generator Platform Upgrades;"
AR No. 99-31, "Conoseal Removal/Installation;"
AR No. 99-32, "RCP Seal Maintenance;"
AR No. 99-34, "ISI/NDE;"
AR No. 99-35, "S/G Sludge Lancing;"
AR No. 99-36, "Replace Proximity Switches in U1 Lower Cavity;"
AR No. 99-37, "Replace Defective Top Nozzles;"

AR No. 99-38, “Miscellaneous Hot Valve Work.”

Procedures

The PI Verification procedure was in draft form and unavailable for formal review.