June 12, 2001

Dr. Robert C. Mecredy
Vice President, Nuclear Operations
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, NY 14649

SUBJECT: R. E. GINNA - NRC INSPECTION REPORT 05000244/2001-003

Dear Dr. Mecredy:

On May 12, 2001, the NRC completed an inspection of your R. E. Ginna facility. The enclosed report documents the inspection findings which were discussed on May 18, 2001, with Mr. J. Widay and other members of your staff.

This inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

No findings of significance were identified.

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Sincerely,

/RA/

Michele G. Evans, Chief
Projects Branch 1
Division of Reactor Projects

Docket No. 05000244
License No. DPR-18

Enclosure: Inspection Report 05000244/2001-003

Attachment (1) Supplementary Information

cc w/encl:
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REGION I

Docket No:	05000244
License No:	DPR-18
Report No:	05000244/2001-003
Licensee:	Rochester Gas and Electric Corporation (RG&E)
Facility:	R. E. Ginna Nuclear Power Plant
Location:	1503 Lake Road Ontario, New York 14519
Dates:	April 1, 2001 through May 12, 2001
Inspectors:	R. A. Fernandes, Acting Senior Resident Inspector C. R. Welch, Resident Inspector L. M. Cline, Division of Reactor Safety T. A. Moslak, Health Physicist S. Dennis, Division of Reactor Safety G. Smith. Senior Physical Security Inspector
Approved by:	M.G. Evans, Chief Projects Branch 1 Division of Reactor Projects

SUMMARY OF FINDINGS

IR 05000244-01-003, on 04/01-05/12/2001; Rochester Gas & Electric; R. E. Ginna Nuclear Power Plant, resident inspector report.

The inspection was conducted by resident inspectors and regional specialists. This inspection identified no significant findings. The significance of most findings is indicated by their color (Green, White, Yellow, or Red) using Inspection Manual Chapter (IMC) 0609, "Significance Determination Process (SDP)." Findings for which the SDP does not apply are indicated by "No Color" or by the severity level of the applicable violation. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described at its Reactor Oversight Process website at http://www/nrc/gov/NRC/ADAMS/index.html.

A. Inspector Identified Findings

No findings of significance were identified.

Report Details

SUMMARY OF PLANT STATUS

Ginna began and ended the period at full power.

1. REACTOR SAFETY
Initiating Events, Mitigating Systems, and Barrier Integrity [Reactor - R]

R04 Equipment Alignment

a. Inspection Scope

The inspectors performed partial walkdowns of the following system trains.

- Off-site AC power circuit Nos. 751 and 767.
- Component cooling water train B.

These inspections verified that key valves and electrical circuit breakers were properly aligned in accordance with station procedures and drawings. During the walkdowns, the inspectors also evaluated the material conditions and general housekeeping of the systems and adjacent spaces. Ginna's updated final safety analysis report, technical specifications, and probabilistic safety assessment were used as references.

b. Findings

No findings of significance were identified.

R05 Fire Protection

a. Inspection Scope

The inspectors toured the following plant areas to assess RG&E's control of combustible materials and ignition sources, and the physical condition of installed fire suppression and detection systems. The inspectors also observed hot work in the auxiliary building intermediate level and verified that the Fire Protection Program and associated implementing procedures were appropriately followed.

- East end auxiliary building basement.
- West end auxiliary building basement.
- East end auxiliary building intermediate level.
- Intermediate building basement clean side.

b. Findings

No findings of significance were identified.

R12 Maintenance Rule Implementation

a. Inspection Scope

The inspectors reviewed RG&E's maintenance rule implementation for the below listed performance problems. This inspection evaluated system scoping, performance criteria/goal monitoring, and problem classification.

- Check valve 9633B failed its full-open verification test, Action Report (AR) No. 2000-0823.
- Alarm switch on 52/RHRP1B failed to close, AR No. 2001-0288.
- Air leak on the turbine driven auxiliary feed pump discharge valve to "B" steam generator, AR No. 2001-0266.

b. Findings

No findings of significance were identified.

R13 Maintenance Risk Assessments and Emergent Work Control

a. Inspection Scope

The inspectors evaluated the effectiveness of RG&E's maintenance risk assessments required by paragraph a(4) of 10 CFR 50.65. This inspection included discussions with control room operators and scheduling department personnel regarding the use of RG&E's online risk monitoring software. The inspectors reviewed equipment tracking documentation, daily work schedules, and performed plant tours to gain reasonable assurance that actual plant configuration matched the assessed configuration. Additionally, the inspectors verified that RG&E's risk management actions, for both planned and emergent work, were consistent with those described in procedure IP-PSH-2, "Integrated Work Schedule Risk Management." Risk assessments for the following out of service systems, structures, and/or components were reviewed:

- On April 10, 2001, 'A' residual heat removal system main control board switch was replaced with containment penetration Nos. 206 and 307 degraded.

- On May 11, 2001, a temporary modification was initiated to install additional cooling for the rod control drive cabinets.

b. Findings

No findings of significance were identified.

R15 Operability Evaluations

a. Inspection Scope

The inspectors reviewed the below listed operability evaluations to determine if system operability was properly justified. This inspection included discussion with plant personnel and reviews of applicable technical specifications and design bases information.

- Technical Support Center diesel generator damper failure, AR No. 2001-0622.
- Containment spray sodium hydroxide air operated addition valve, AR No. 2001-047.

b. Findings

No findings of significance were identified.

R17 Permanent Plant Modifications

a. Inspection Scope

The inspectors reviewed the below listed plant change records (PCRs) to verify that the modifications did not degrade the affected system's design bases, licensing bases, or performance capability. This inspection included reviews of associated engineering evaluations, visual observations of related maintenance activities, reviews of post modification testing, and verification of selected document updates.

- PCR No. 2001-0001, Amptector jumper for ground function noise reduction.
- PCR No. 1998-022, Intermediate building block wall structural modifications.
- Setpoint PCR No. 2000-0002, Accumulator Low Level Alarm Setpoint Change.

b. Findings

No findings of significance were identified.

R19 Post Maintenance Testing

a. Inspection Scope

The inspectors reviewed the post maintenance tests for the following work orders (WOs) to verify that RG&E appropriately demonstrated the components' ability to perform their intended safety function:

- WO No. 20100226, 'A' residual heat removal pump main control board switch replacement.

- WO No. 20100173, Containment fan "D" DB-50 circuit breaker maintenance.

- WO No. 20003293, 'A' auxiliary feed pump discharge flow loop calibration.

- WO No. 2000-0033, Replacement of stop check valves 9633A and 9633B.

b. Findings

No findings of significance were identified.

R22 Surveillance Testing

a. Inspection Scope

The inspectors witnessed the performance and/or reviewed test data for the following activities to verify that the test demonstrated the associated system's functional capability and operational readiness:

- PT-2.8Q, Component Cooling Water Pump Quarterly Test
- PT-12.2, Emergency Diesel Generator B
- PT-36Q-D, Standby Auxiliary Feedwater Pump Quarterly Test

b. Findings

No findings of significance were identified.

R23 Temporary Plant Modifications

a. Inspection Scope

The inspectors reviewed the following temporary modification (TMOD) to verify that the safety functions of the associated system were not affected. This was accomplished by: (1) a review of the TMOD packages; (2) an assessment of the 10 CFR 50.59 evaluation; and (3) a plant walkdown of the installed TMOD to ensure consistency with the TMOD documentation.

- No. 2001-0008 Control Rod Drive Cabinet Fans and Air Conditioning Unit.

b. Findings

No findings of significance were identified.

2. RADIATION SAFETY

Occupation Radiation Safety [OS]

OS1 Access Control to Radiologically Significant Areas

a. Inspection Scope

During the period April 30 through May 3, 2001, the inspector conducted the following activities to verify that the licensee was properly implementing physical and administrative controls for access to locked high radiation areas and other radiologically controlled areas, and that workers were adhering to these controls when working in these areas.

Independent radiation surveys were performed in the radiologically controlled areas (RCA) of the Auxiliary Building and Intermediate Building (Hot Side) to confirm the accuracy of posted survey results, and assess the adequacy of radiation work permits and associated controls. Keys to Technical Specification Locked High Radiation Areas were inventoried and these areas were verified to be properly secured and posted during plant tours.

The inspector attended the ALARA pre-job briefings and reviewed the exposure controls specified in radiation work permits (RWPs) and the associated ALARA Reviews for the following in-progress jobs:

- Reactor Containment Entry During Power Operations, RWP No. 011008, ALARA Review No. 010052
- Removal of Irradiated Material from Canister in Spent Fuel Pool, RWP No. 011025, ALARA Review No. 010100

For these tasks, the inspector interviewed selected workers on their knowledge of the relevant radiation work permit, electronic dosimetry setpoints, and job site radiological conditions.

The inspector reviewed pertinent information regarding cumulative exposure history, current exposure trends, and worker exposure records in order to assess the licensee's effectiveness in establishing exposure goals and in limiting worker dose.

The inspector observed technicians performing functional source checks on a variety of portable survey instruments and contamination counting equipment to verify the operability and accuracy of in-use instruments.

The inspector reviewed selected ARs related to the control of personnel exposure and work activities in radiologically controlled areas to evaluate the licensee's threshold for identifying problems regarding the implementation of the radiation protection program and the promptness and effectiveness of the resulting corrective actions. Additionally, ARs were evaluated against the criteria contained in 10CFR 20, site Technical Specifications, and site procedures to determine the regulatory significance of the

identified problem. Included in this review were AR Nos. 2001-0093, 2001-0105, 2001-0121, 2001-0147, 2001-0191, 2001-0215, 2001-0250, 2001-0422, 2001-0467, and 2001-0484.

In evaluating the effectiveness of the licensee's problem identification and resolution program, the inspector reviewed the following Radiation Protection Department self-assessments and critique of performance issues identified during the past refueling outage:

- Self-Assessment 2000-0013, Internal Dosimetry
- Self-Assessment 2001-0014, Refueling Outage 2000 Dosimetry Evaluation
- Self-Assessment 2000-0016, Electronic Dosimeter Dose Rate Alarms
- Self-Assessment 2000-0017, Electronic Dosimeter Dose Alarms
- Refueling Outage 2000, Radiation Protection Department Performance Critique

b. Findings

No findings of significance were identified.

3. SAFEGUARDS

3PP4 Security Plan Changes

a. Inspection Scope

An in-office review was conducted of changes to the Physical Security Plan, identified as Revision T, submitted to the NRC on October 5, 2000, in accordance with the provisions of 10 CFR 50.54(p). The review was conducted to confirm that the changes were made in accordance with 10 CFR 50.54(p), and that the changes did not decrease the overall effectiveness of the plan.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES [OA]

OA1 Performance Indicator Verification

a. Inspection Scope

The inspectors reviewed the monthly operating reports, Licensee Event Reports, and NRC inspection reports from April 2000 through March 2001 to assess the accuracy and completeness of the following initiating event cornerstone performance indicators (PIs), reported for the first quarter 2001:

- Unplanned scrams per 7,000 critical hours.
- Scrams with a loss of normal heat removal.

- Unplanned power changes per 7,000 critical hours.

The inspectors also reviewed RG&E's safety system unavailability determination for Action Report 2001-0059, "Failure of Motor Operated Valve (MOV) 857B to Operate."

b. Findings

No findings of significance were identified.

The inspectors noted that RG&E's determination that the failure of MOV 857B to operate resulted from a "design deficiency" versus poor maintenance. Accordingly, per the Nuclear Energy Institute (NEI) Regulatory Assessment Performance Indicator Guideline (NEI 99-02), RG&E excluded approximately 330 hours of fault exposure time (unavailability) from the associated mitigating systems cornerstone PI (High Pressure Injection System Unavailability). Although the excluded fault exposure hours were considerable, including them would not have changed the PI from Green to White. If included, the PI would shift to approximately the middle of the Green band.

OA6 Meetings

a. Exit Meeting Summary

On May 18, 2001, the inspectors presented their overall findings to members of RG&E management led by Mr. J. Widay. RG&E management acknowledged the findings presented and the inspectors' conclusions. No proprietary information was identified.

ATTACHMENT 1

PARTIAL LIST OF PERSONS CONTACTED

RG&E

J. Widay	VP, Plant Manager
P. Bamford	Primary Systems and Reactor Engineering Manager
R. Biedenbach	Safety/Fire Coordinator
M. Flaherty	Configuration Support Manger
B. Flynn	Scheduling Manager
G. Graus	I&C/Electrical Engineering Manager
J. Hotchkiss	Mechanical Maintenance Manager
G. Joss	ISI/IST Coordinator
F. Mis	Acting Radiation Protection and Chemistry Manager
T. Plantz	Maintenance Systems Manager
R. Ploof	Balance of Plant Systems Engineering Manager
P. Polfleit	Corporate Emergency Planner
R. Popp	Production Superintendent
J. Smith	Maintenance Superintendent
J. Wayland	I&C/Electrical Maintenance Manager
T. White	Operations Manager
G. Wrobel	Nuclear Safety & Licensing Manager
J. Bement	Foreman, Radiation Protection
G. Combs	Radiation Protection Technician
K. Cona	System Engineer
K. Gould	Senior Health Physicist
A. Herman	Senior Health Physicist
N. Leoni	Quality Assessment Coordinator
K. Magnuson	ALARA Technician
B. Meighan	ALARA Coordinator
P. Perry	Mechanical Maintenance Technician
J. St. Martin	Licensing Engineer
J. Traynor	Quality Assurance Analyst, Independent Assessment

LIST OF ACRONYMS USED

ALARA	As Low As is Reasonably Achievable
AR	Action Report
CFR	Code of Federal Regulations
MC	Manual Chapter
MOV	Motor-Operated Valve
NEI	Nuclear Energy Institute
NRC	Nuclear Regulatory Commission
PCR	Plant Change Record
PI	Performance Indicator
RCA	Radiologically Controlled Area
RG&E	Rochester Gas and Electric Corporation
RWP	Radiation Work Permit
TMOD	Temporary Modification
WO	Work Order

LIST OF ADDITIONAL DOCUMENTS REVIEWED

Procedure A-1, Radiation Control Manual, Revision 59
Procedure A-1.8, Radiation Work Permits, Revision 10
Procedure A-1.1, Access Control to Locked High Radiation and Very High Radiation Areas, Revision 37
Procedure A-1.6.1, ALARA Job Reviews, Revision 24
Self-Assessment 2000-0013, Internal Dosimetry
Self-Assessment 2001-0014, Refueling Outage 2000 Dosimetry Evaluation
Self-Assessment 2000-0016, Electronic Dosimeter Dose Rate Alarms
Self-Assessment 2000-0017, Electronic Dosimeter Dose Alarms
Refueling Outage 2000, Radiation Protection Department Performance Critique