



# New Advances in Computing TBR and Application to ARIES Power Plants

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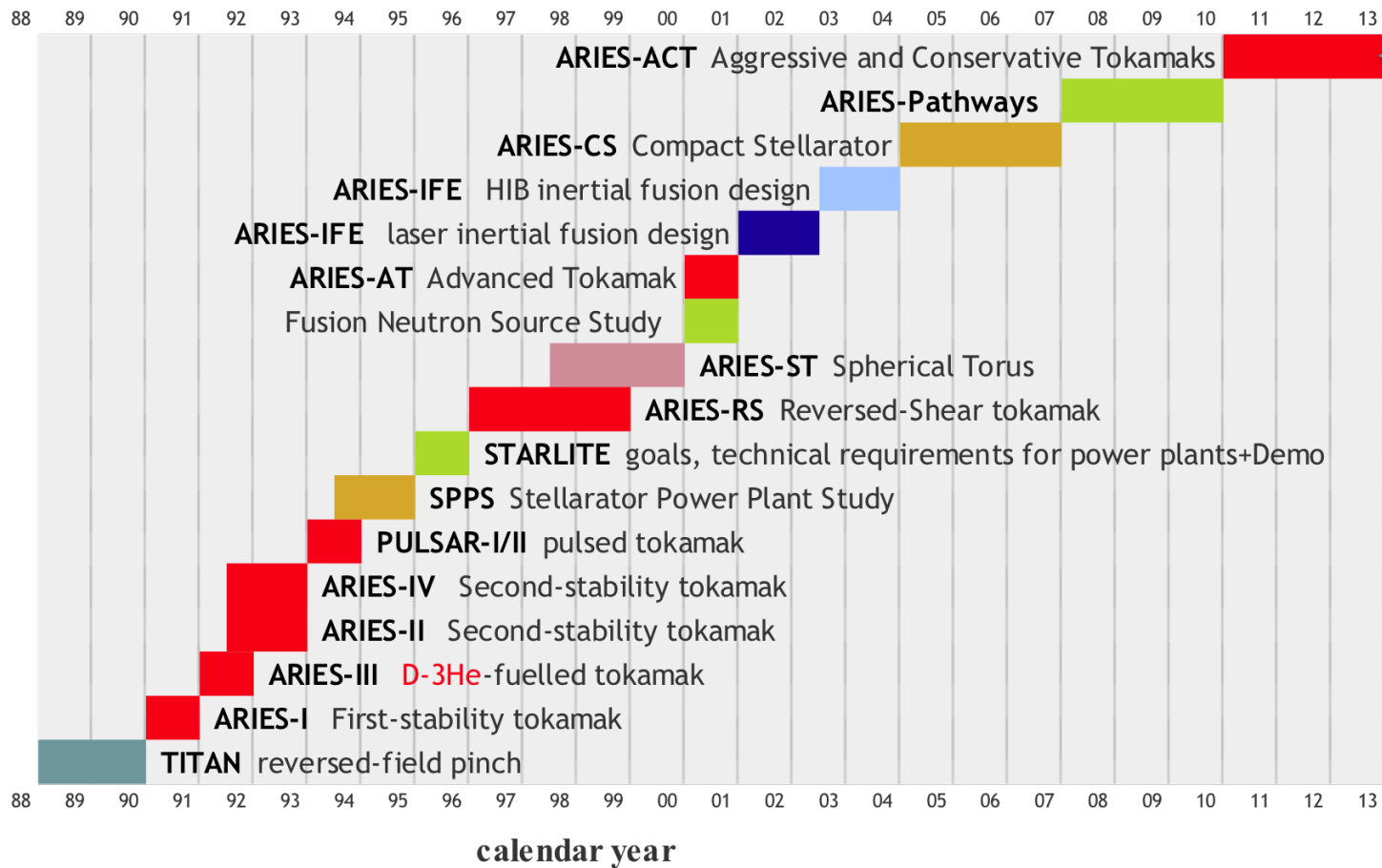
<http://fti.neep.wisc.edu/UWNeutronicsCenterOfExcellence>

**VLT Conference Call  
September 19, 2012**



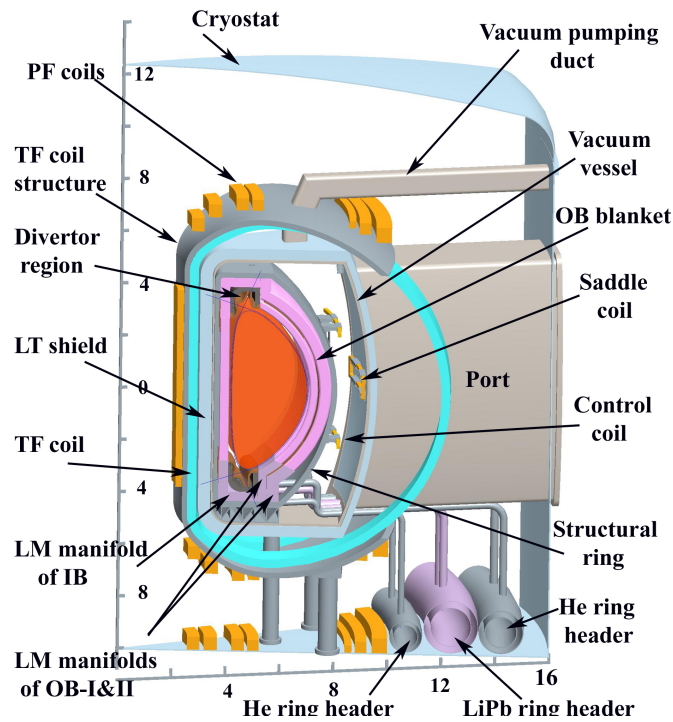
# ARIES Designs (1988 – 2012)

■ Tokamak (8) ■ Stellarator (2) ■ Laser (1) ■ Spherical Torus (1) ■ RFP (1) ■ Heavy Ion Beam (1) ■ Other (3)

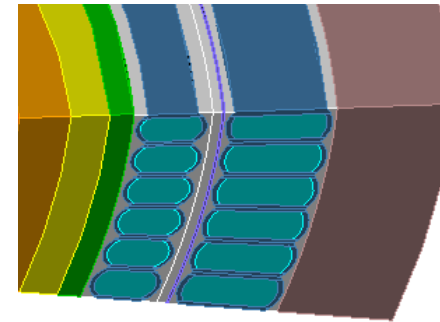




# ARIES-ACT Design

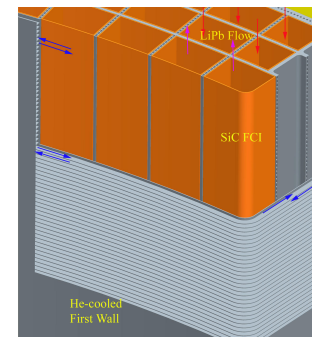


## Two Blanket Designs



SiC/LiPb

$\eta_{th} \sim 60\%$



DCLL

$\eta_{th} \sim 45\%$

$R = 5.5$  m;  $a = 1.375$  m;  $A = 4$ ;  $P_f \sim 1800$  MW; 16 TF magnets; 16 Toroidal modules; **SiC/LiPb blanket**.  
No blanket behind divertor (only LiPb manifolds for inboard blanket).

**ARIES breeding requirements\***: calculated TBR = 1.05 with  ${}^6\text{Li}$  enrichment < 90%.

• L. El-Guebaly, A. Jaber and S. Malang, "State-of-the-Art 3-D Assessment of Elements Degrading the TBR of the ARIES DCLL Blanket," Fusion Science and Technology 61, #4 (May 2012) 321-331.



# We Addressed Several Breeding-Related Questions that Puzzled Fusion Community for Decades

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## Breeding-related questions:

- How does blanket **structure** (first wall, side, and back walls, cooling channels, etc.) degrade TBR?
  - Which change to blanket **thickness and/or Li enrichment** is more enhancing to TBR?
  - How does advanced physics (that requires embedding **stabilizing shells** within blanket) degrade breeding?
  - Could required **TBR be achieved** in presence of several design elements (such as plasma heating and current drive **ports**) that compete for best available space for breeding?
  - Does blanket offer **flexible approach** to handle any shortage and surplus of T?
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- **Past studies** answered some questions by addressing **individual issues** – one at a time.
  - **Our state-of-the-art 3-D analysis** examined all questions **collectively in integral fashion** to account for inter-dependence and synergistic effects.



# Questions Addressed with Sophisticated 3-D Neutronics Codes

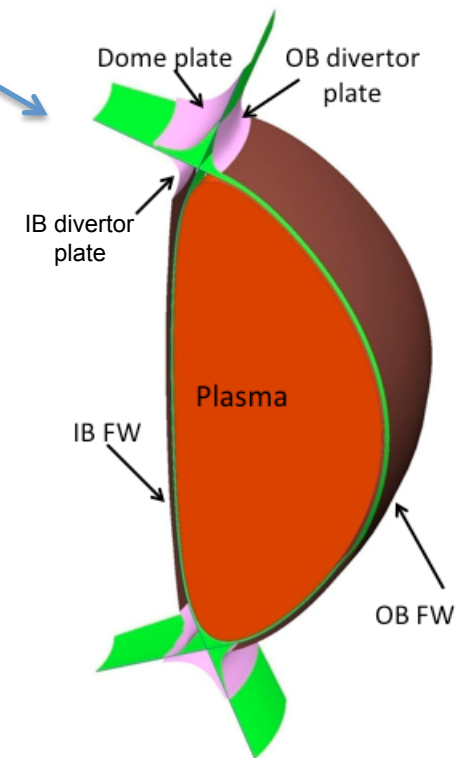
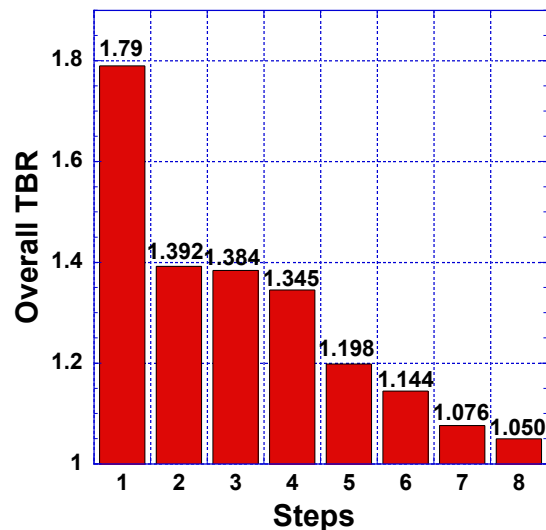
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- UW Computational Nuclear Engineering Research Group (**CNERG**) developed **most innovative computational tool** in recent years.
- **DAGMC code** permits fully accurate modeling of complex devices by integrating CAD geometry directly with 3-D MCNP code.
- To point out terms that contribute to decrease/increase in TBR, we also developed a **novel stepwise approach** that allows adding various blanket components “step-by-step.”
- This unique capability allows **fully accurate presentation** of blanket geometry with **high fidelity in 3-D TBR results**. Note that 1% less TBR means T shortage of ~1 kg/y, costing \$30-100M to purchase annually from external sources.



# Stepwise Approach

- Build CAD model from scratch, starting with FW/divertor skeleton
- Couple CAD with MCNP using DAGMC code
- No homogenization within breeding zones
- Model each individual component using CAD, import CAD model into neutronics code, then add in multiple steps:
  - FW and other walls for blanket
  - Other design elements (shield, assembly gaps, stabilizing shells, penetrations, etc.)
- Record impact of 7 individual design elements on TBR
- Vary Li enrichment from natural to 90% to determine operating enrichment.



ARIES-ACT-SiC  
One module (22.5°)

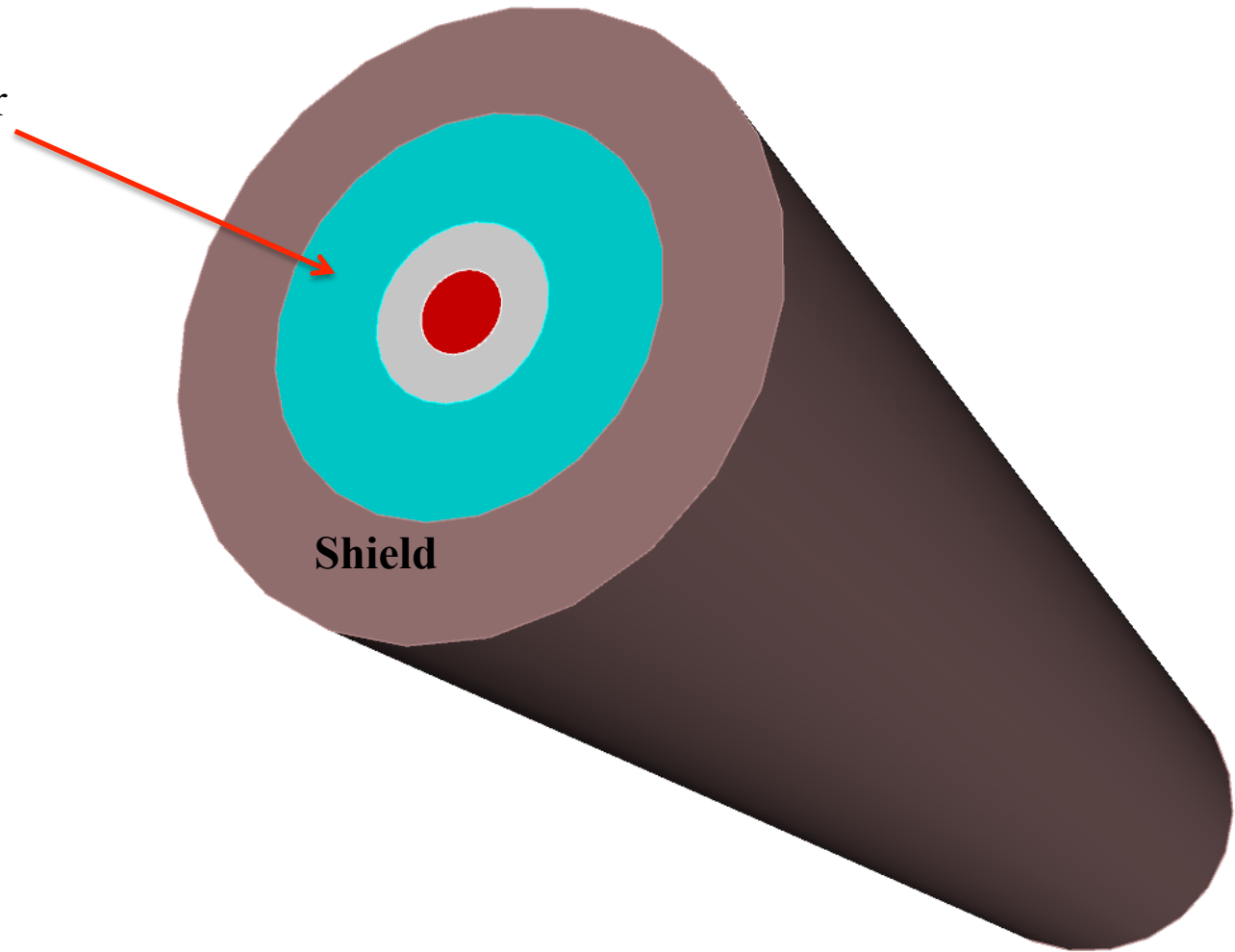
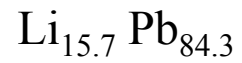


# 1-D Infinite Cylinder

(to estimate maximum achievable TBR for 100 % LiPb; 90% Li enrichment; no structure)

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2 m thick LiPb Breeder



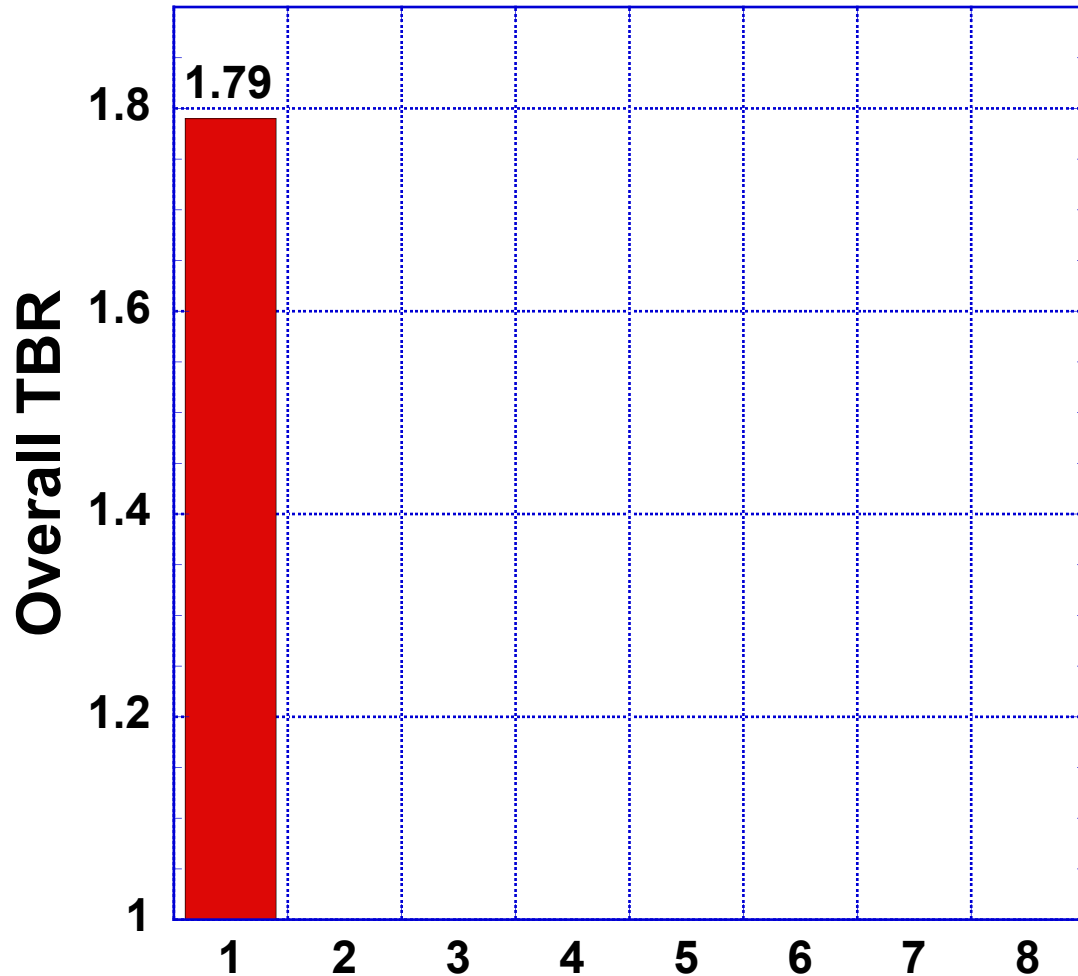


# 1-D Infinite Cylinder

(to estimate maximum achievable TBR for 100 % LiPb; 90% Li enrichment; no structure)

## 90% Enriched Li-6

1. 1-D infinite Cylinder: 100% LiPb breeder surrounded with FS shield

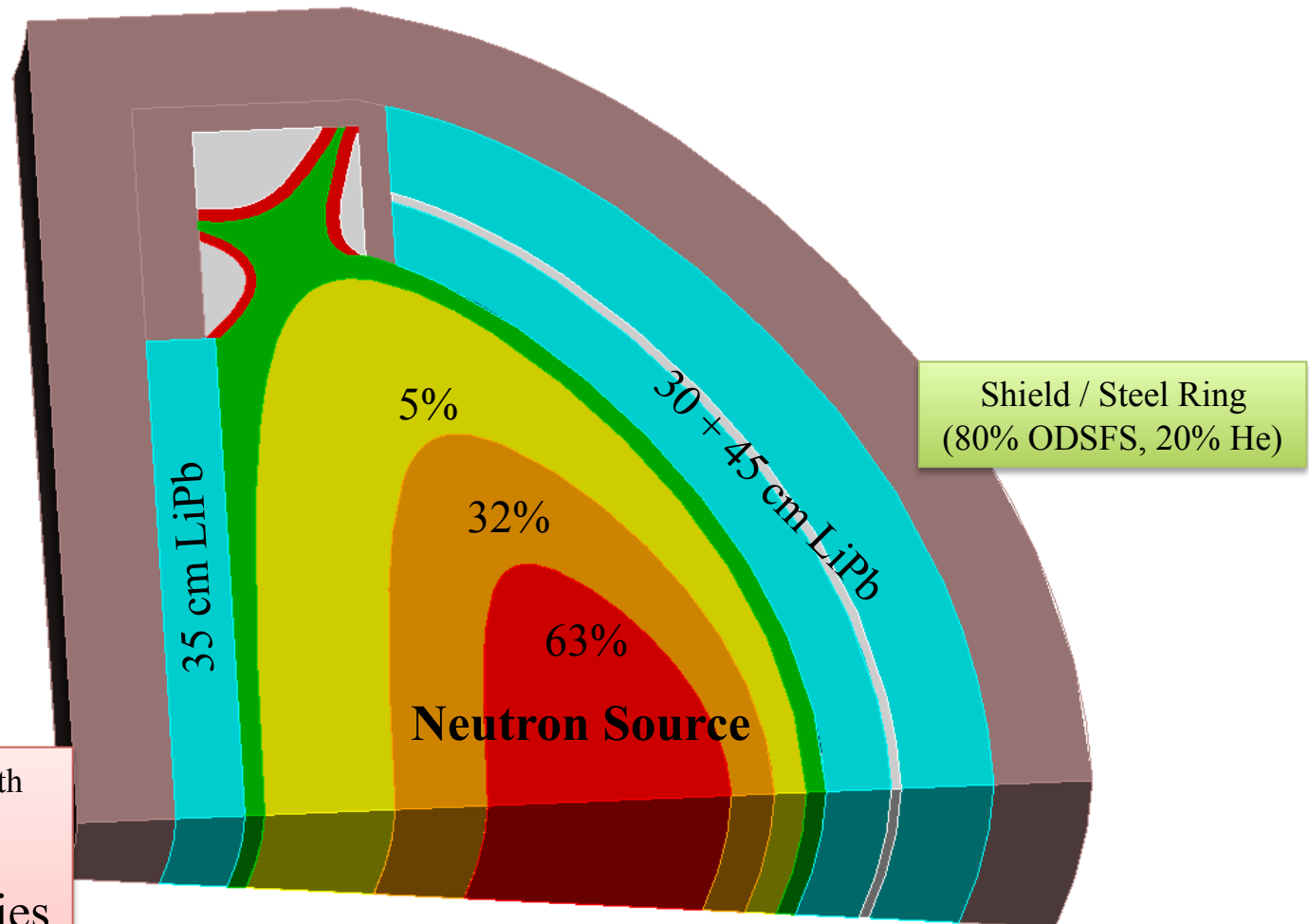


← Required Calculated TBR = 1.05





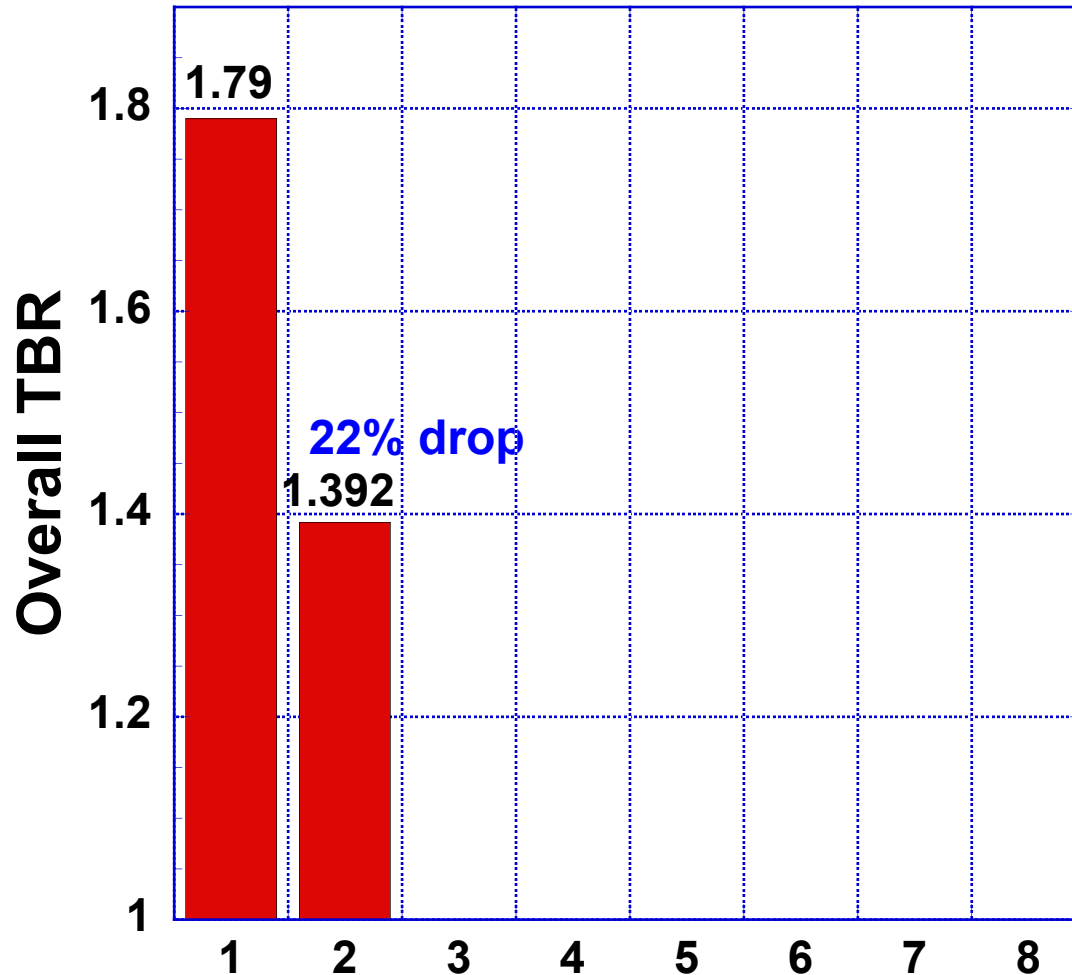
## 3-D Toroidal Model: $\text{Li}_{15.7}\text{Pb}_{84.3}$ Confined Radially/ Vertically to Blanket. Shield and Divertor Added



Upper half of 1/32<sup>th</sup>  
module with three  
reflecting boundaries  
at both sides and at  
midplane



## 3-D Toroidal Model: $\text{Li}_{15.7}\text{Pb}_{84.3}$ Confined Radially/ Vertically to Blanket. Shield and Divertor Added



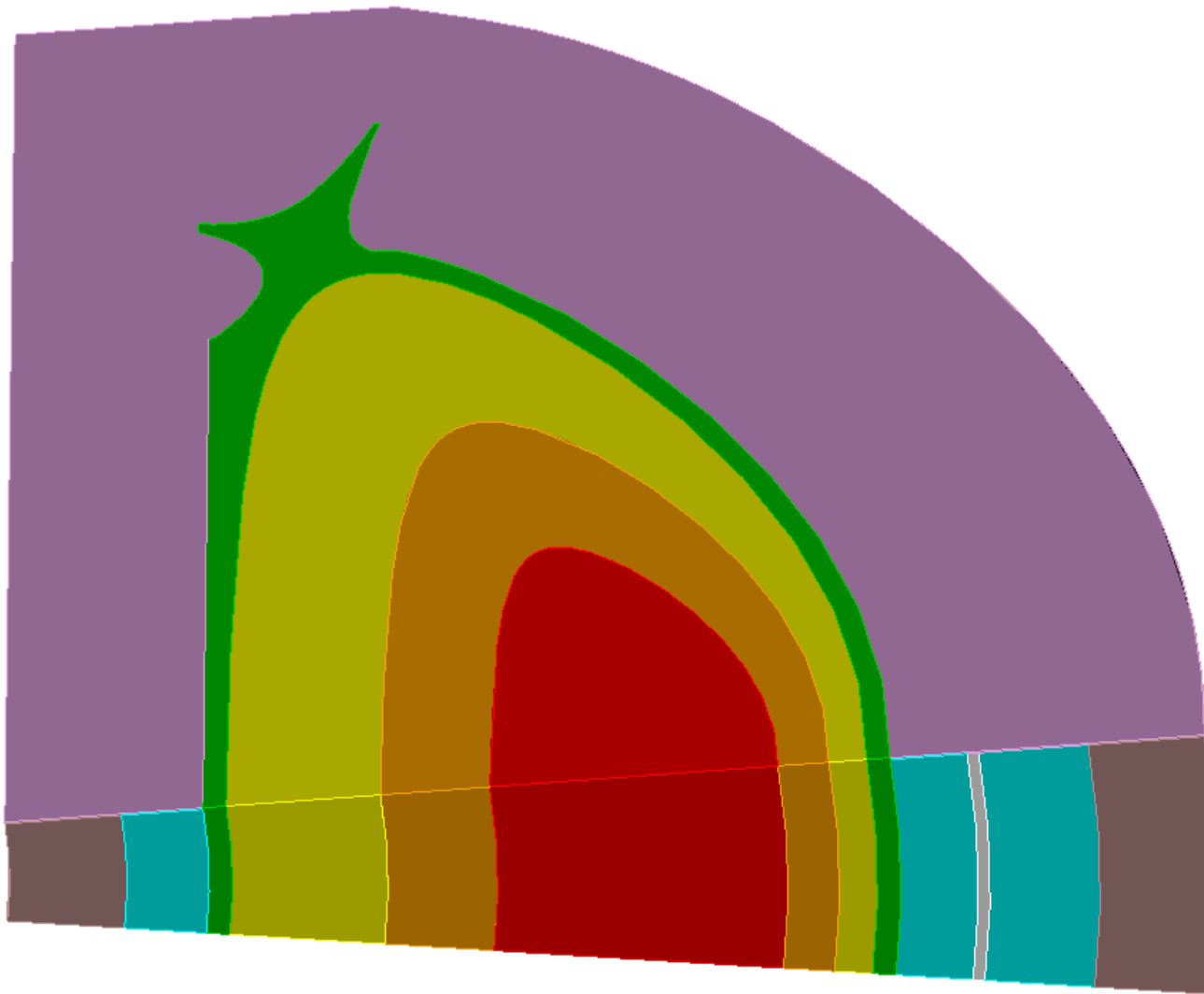
### 90% Enriched Li-6

1. 1-D infinite Cylinder: 100% LiPb breeder surrounded with FS shield
2. 3-D Toroidal Model: LiPb confined to 35 cm IB blanket and 30+45 cm OB blanket



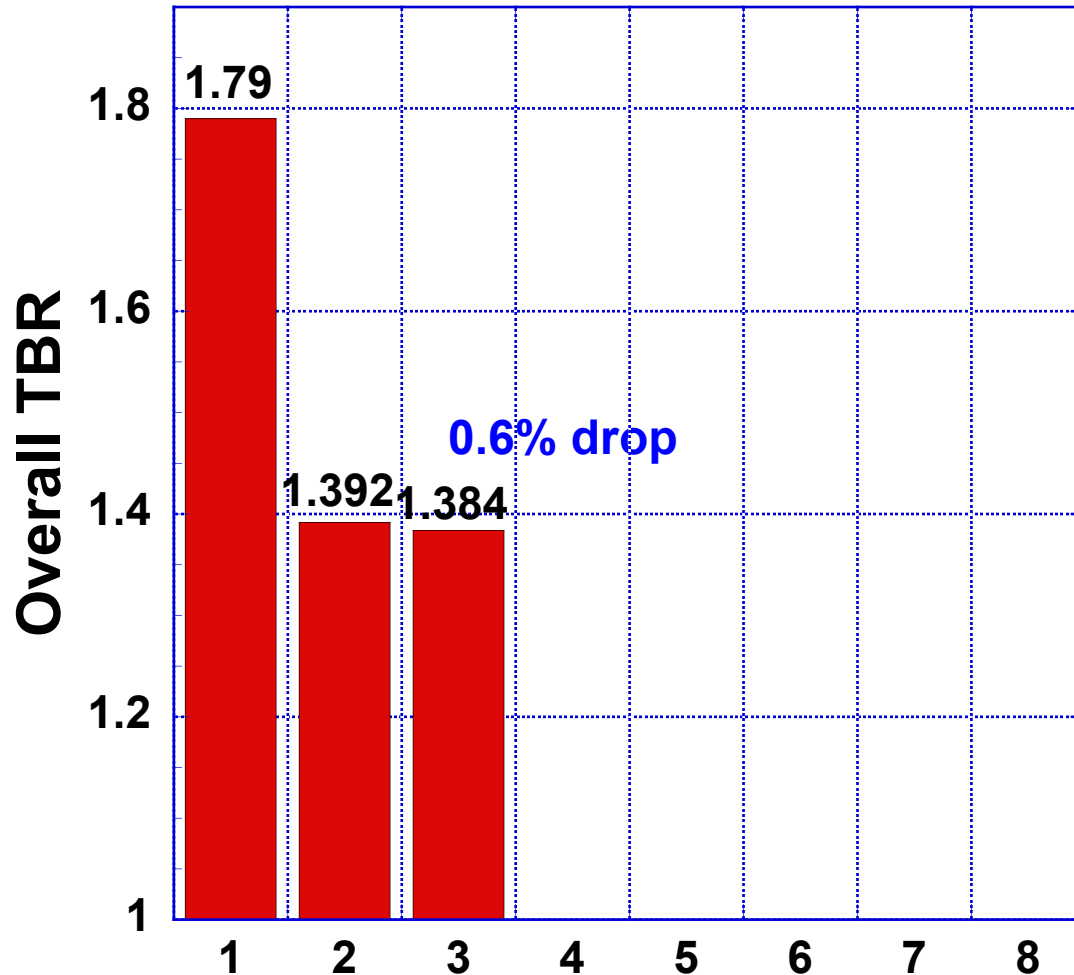
# 2 cm Wide Assembly Gaps Between Modules (purple)

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# 2 cm Wide Assembly Gaps Between Modules

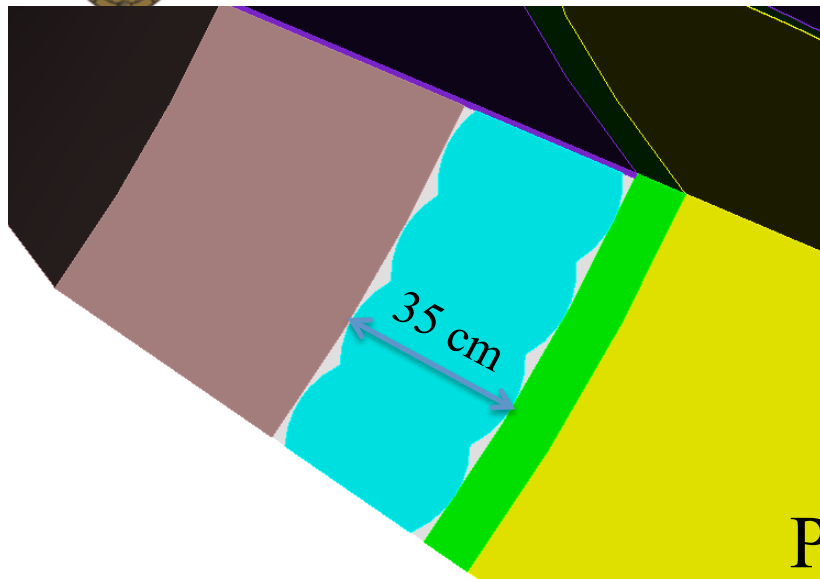


## 90% Enriched Li-6

1. 1-D infinite Cylinder: 100% LiPb breeder surrounded with FS shield
2. 3-D Toroidal Model: LiPb confined to 35 cm IB blanket and 30+45 cm OB blanket
3. Add **assembly gaps** between blanket modules



# Segment Blankets into Sectors and Curve FW and BW of each Sector



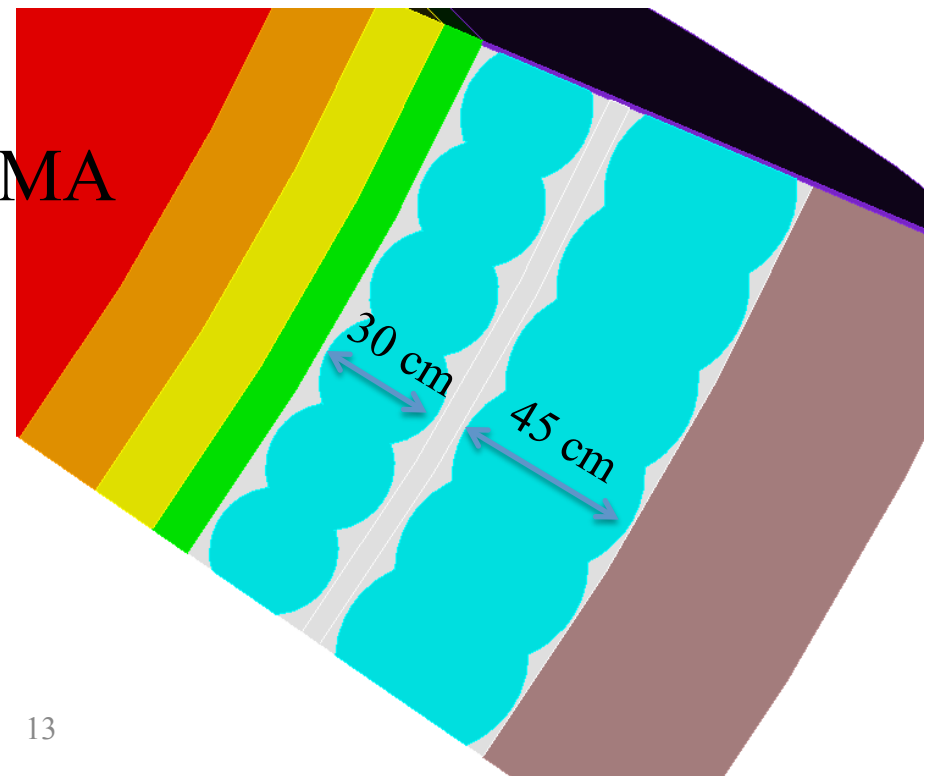
**IB**

Maximum radial blanket thickness = 35 cm

PLASMA

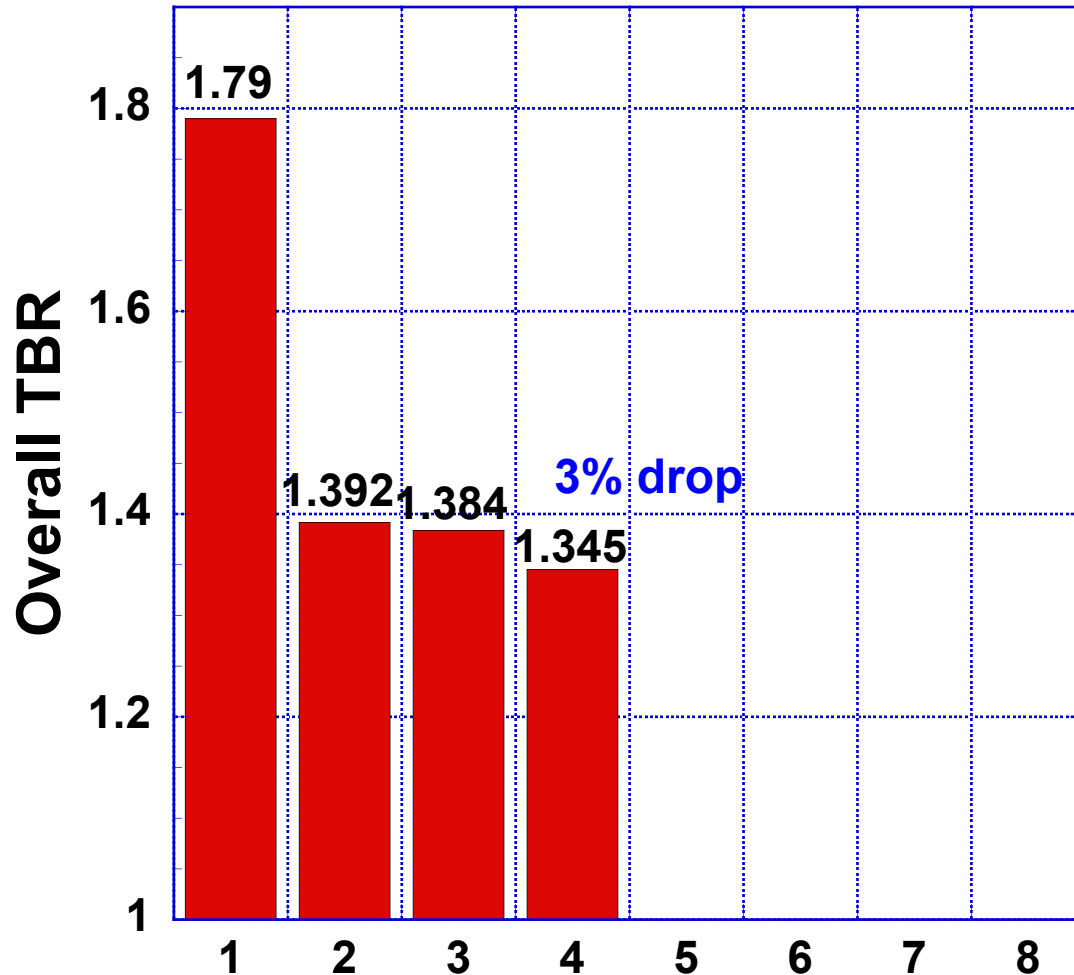
**OB**

Max OB-I radial blanket thickness = 30 cm  
Max OB-II radial blanket thickness = 45 cm





# Segment Blankets into Sectors and Curve FW and BW of each Sector



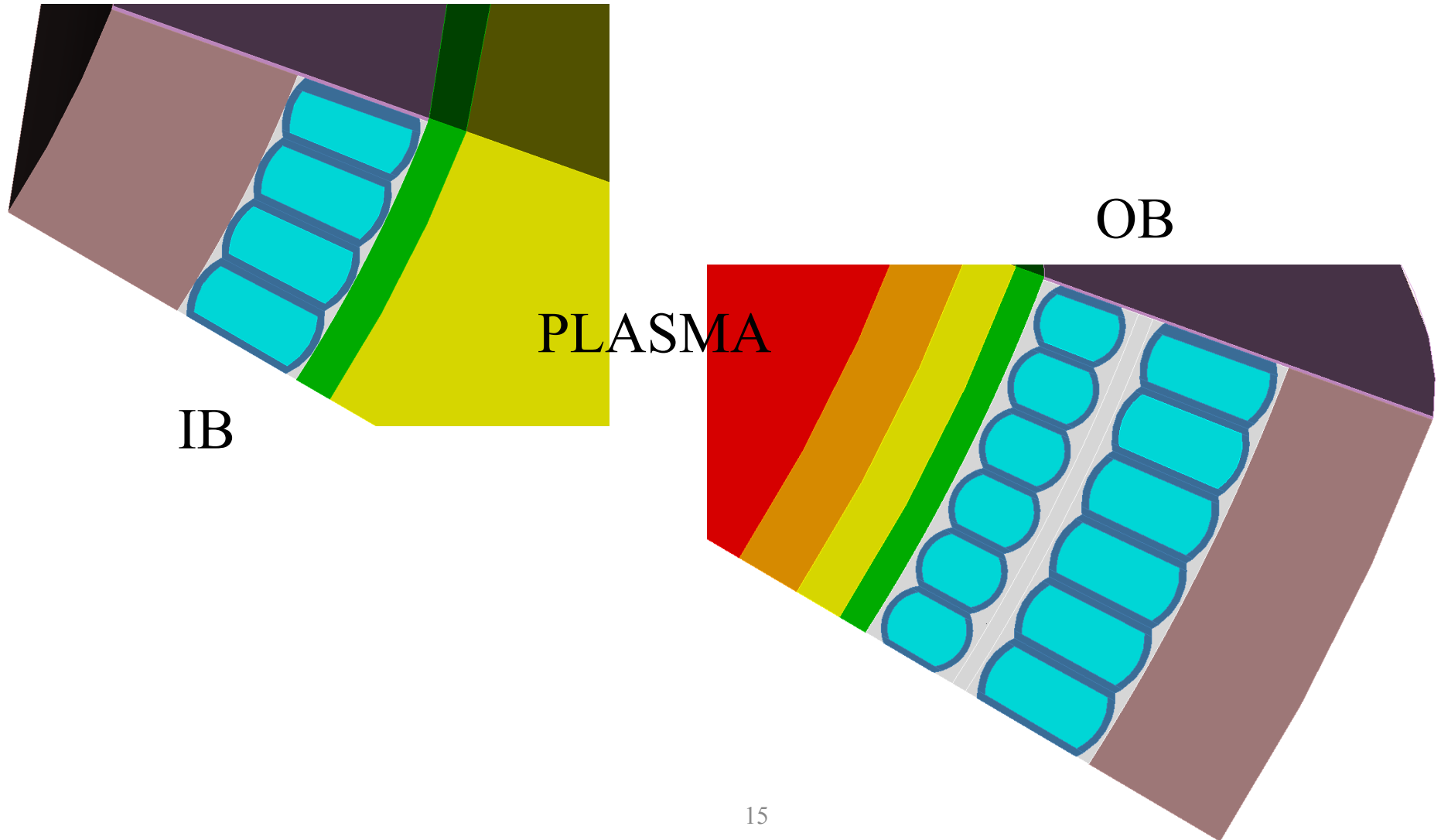
## 90% Enriched Li-6

1. 1-D infinite Cylinder: 100% LiPb breeder surrounded with FS shield
2. 3-D Toroidal Model: LiPb confined to 35 cm IB blanket and 30+45 cm OB blanket
3. Add assembly gaps between blanket modules
4. **Curve** IB and OB blanket sectors



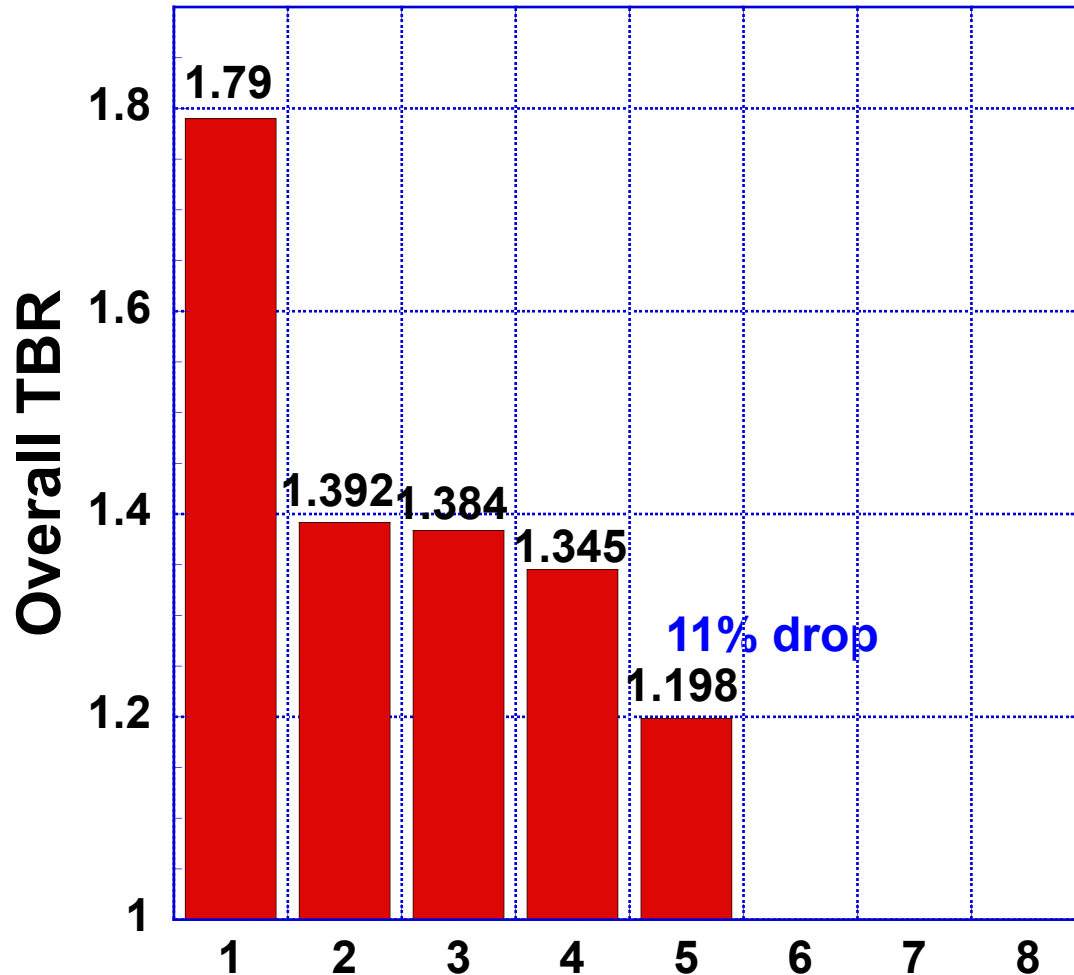
# SiC/LiPb Materials Assigned to Walls

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# SiC/LiPb Materials Assigned to Walls



## 90% Enriched Li-6

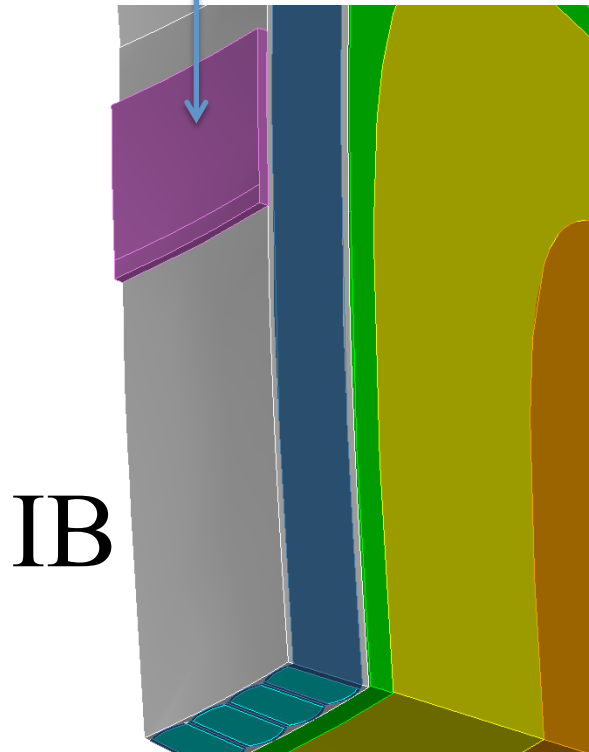
1. 1-D infinite Cylinder: 100% LiPb breeder surrounded with FS shield
2. 3-D Toroidal Model: LiPb confined to 35 cm IB blanket and 30+45 cm OB blanket
3. Add assembly gaps between blanket modules
4. Curve IB and OB blanket sectors
5. Add **blanket walls**



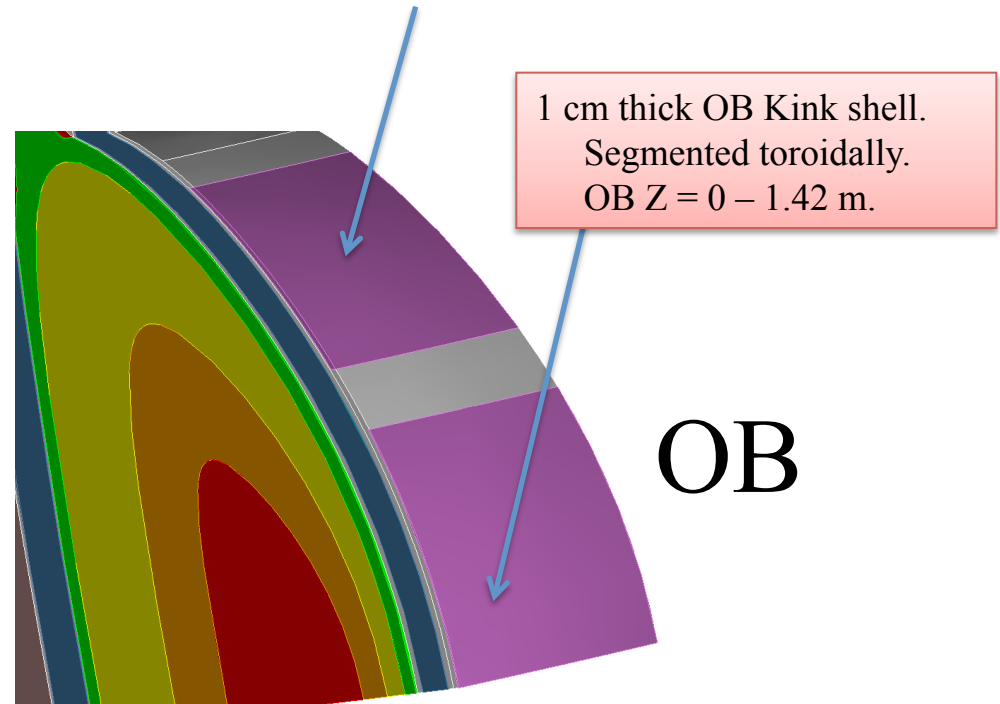


# W Stabilizing Shells Added to IB & OB (purple)

4 cm thick IB Vertical Stabilizing shell.  
Continuous toroidally.  
100% W-TiC.

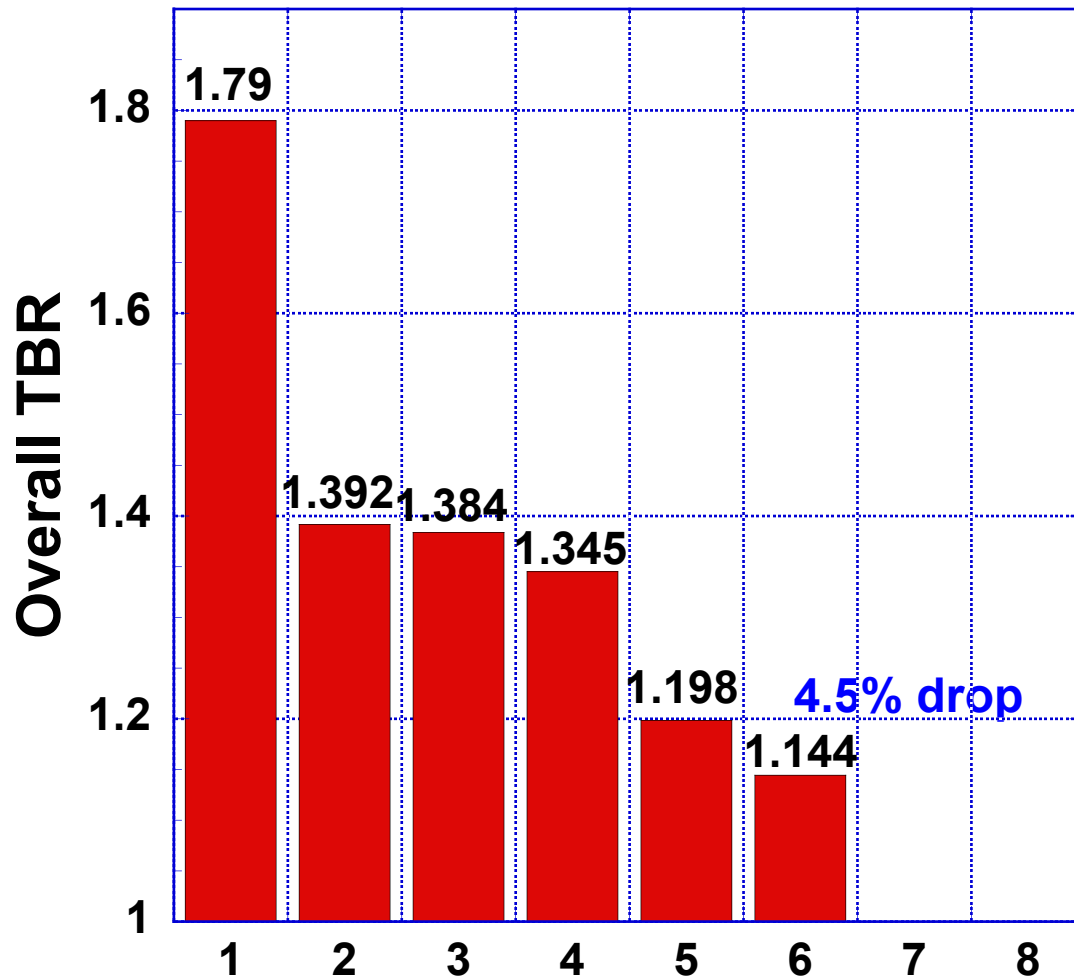


4 cm thick OB Vertical Stabilizing (VS) shell.  
Continuous toroidally.  
100% W-TiC.  
OB Z = 1.76 – 2.81 m.





# W Stabilizing Shells Added to IB & OB

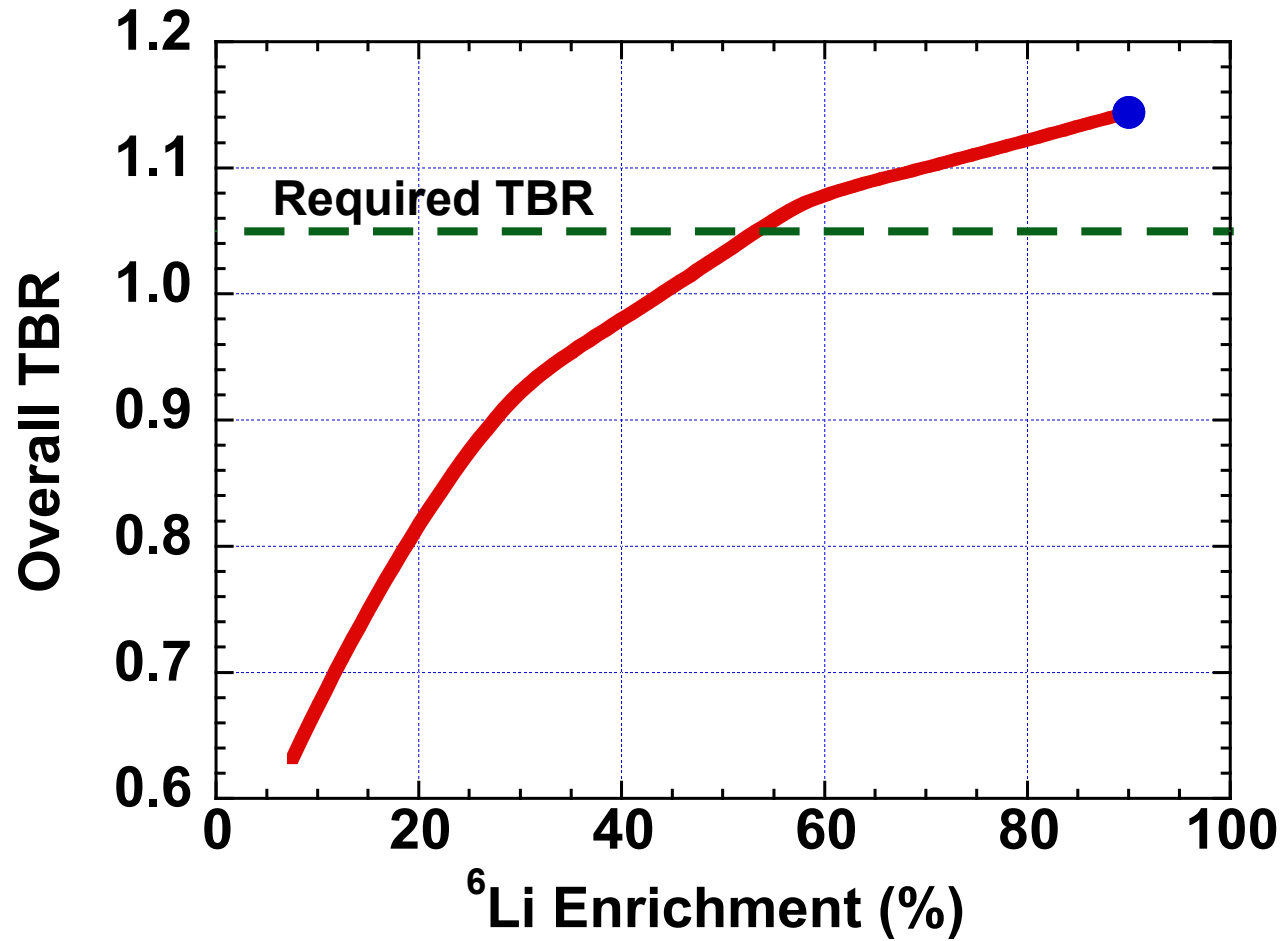


## 90% Enriched Li-6

1. 1-D infinite Cylinder: 100% LiPb breeder surrounded with FS shield
2. 3-D Toroidal Model: LiPb confined to 35 cm IB blanket and 30+45 cm OB blanket
3. Add assembly gaps between blanket modules
4. Curve IB and OB blanket sectors
5. Add blanket walls
6. Add **stabilizing shell**

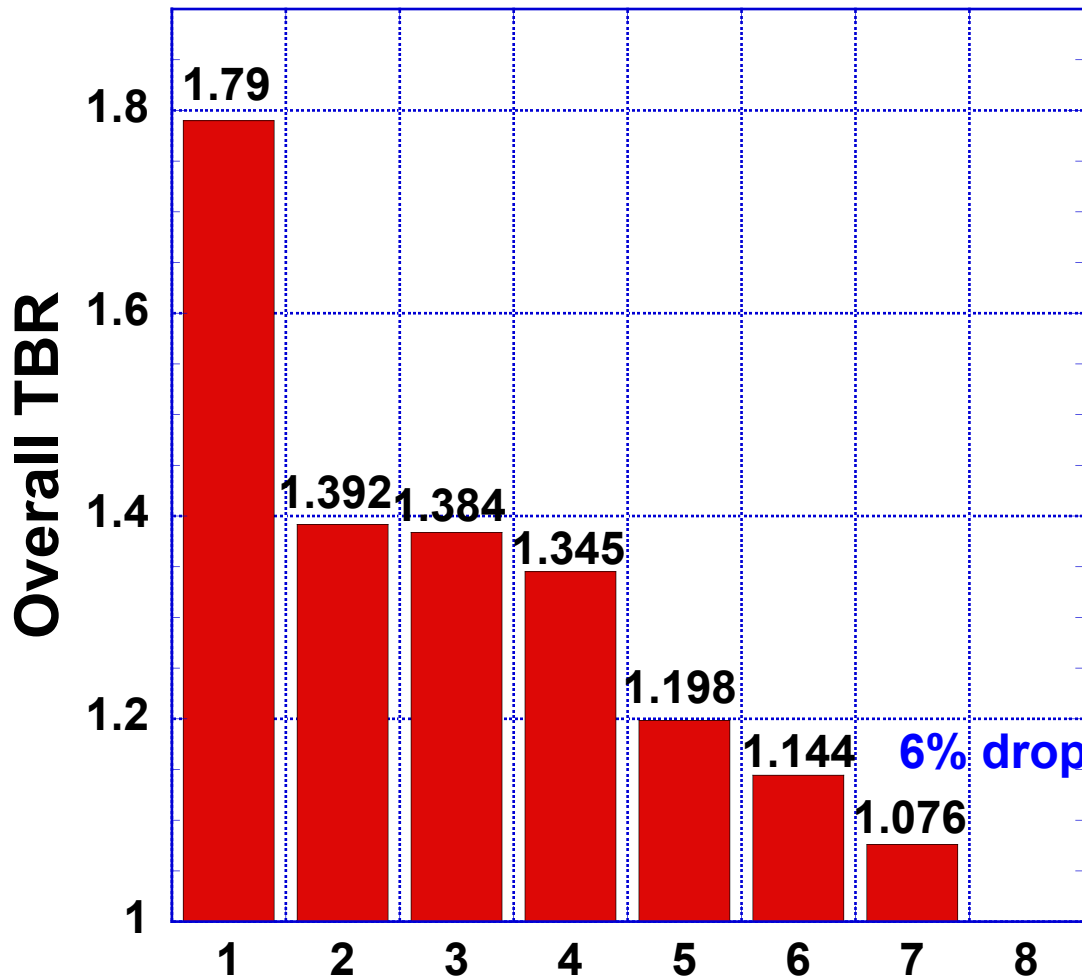


# Vary Li-6 Enrichment





# Vary Li-6 Enrichment



90% Li-6 enrichment

1. 1-D infinite Cylinder: 100% LiPb breeder surrounded with FS shield
2. 3-D Toroidal Model: LiPb confined to 35 cm IB blanket and 30+45 cm OB blanket
3. Add assembly gaps between blanket modules
4. Curve IB and OB blanket sectors
5. Add blanket walls
6. Add stabilizing shell
7. 58% Li-6 enrichment



# Penetrations

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## Footprints at FW:

Plasma control, heating, and fueling:

2 m<sup>2</sup> ICRF (or 0.5 m<sup>2</sup> EC)

2 m<sup>2</sup> LH

0.008 m<sup>2</sup> fueling ducts

Diagnostics:

3 m<sup>2</sup>

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**Total**

**7.0 m<sup>2</sup>** (or 5.5 m<sup>2</sup>)

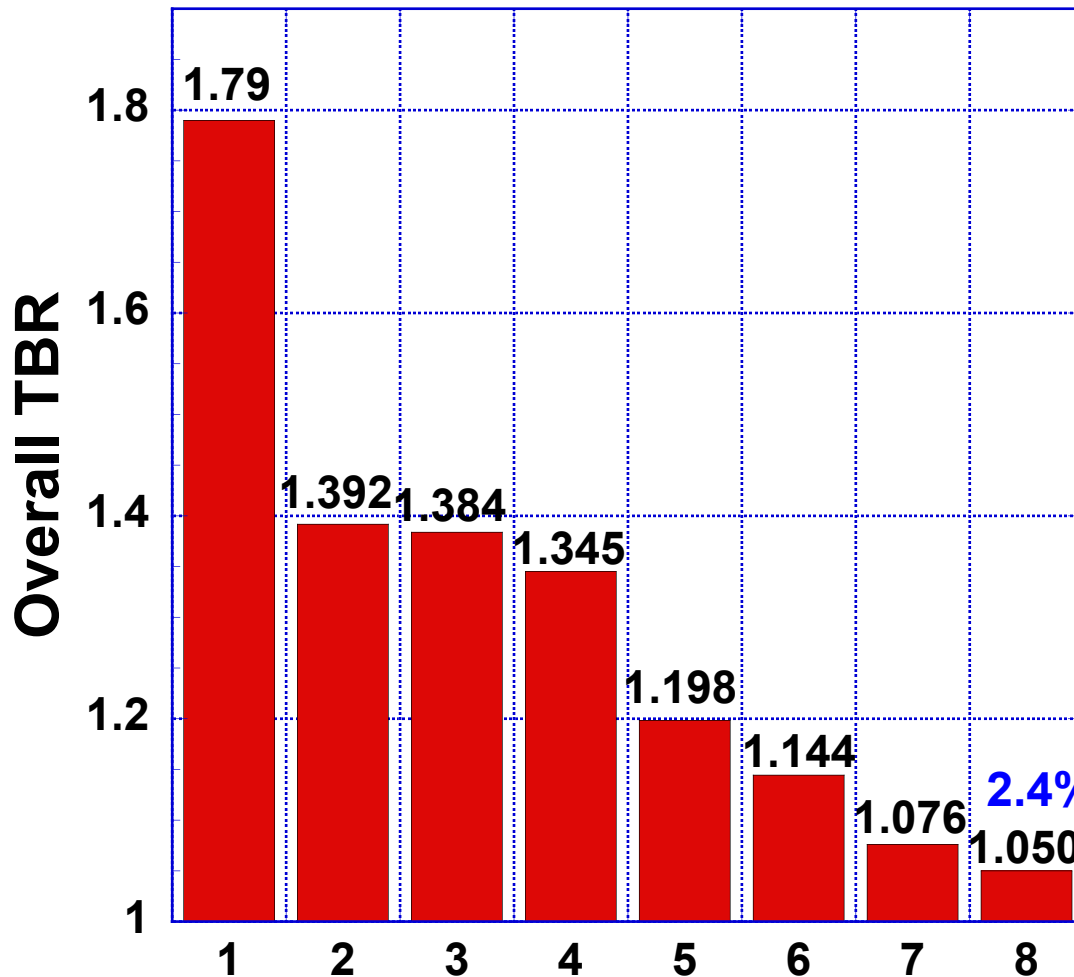
$$\text{Fraction of OB surface area} = \frac{\sim 7.0 \text{ m}^2}{313 \text{ m}^2} \approx 2.24\%$$

Maximum fraction could reach 4% of OB area.

We considered **4% of OB FW area (12 m<sup>2</sup>)**  
for ARIES-ACT penetrations



# Including Penetrations



90% Li-6 enrichment

1. 1-D infinite Cylinder: 100% LiPb breeder surrounded with FS shield
2. 3-D Toroidal Model: LiPb confined to 35 cm IB blanket and 30+45 cm OB blanket
3. Add assembly gaps between blanket modules
4. Curve IB and OB blanket sectors
5. Add blanket walls
6. Add stabilizing shell
7. 58% Li-6 enrichment
8. Add penetrations (4% of OB FW area)



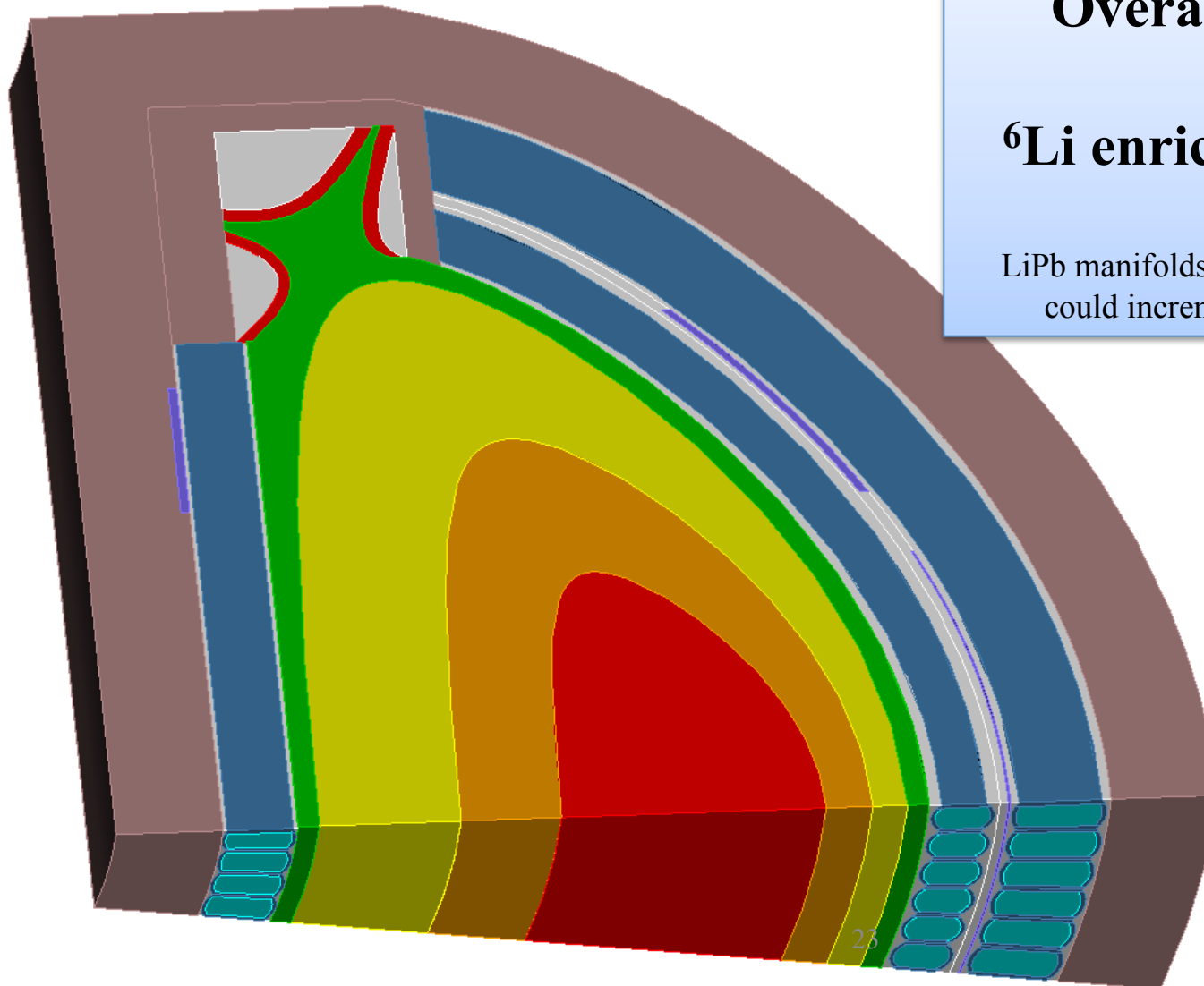
# Isometric View of Detailed Blanket

(Upper half of 1/32<sup>th</sup> (11.25°) toroidal module)

**Overall TBR = 1.05**

**<sup>6</sup>Li enrichment = ~ 60%**

LiPb manifolds behind upper/lower divertor  
could increment TBR by few percent.





# Conclusions and General Observations

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- 3-D analysis showed **progressive reduction of theoretical TBR** (~1.8) down to more realistic TBR (1.05) **when real geometry of LiPb/SiC blanket is addressed.**
- **Main findings and results:**
  - ARIES-ACT-SiC **blanket complies with ARIES breeding requirements** (calculated TBR of 1.05 with 60%  $^6\text{Li}$  enrichment (< 90%))
  - Limiting the blanket coverage radially and vertically has the largest impact on TBR (**22%**)
  - Shaping the blanket and adding the SiC structure have **14%** reduction in TBR
  - Inclusion of stabilizing shells has **~5%** impact on TBR
  - Adding penetrations and assembly gaps has smaller (**3%**) but still significant impact on TBR.
- **TBR verification:**
  - Achievable **TBR** will not be **verified** till after Demo operation with fully integrated blanket and T extraction and processing systems.
- Because many **uncertainties** in operating system govern achievable TBR during plant operation, it is a must requirement for any blanket design to have **flexible approach.**
- Most attractive scheme for LiPb breeder is to **operate with  $^6\text{Li}$  enrichment < 90%** and **increase or decrease  $^6\text{Li}$  enrichment online** shortly after plant operation\* .
- This scheme helps **mitigate concerns** about danger of placing plant at risk due to **T shortage** as well as problem of **handling and safeguarding any surplus of T.**

\* L. El-Guebaly and S. Malang, "Toward the Ultimate Goal of Tritium Self-Sufficiency: Technical Issues and Requirements Imposed on ARIES Advanced Fusion Power Plants," Fusion Engineering and Design 84 (Dec 2009) 2072-2083.