

April 23, 2003

Mr. M. Bezilla
Vice President
FirstEnergy Nuclear Operating Company
Beaver Valley Power Station
Post Office Box 4
Shippingport, Pennsylvania 15077

SUBJECT: INSPECTION OF NUCLEAR REACTOR SAFEGUARDS INTERIM
COMPENSATORY MEASURES, TEMPORARY INSTRUCTION 2515/148,
REVISION 1, INSPECTION REPORT NOS. 50-334/03-007, 50-412/03-007

Dear Mr. Bezilla:

On March 20, 2003, the U.S. Nuclear Regulatory Commission (USNRC) completed a special inspection at the Beaver Valley Power Station, in accordance with Temporary Instruction 2515/148, Revision 1, "Inspection of Nuclear Reactor Safeguards Interim Compensatory Measures." The enclosed report documents the inspection findings which were discussed on March 20, 2003, with Mr. Ken Halliday, Manager of Security, and other members of your staff.

This inspection examined activities conducted under your license, as they relate to safety and to compliance with the Commission's rules and regulations, and with the conditions of your license. The specific purpose of this inspection was to verify that FirstEnergy Nuclear Operating Company (FENOC) effectively implemented the interim compensatory measures (ICMs) at the Beaver Valley Power Station (BVPS) imposed by the Order Modifying License issued February 25, 2002; and that the ICMs were successfully incorporated into your site physical protection strategy and security program. Additionally, the objective of the inspection was to ensure that relevant procedures were developed and implemented, and that interface agreements with offsite organizations were developed and coordinated, as required. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

Based on the results of this inspection, no findings of significance were identified. The inspection team verified and confirmed that FENOC had effectively implemented the specifications of the Order Modifying License issued February 25, 2002 at the Beaver Valley Power Station (BVPS).

Mr. M. Bezilla

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Should you have any questions concerning this inspection, we will be pleased to discuss them with you.

Sincerely,

/RA by R. V. Crlenjak Acting for/

Wayne D. Lanning, Director
Division of Reactor Safety

Docket Nos. 50-334, 50-412
License Nos. DPR-66, NPF-73

Enclosure: NRC Inspection Report Nos. 50-334/03-07, 50-412/03-07
(CONTAINS SAFEGUARDS INFORMATION (SGI))

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