September 7, 2000

Mr. Oliver D. Kingsley
President, Nuclear Generation Group
Commonwealth Edison Company
ATTN: Regulatory Services
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: QUAD CITIES - NRC INSPECTION REPORT 50-254/2000012(DRS); 50-265/2000012(DRS)

Dear Mr. Kingsley:

On August 25, 2000, the NRC completed the baseline annual inspection of evaluations of changes, tests, or experiments (10 CFR 50.59) at your Quad Cities Nuclear Power Plant. The results of this inspection were discussed on August 25, 2000, with Mr. G. Barnes and other members of your staff.

This inspection was an examination of activities conducted under your license as they relate to changes to facility structures, systems, and components, normal and emergency procedures, and the updated final safety analysis report in accordance with the requirements of 10 CFR 50.59. Within these areas, the inspection consisted of selected examination of procedures and representative records, and interviews with personnel.

There were no findings identified during this inspection.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available **electronically** for public inspection in the NRC Public Document Room **or** from the *Publicly Available Records (PARS) component of NRC's document system (ADAMS)*. *ADAMS is accessible from* the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

Sincerely,

/RA by Ronald N. Gardner Acting For/

John M. Jacobson, Chief
Mechanical Engineering Branch
Division of Reactor Safety

Docket No. 50-254; 50-265
License No. DPR-29, DPR-30

Enclosure: Inspection Report 50-254/2000012(DRS); 50-265/2000012(DRS)

See Attached Distribution

cc w/encl: D. Helwig, Senior Vice President, Nuclear Services
C. Crane, Senior Vice President, Nuclear Operations
H. Stanley, Vice President, Nuclear Operations
R. Krich, Vice President, Regulatory Services
DCD - Licensing
J. Dimmette, Jr., Site Vice President
G. Barnes, Quad Cities Station Manager
C. Peterson, Regulatory Affairs Manager
M. Aguilar, Assistant Attorney General
State Liaison Officer, State of Illinois
State Liaison Officer, State of Iowa
Chairman, Illinois Commerce Commission
W. Leech, Manager of Nuclear
MidAmerican Energy Company

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John M. Jacobson, Chief
Mechanical Engineering Branch
Division of Reactor Safety

Docket No. 50-254; 50-265
License No. DPR-29, DPR-30

Enclosure: Inspection Report 50-254/2000012(DRS); 50-265/2000012(DRS)

See Attached Distribution
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M. Aguilar, Assistant Attorney General
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Chairman, Illinois Commerce Commission
W. Leech, Manager of Nuclear
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U. S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos:	50-254, 50-265
License Nos:	DPR-29, DPR-30
Report No:	50-254/2000012(DRS); 50-265/2000012(DRS)
Licensee:	Commonwealth Edison Company
Facility:	Quad Cities Nuclear Generating Plant
Location:	22712 206th Avenue N. Cordova, IL 61242
Dates:	August 21 through 25, 2000
Inspector:	Gerard O'Dwyer, Reactor Engineer
Approved by:	John M. Jacobson, Chief Mechanical Engineering Branch Division of Reactor Safety

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety	Radiation Safety	Safeguards
● Initiating Events	● Occupational	● Physical Protection
● Mitigating Systems	● Public	
● Barrier Integrity		
● Emergency Preparedness		

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: http://www.nrc.gov/NRR/OVERSIGHT/index.html.

SUMMARY OF FINDINGS

IR 50-254/2000012(DRS); IR 50-265/2000012(DRS), on 08/21-08/25/2000; Commonwealth Edison Company; Quad Cities Nuclear Power Plant. Reactor Safety Specialist report.

This report covers the initial annual baseline inspection on evaluations of changes, tests, or experiments (10 CFR 50.59 screenings and evaluations) and corrective actions relating to 10 CFR 50.59 discrepancies. This was a five day inspection by a Region III specialist engineer. No findings were identified during this inspection.

Report Details

1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems and Barrier Integrity

1R02 Evaluations of Changes, Tests or Experiments (Inspection Procedure 71111.02)

.1 Review of 10 CFR 50.59 Evaluations and Screenings for Changes, Tests or Experiments

a. Inspection Scope

The inspector reviewed nine evaluations performed pursuant to 10 CFR 50.59. The evaluations related to permanent plant design changes, special tests, procedure changes, and changes to the updated final safety analysis report. The inspector confirmed that the evaluations were thorough and that prior NRC approval was obtained as appropriate. The inspector also reviewed 12 screenings where the licensee had determined that a 10 CFR 50.59 evaluation was not necessary. In regard to the changes reviewed where no 10 CFR 50.59 evaluation was performed, the inspector verified that the changes were minor and did not require a 10 CFR 50.59 evaluation. These evaluations and applicability checks were chosen based on risk significance of samples from the cornerstones.

b. Findings

There were no findings identified.

4. OTHER ACTIVITIES

4OA2 Identification and Resolution of Problems

a. Inspection Scope

The inspector reviewed 13 problem identification forms concerning 50.59 evaluations and screenings to verify that the licensee had an appropriate threshold for identifying issues. The inspector evaluated the effectiveness of the corrective actions for the identified issues.

b. Findings

There were no findings identified.

4OA6 Management Meetings

Exit Meeting Summary

The inspector presented the inspection results to Mr. G. Barnes, Station Manager, and other members of licensee management at the exit meeting held on August 25, 2000. The licensee acknowledged the results of the inspection. No proprietary information was identified.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

E. Anderson, Radiation Protection Manager
G. Barnes, Station Manager
K. Bethard, Regulatory Assurance Coordinator
J. Dimmette, Jr., Site Vice-President - Quad Cities
M. Hayse, Nuclear Oversight Assistant Manager
W. Porter, Design Engineering Manager
R. Svaleson, Shift Operations Supervisor
C. VanDenburgh, Engineering Administration Manager
M. Wagner, Regulatory Assurance Engineer

NRC

J. Adams, Quad Cities Resident Inspector
C. Miller, Quad Cities Senior Resident Inspector
K. Walton, Quad Cities Resident Inspector

LIST OF DOCUMENTS REVIEWED

The following is a list of licensee documents reviewed during the inspection, including documents prepared by others for the licensee. Inclusion on this list does not imply that NRC inspectors reviewed the documents in their entirety, but, rather that selected sections or portions of the documents were evaluated as part of the overall inspection effort.

Procedures

NSP-AP-4004, Corrective Action Program Procedure, Revision 4
NSP-CC-3001, Operability Determination Process, Revision 0
NSP-CC-3011, UFSAR and Fire Protection Report Update Procedure, Revision 1, 11/12/98
RS-AA-104, 10 CFR 50.59 Safety Evaluation Process, Revision 0, 08/09/99

10 CFR 50.59 Evaluations

SE-99-0096, Evaluation of RHRSW Vault Flood Protection Leakage Test Results
SE-00-0010, EDG Endurance and Margin/Full Load Reject/Hot Restart Test - TS Required
SE-00-0018, Revise Unit 1, U2 & U1/2 DG Auto Start Circuit
SE-00-0039, DCP, Install “Re-Crank” Mechanism for EDGs
SE-00-0065, Unit 1 & 2 HPCI System Second Start Logic Change
SS-H-99-0157, DG Monthly Load Test, Referenced SE-99-045, DG Monthly Load Test Revision To Prevent Unnecessary Transfers.
SS-H-99-0170, Unit 2 ADS Inhibit Switch Procedure Changes, Referenced SE-98-111, Unit 1 Remote ADS Switch Installation
SS-H-99-0175, Diesel Generator Endurance and Margin/Full Load Reject/Hot Restart Test Procedure Change, Referenced SE-99-045 and SE-99-024, ½ DG Speed Sensing Panel Replacement
SS-H-99-0176, DG Monthly Load Test Procedure Change, Referenced SE-99-024

10 CFR 50.59 Screenings

SS-H-99-0189, Transfer of Unit One 125 VDC Bus Between Normal and Alternate Battery, Referenced SS-P-99-1195
SS-H-00-0060, IST Procedures' Reference Name Changes
SS-F-00-0071, T-Alt, Install an Alternate CRD Test Switch (Bypass Original Damaged Switch)
SS-F-00-0105, As-Built of 1A RHRSW Pump Spring Supports Drawing Change
SS-F-00-0149, Undervoltage Relay Trip Circuits Drawing Change at Bus 18, 19, 28, and 29
SS-F-00-0165, Drawing Change Service Water, Heating Systems and Reactor Building Doors
SS-F-00-0166, Drawing Change (as built) to M-3019 sh 3, RHRSW Pump Shaft Sleeve Design
SS-F-99-0263, Drawing Change Breaker for RPS Bus 1B and 2B Breakers
SS-F-99-0268, Replace 20 amp Fuses in ADS Panel 2201(2)-32 Drawing Change
SS-F-99-0277, RHRSW Vault Flood Protection Level Switches Drawing Change
SS-P-99-1067, "A" Loop LPCI RHRS Outage Logic Test Procedure Change
SS-P-99-1266, "B" Loop LPCI and Containment Cooling Modes of RHRS Non-Outage Logic Test

Problem Identification Forms

Q1999-03494, 10/14/99, Inadequate 50.59 Safety Evaluation for Proposed UFSAR Change
Q1999-03365, 10/05/99, Inappropriate 10 CFR 50.59 Documentation
Q2000-00706, 02/14/00, 50.59 Screening Missed Lead Seal Requirement
Q2000-01646, 04/24/00, N. O. Identifies Inadequate Safety Evaluation (SE-99-104)
Q2000-01670, 04/27/00, N.O. Identifies Problems with Safety Evaluations During NSRB Reviews
Q2000-01810, 05/11/00, DCP 9800247 50.59 Screening Requires Additional Basis Description per QRT
Q2000-01853, 05/17/00, Inadequate Process and Procedure Adherence Issue with Completed 50.59s
Q2000-01866, 05/18/00, Potentially Conflicting UFSAR Record Retention Requirements
Q2000-01867, 05/18/00, Review of Record Retention File System
Q2000-02018, 06/2/00, DCP 9300113 Did Not Revise the UFSAR and 50.59 Did Not Address Duct Changes
Q2000-02515, 07/11/00, N. O. Identifies Unacceptable 50.59 Validation
Q2000-02524, 07/11/00, Nuclear Oversight Identifies Outdated Information in Multiple Safety Evaluations
Q2000-02582, 07/18/00, Contractor Authored Appendix R DCP 50.59 Rejected by PORC