



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
611 RYAN PLAZA DRIVE, SUITE 400
ARLINGTON, TEXAS 76011-8064**

September 26, 2001

William A. Eaton, Vice President
Operations - Grand Gulf Nuclear Station
Entergy Operations, Inc.
P.O. Box 756
Port Gibson, Mississippi 39150

SUBJECT: GRAND GULF - NRC INSPECTION REPORT 50-416/01-06

Dear Mr. Eaton:

On August 23, 2001, the NRC completed an inspection at your Grand Gulf Nuclear Station. The enclosed report documents the inspection findings which were discussed on August 23, 2001, with you and other members of your staff.

This inspection was an examination of activities conducted under your license as they relate to the identification and resolution of problems, and compliance with the Commission's rules and regulations, and with the conditions of your operating license. Within these areas, the inspection involved selected examination of procedures and representative records, observations of activities, and interviews with personnel.

On the basis of the sample selected for review, there were no findings of significance identified during the inspection. The team concluded that problems were properly identified, evaluated and resolved within the problem identification and resolution programs.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter, its enclosure(s), and your response will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Anthony T. Gody, Chief
Operations Branch
Division of Reactor Safety

Docket: 50-416
License: NPF-29

Entergy Operations, Inc.

-2-

Enclosure:
NRC Inspection Report
50-416/01-06

cc w/enclosure:
Executive Vice President
and Chief Operating Officer
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Jackson, Mississippi 39286-1995

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Washington, DC 20005-3502

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Mississippi Department of Natural
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President, District 1
Claiborne County Board of Supervisors
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General Manager
Grand Gulf Nuclear Station
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Attorney General
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Baton Rouge, Louisiana 70804-9005

Office of the Governor
State of Mississippi
Jackson, Mississippi 39201

Entergy Operations, Inc.

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Dr. F. E. Thompson, Jr.
State Health Officer
State Board of Health
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 Staff Chief, DRP/TSS (**PHH**)
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 GG Site Secretary (**MJS**)

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ENCLOSURE

U.S. NUCLEAR REGULATORY COMMISSION
REGION IV

Docket(s): 50-416
License(s): NPF-29
Report No.: 50-416/01-06
Licensee: Entergy Operations, Inc.
Facility: Grand Gulf Nuclear Station
Location: Waterloo Road
Port Gibson, Mississippi
Dates: August 13-24, 2001
Inspector(s): R. Lantz, Lead Inspector, Operations Branch
S. McCrory, Senior Operations Engineer, Operations Branch
T. Hoeg, Senior Resident Inspector, Project Branch A
Approved By: A. Gody, Chief
Operations Branch
Division of Reactor Safety

SUMMARY OF FINDINGS

Grand Gulf Nuclear Station NRC Inspection Report 50-416/01-06

IR 05000416-01-06 on 8/13-23/2001, Entergy Operations, Inc., Grand Gulf Nuclear Station, Identification and Resolution of Problems.

The inspection was conducted by one regional operations engineer, one regional senior operations engineer, and one senior resident inspector. The inspection identified one issue having very low safety significance (without a color). The significance of most findings is indicated by their color (Green, White, Yellow, Red) using IMC 0609 "Significance Determination Process" (SDP). The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described at its Reactor Oversight Process website at <http://www.nrc.gov/NRR/OVERSIGHT/index.html>. Findings to which the SDP does not apply are indicated by "Without a Color" or by the severity level of the applicable violation.

Identification and Resolution of Problems

The licensee adequately identified problems and put them into the corrective action program. The licensee adequately used risk in prioritizing the extent to which individual problems would be evaluated and in establishing schedules for implementation of corrective actions. Licensee audits and assessments critically assessed the licensee's problem identification and resolution activity and identified needs for improvement in a number of areas including root cause evaluation, timely condition report initiation, and condition report backlogs. During inspection interviews, workers at the site expressed no reservations to input safety issues into the problem identification and resolution program. The licensee implemented corrective actions in a timely manner. The licensee implemented effective corrective actions to prevent recurrence of significant conditions adverse to quality.

Licensee Identified Violations

One violation of very low safety significance was identified by the licensee and reviewed by the inspectors. Corrective actions taken and planned by the licensee appear reasonable. The violation is described in Section 4OA7 of this report.

Report Details

4. OTHER ACTIVITIES (OA)

4OA2 Identification and Resolution of Problems

a. Effectiveness of Problem Identification

(1) Inspection Scope

The team reviewed items selected across the seven cornerstones of safety to determine if problems were being properly identified, characterized, and entered into the corrective action program for evaluation and resolution. Specifically, the team's review included a selection of approximately 200 condition reports (CRs) that had been opened or that related to issues of regulatory noncompliance since August 31, 2000. The team also reviewed a total of 18 licensee audits and assessments, trending reports, system health reports, and various other reports and documents related to the problem identification and resolution program. The team compared the audit and assessment results with self-revealing and NRC-identified issues to determine the effectiveness of the audits and assessments.

The team evaluated the CRs to determine the licensee's threshold for identifying problems and entering them into the corrective action program. Also, the licensee's efforts in establishing the scope of problems were evaluated by reviewing pertinent work orders, engineering requests, self-assessment results, and action plans.

The team also conducted limited plant walkdowns and interviewed plant personnel to identify other processes by which problems and issues could be identified.

(2) Issues and Findings

The team determined that the licensee was effective at identifying problems and entering them into the corrective action program. This was evidenced by the relatively few deficiencies identified by external organizations (including the NRC) that had not been previously identified by the licensee during the review period. Licensee audits and assessments were self-critical and thorough and identified issues similar to those that were self-revealing or raised during NRC inspections. The team identified no instances where conditions adverse to quality were being handled outside the corrective action program.

No specific findings relating to effectiveness of problem identification were identified.

b. Prioritization and Evaluation of Issues

(1) Inspection Scope

The team reviewed approximately 200 CRs and supporting documentation, including analyses of the problem causes, to ascertain whether the licensee's evaluation of the problems identified considered the full extent of conditions, generic implications,

common causes, and previous occurrences. In addition, the team reviewed the licensee's evaluation of selected industry experience information, including operating event reports and NRC and vendor generic notices, to assess if issues applicable to Grand Gulf were appropriately identified and addressed.

(2) Issues and Findings

Based on a review of the licensee's records, the team concluded that the licensee effectively evaluated and prioritized issues. Isolated, minor examples were noted where the records did not fully document all pertinent actions and discussions concerning the issue. These examples included lack of documentation for completed corrective actions and generic concerns that had been discussed and evaluated at various meetings, including the Condition Review Group. For the more risk significant CRs, the team determined that the evaluations were of sufficient depth, the root cause determinations were accurate, risk aspects of the conditions had been appropriately considered, and documentation was sufficient to support independent review of the corrective action response.

No specific findings relating to effectiveness of problem identification were identified.

c. Effectiveness of Corrective Actions

(1) Inspection Scope

The team reviewed the Condition Reports and other supporting documentation related to problem identification and resolution described in 4OA2.a.(1) above to verify that corrective actions were effective and implemented in a timely manner commensurate with safety, including corrective actions to address common cause or generic concerns. The team also conducted limited plant walkdowns and interviewed plant personnel to independently verify and assess the completion and effectiveness of corrective actions implemented by the licensee.

(2) Issues and Findings

The team concluded that corrective actions for those conditions reviewed were timely and effective. Evaluations for assignment of corrective actions for significant conditions adverse to quality were completed within program goals. The team did not identify any pattern of recurrence of significant conditions for which previous corrective actions to prevent recurrence were ineffective.

No specific findings were identified in the area of effectiveness of corrective actions.

d. Assessment of Safety-Conscious Work Environment

(1) Inspection Scope

During interviews with approximately 20 individuals from the licensee's staff, which represented a cross-section of functional organizations and supervisory and

non-supervisory personnel, the inspectors assessed the individual's willingness to identify safety issues and enter those issues in the corrective action process. These interviews assessed whether conditions existed that would challenge the establishment of a safety-conscious work environment.

(2) Issues and Findings

The team concluded, based on information collected from these interviews, that employees were willing to identify issues and accepted the responsibility to proactively identify and enter safety issues into the corrective action program.

No specific findings were identified in the area of assessment of safety-conscious work environment.

4OA3 Event Followup (71153)

(Closed) LER 05000416/2001-02

During the performance of pressure decay surveillance tests of the 119 foot inner and outer containment air lock door seal systems on April 2, 2001 and April 6, 2001 respectively, the licensee exceeded allowable technical specification limits of Technical Specification surveillance requirement 3.6.1.2.4. The leakage in both cases was attributed to clevis valve failures due to normal wear of the valve seals from excessive service life. In both cases, the clevis valves were repaired and the systems returned to service. As a result, in accordance with 10CFR50.73(a)(2)(vii) the licensee submitted LER 2001-02 for a single cause or condition that caused two independent trains to become inoperable in a single system designed to control the release of radioactive material.

The inspectors reviewed the licensee's common cause failure analysis and corrective actions associated with CR 2001-0526, CR 2001-0559 and CR 2001-0563 and considered them satisfactory. The licensee determined that even though the door seal leak rate exceeded specifications, the system was still capable of performing its safety function. This event did not involve any actual or credible impact on containment barrier integrity. This event did not constitute a violation of NRC requirements. This LER is closed.

4OA6 Exit Meeting

The team discussed these findings and other minor observations with Mr. Bill Eaton, Vice President, Operations, and other members of the licensee's staff, on August 23, 2001. Licensee management provided some additional information regarding lack of documentation of discussions of generic implications and extent of condition on one condition report that had taken place during the Condition Reporting Group meeting. The licensee provided no further comment on other findings or issues.

Licensee management did not identify as proprietary any materials examined during the inspection.

4OA7 Licensee Identified Violations

The following finding of very low safety significance was identified by the licensee and is a violation of NRC requirements and meets the criteria of Section VI.A of the NRC Enforcement Policy , NUREG-1600, for being dispositioned as a Non-Cited Violation (NCV).

If you deny the noncited violation, you should provide a response with the basis for your denial, within 30 days of the date of this inspection report, to the Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington DC 20555-0001; with copies to the Regional Administrator, Region IV; the Director, Office of Enforcement, United States Nuclear Regulatory Commission, Washington, DC 20555-0001; and the NRC Resident Inspector at the Grand Gulf facility.

NCV Tracking Number

Requirement Licensee Failed to Meet

50-416/2001-006-01

Technical Specification 5.4.1 requires procedures be established, implemented, and maintained for a Plant Fire Protection Program. Procedure 10-S-03-4, "Fire Prevention: Control of Combustible Material," Revisions 11 and 12 requires the use of combustible material permits to control the storage and use of combustible materials at the Grand Gulf Nuclear Station. Between January 2000 and August 2001, the licensee introduced combustible materials on numerous occasions into areas governed by the applicable procedure without proper use of a combustible material permit. The corrective actions to address this condition were contained in the licensee's condition report CR-GGN-2001-0509. This is being treated as a Non Cited Violation.

ATTACHMENT

PARTIAL LIST OF PERSONS CONTACTED

Licensee

A. Barfield, Manager, Design Engineering
C. Bottemiller, Manager, Licensing
C. Brooks, Sr. Licensing Specialist
V. Buckley, ERD Administrator
S. Burris, Supervisor, Valve Component Program
K. Christian, Supervisor, Code Program
R. Courtney, Manager, Corrective Action and Assessment (Acting)
M. Cumbest, Fire Protection Engineer
A. Dahl, Human Performance Coordinator
W. Eaton, Vice President
M. Ellis, OE Coordinator
C. Ellsaesser, Manager, Corrective Action and Assessment
D. Franklin, Division I/II Diesel Generator System Engineer
P. Griffith, Division I/II Diesel Generator System Engineer
C. Lambert, Director, Engineering
D. McDirmid, System Engineering
L. Patterson, Manager, Planning and Scheduling/Outage
C. Quick, Supervisor, Minor Modifications
M. Renfroe, Manager, Engineering Programs and Components
J. Roberts, Director, Nuclear Safety Assurance
J. Robertson, Manager, Quality Assurance
S. Rogers, Division III Diesel Generator System Engineer
G. Sparks, Manager, Operations
T. Thurmon, Supervisor, Configuration Management
J. Venable, General Manager
W. White, Supervisor, Engineering Programs and Components
D. Wiles, Manager, Engineering Support
R. Wright, Supervisor, System Engineering
G. Young, Valve Design Engineer

NRC

R. Deese, Resident Inspector

ITEMS OPENED AND CLOSED

Closed

2001-002 LER Containment Air Lock Leakage in Excess of Technical Specification Allowable Rate, (Section 4OA3)

Opened and Closed

2001-006-01 NCV Failure to follow Combustible Material Control Procedure, (Section 4OA7)

PARTIAL LIST OF DOCUMENTS REVIEWED

The following documents were selected and reviewed by the inspectors to accomplish the objectives and scope of the inspection and to support any findings:

Procedures

LI-102, Corrective Action Process, Revision 1.

01-S-03-9, GGNS Corrective Action Review Board, Revision 1

02-S-01-4, Shift Relief and Turnover, Revision 31.

06-OP-1P45-Q-0002, Floor Equipment and Chemical Drain Isolation Valve Operational Check, Revision 106

06-OP-1C41-M-0001, Standby Liquid Control System Operability, Revision 108

06-OP-1B21-R-0006, Containment, Drywell and Auxiliary Building Isolation Valves Functional Test, Revision 105

06-OP-1D17-M-0006, Off-Gas Post Treatment Radiation Monitor Functional Test, Revision 100

06-OP-1D17-Q-0004, Fuel Handling Area Ventilation Radiation Monitor Functional Test, Revision 101

06-OP-1P81-V-0003, HPCS Diesel Generator 13 Operability Verification, Revision 104

06-OP-1P75-M-0001, "Standby Diesel Generator 11 Functional Test," Revision 112

06-OP-1P75-M-0002, "Standby Diesel Generator 12 Functional Test," Revision 111

06-OP-1P75-R-0003, "Standby Diesel Generator 11: 18 Month Functional Test," Revision 106

06-OP-1P75-R-0004, "Standby Diesel Generator 12: 18 Month Functional Test," Revision 106

04-1-01-P75-1, "System Operating Instruction: Standby Diesel Generator System," Revision 57

04-1-03-P75-1, "Equipment Performance Instruction: Div 1 / 2 Diesel Generator Unexcited Run," Revision 3

04-1-01-N71-1, Circulating Water System, Revision 57

04-1-01-E51-1, Reactor Core Isolation Cooling System, Revision 115

04-1-01-E22-1, High Pressure Core Spray, Revision 105

05-1-02-VI-2, Hurricanes, Tornadoes and Severe Weather, Revision 104

01-S-06-2, Conduct of Operations, Revision 111

01-S-06-5, Reportable Events or Conditions, Revision 105

10-S-03-3, Control of Ignition Sources, Revision 9

10-S-03-4, Control of Combustible Material, Revisions 10, 11 and 12

01-S-07-9, Housekeeping, Revision 21

01-S-10-1, Fire Protection Plan, Revision 101

01-S-08-8, ALARA Program, Revision 17 and 18

Operational Experience Reviews

GGNS Operating Experience Group review documents for NRC Information Notices IN 00-17, IN 00-18, IN 00-20, IN 00-21, IN 01-04, IN 01-06, IN 01-10 and IN 01-12 and Part 21 Reports 2000-18, 2000-19, 2000-20, 2000-21, 2000-22, 2000-23, 2000-24, 2000-25, 2001-18, 2001-19, 2001-3, 2001-7, 2001-11, 2001-14, 2001-22

The following condition reports (format CR-OPX-20YY-####) were associated with the above NRC Information notices and Part 21 reports:

00-0004	00-0031	01-0039
00-0006	00-0037	01-0051
00-0014	00-0038	01-0063
00-0016	00-0039	01-0070
00-0020	01-0008	01-0071
00-0029	01-0019	01-0117
00-0030	01-0022	01-0125

Licensee Event Reports (LER)

LER 2000-03, Use of Inaccurate Meteorological Data (Voluntary)

LER 2000-04, Inadvertent Reactor Core Isolation Cooling Isolation Due to Failure to Follow Procedure

LER 2000-05, Automatic Reactor Scram Due to Offsite 500 KV Circuit Breaker Failure

LER 2000-06, Unanalyzed Condition - Turbine Control Valves May Move in Excess of Design Assumptions

LER 2001-01, Tech Spec LCO Time Exceeded Due to Two Sequential Equipment failures

LER 2001-02, Containment Air Lock Leakage in Excess Of Technical Specification Allowable Rate

Maintenance Action Items (MAI)

283945	289413	300951
284157	294041	300953
284406	296089	302453
289057	297594	302718
289144	297636	

Condition Reports (CRs)

99-0145	00-1481	01-0065	01-0248	01-0577	01-0886
99-0460	00-1484	01-0072	01-0253	01-0586	01-0924
99-1054	00-1486	01-0077	01-0285	01-0591	01-0954
99-1083	00-1493	01-0082	01-0293	01-0596	01-0955
99-1396	00-1572	01-0084	01-0303	01-0611	01-0963
99-1653	00-1587	01-0089	01-0305	01-0670	01-0974
99-1673	00-1599	01-0090	01-0313	01-0672	01-0977
99-1727	00-1604	01-0091	01-0314	01-0674	01-0994
00-0021	00-1607	01-0092	01-0318	01-0680	01-0996
00-0039	00-1617	01-0093	01-0321	01-0681	01-1002
00-0158	00-1618	01-0104	01-0318	01-0688	01-1022
00-0232	00-1621	01-0109	01-0353	01-0704	01-1030
00-0312	00-1676	01-0136	01-0386	01-0705	01-1063
00-0585	00-1681	01-0137	01-0391	01-0718	01-1064
00-0895	00-1729	01-0148	01-0395	01-0721	01-1068
00-0972	00-1731	01-0149	01-0396	01-0731	01-1072
00-1121	00-1758	01-0151	01-0397	01-0735	01-1088
00-1284	00-1761	01-0163	01-0448	01-0736	01-1099
00-1295	00-1768	01-0169	01-0458	01-0750	01-1116
00-1296	00-1774	01-0171	01-0477	01-0755	01-1119
00-1304	00-1776	01-0179	01-0501	01-0776	01-1121
00-1315	00-1786	01-0197	01-0505	01-0801	01-1146
00-1317	00-1789	01-0198	01-0509	01-0808	01-1147
00-1318	00-1801	01-0208	01-0509	01-0809	01-1149
00-1334	00-1807	01-0212	01-0514	01-0810	01-1156
00-1348	00-1813	01-0214	01-0524	01-0820	01-1157
00-1351	00-1817	01-0215	01-0530	01-0830	01-1175
00-1352	00-1867	01-0216	01-0552	01-0831	01-1182
00-1358	01-0013	01-0219	01-0559	01-0833	01-1196
00-1370	01-0016	01-0231	01-0560	01-0848	01-1371
00-1402	01-0030	01-0241	01-0561	01-0853	01-1377
00-1419	01-0052	01-0245	01-0563	01-0859	01-1385
00-1474	01-0059	01-0246	01-0565	01-0884	01-1386

01-1417

Assessments and Audits

Reactor Engineering & Operations Interface Self Assessment ES-2000-01-GGN
System Engineering - Performance Monitoring Assessment S-2000-05-GGN
Procurement Engineering Self Assessment ES-2000-18-GGN
Configuration Control CR Assessment 3rd Quarter 2000 ES-2000-19-GGN
GGNS RF-11 Outage Analysis MA-2001-12-GGN
GGNS Purchasing/Inventory Process MT-2000-02-GGN
Records Management & Document Control Self Assessment OA-2000-03-GGN
Predictive Maintenance Program Utilization and Effectiveness Assessment OA-2000-05-GGN
Equipment Reliability Gap Analysis for GGNS Top Decile Action Plan OA-2000-07-GGN
Human Performance OA-2000-09-GGN
GGNS 3rd Quarter 2000 Trend Report Recommendations OA-2000-11-GGN
Safety Assessment of the RF11 Outage Schedule OA-2001-01-GGN
GGNS 4th Quarter 2000 Trend Report Recommendations OA-2001-20-GGN
QA Audit Self Assessment Regarding NRC Findings/Violations for Year 2000 @ GGNS OA-2001-21-GGN
TRG Action Item Assessment TQ-2001-01-GGN
Effectiveness of Corrective Actions for CR-2000-0776 GLO-2001-0132
Assessment of the RP On-The-Job Training process TQ-2001-11-GGN
Configuration Control CR Assessment End of Year 2000 GLO-2001-0142

Miscellaneous

Memorandum (GIN: 2001-00491), "Diesel Generator Reliability Improvement Team," May 14, 2001

Memorandum (GIN: 2001-00214), "CR GGN 2001-1417 [2001-0214]" August 20, 2001

Standard No.: GGNS-MS-53, Attachment 1(Auxiliary Building El. 119' -0"), Revision 0

CEP-IST-PCN-14, "IST Program Change Notice"

Quarterly Trend Report, 3rd Quarter 2000

Quarterly Trend Report, 4th Quarter 2000

Quarterly Trend Report, 1st Quarter 2001

Information Request 1

Grand Gulf PIR (IP 71152) Inspection 50-416/01-006

The inspection will cover the period of Aug 31, 2000 to Aug 10, 2001. All requested information should be limited to this period unless otherwise specified. The information may be provided in either electronic or paper media or a combination of these. Information provided in electronic

media may be in the form of e-mail attachment(s), CDs, or 3½ floppy disks. The agency's text editing software is Corel WordPerfect 8, Presentations, and Quattro Pro; however, we have document viewing capability for MS Word, Excel, Power Point, and Adobe Acrobat (.pdf) text files.

Please provide the following information to Ryan Lantz in the Region IV Arlington office by July 9, 2001

1. Two summary lists, one of all currently open/active items and one of all items completed/resolved/closed since Aug 31, 2000 for:

condition reports of significant conditions adverse to quality
operator work-arounds
engineering review requests
maintenance requests
temporary modifications
procedure change requests
training needs request/evaluation
control room and safety system deficiencies
human performance issues
2. Summary list of all condition reports generated during the specified period and sorted by:

chronology
initiating organization
3. All quality assurance audits and surveillances of corrective action activities since Aug 31, 2000.
4. All corrective action activity resulting from functional area self-assessments and Non-NRC third party assessments since Aug 31, 2000.
5. Corrective action performance trending/tracking reports generated since Aug 31, 2000.
6. Current revision of the following procedures: "Condition Reporting Process," "Station Operating Orders," "Control and Use of Operations Section Directives."
7. Any additional governing procedures/policies/guidelines for:

Condition Reporting
Corrective Action Program
Root Cause Evaluation/Determination
Operator Work-Arounds
Work Requests
Engineering Requests
Temporary Modifications
Procedure Change Requests

Deficiency Reporting and Resolution
Training Needs Request/Evaluation

8. For each of the items applicable to Grand Gulf listed below please provide the following:

Full text of the condition report (please indicate any findings that did not result in a condition report or corrective actions)

Any "Roll-up" or "Aggregating" Conditions Reports related to the generic communication or condition report.

Root Cause analysis report (if applicable)

Risk significance assessments

Probable Cause evaluation (if applicable)

Approved corrective actions

Basis for extending originally approved due dates

Evidence of corrective action completion (work packages, design change documentation, temporary modifications, training lesson plans/material, training attendance records, procedure revisions, etc.)

Part 21 Reports:

- 200-23-1: Dec 11, 2000 SUBJECT: pump model computer code COTRANSA2 error, non-conservative MCPR for single recirc pump trip.
- From ABB Automation Inc. 2000-18-0: Epoxy Seal on Siemens output interface relay SUBJECT: Defective Double Pole, Double Throw relay on ABB product type 27N, 87B, 81,47H and 50D 10CFR 21.21(b) issued Aug 2, 2000
- 2001-19-0: May 11, 2001 SUBJECT: Inovision, R-11 radiation monitor spiking
- From Indian Point 3, "Possible Inoperability of a Safety Related Static Inverter, issued December 15, 2000
- 2001-18-0: May 02, 2001 SUBJECT: ABB Power, Failure of K-Line circuit breaker to close
- 2000-25-1: Dec 16,2000 SUBJECT: YUASA 125 volt battery failed voltage and capacity tests

NRC Information Notices:

- 2000-017, "Crack in Weld Area of Reactor Coolant System Hot Leg Piping @ V.C. Summer
- 2000-020, "Potential Loss of Redundant Safety-Related Equipment Because of the Lack of High-Energy Line Break Barriers"
- 2000-021, "Detached Check Valve Disc Not Detected by Use of Acoustic and Magnetic Nonintrusive Test Techniques"
- 2001-006 05/11/2001 Centrifugal Charging Pump Thrust Bearing Damage Not Detected Due to Inadequate Assessment of Oil Analysis Results and Selection of Pump Surveillance Points

LERs:

- LERS over the past year

NCVs:

- The licensee failed to establish adequate instructions to control lube oil replacement and verification of oil level in the pedestal bearing of the Division II standby diesel generator. NRC IR 50/416 00-02
- Other Events

10. Current System Health Reports or similar system information
11. Listing of plant safety issues generated through the employee concerns program since Aug 1, 2000
12. Listing of action items generated by the plant safety review committees since Aug 31, 2000
13. Current predictive performance summary reports

Information Request 2

Grand Gulf PIR (IP 71152) Inspection 50-416/01-006

Information Request for Ryan Lantz:

Complete CR packages for the following:

2000-0169, 0895, 1296, 1315, 1419, 1621, 1731,
2001-0077, 0171, 0305, 0314, 0577, 0596, 0672, 0674, 0688, 0718, 0736, 0884, 0924, 0994,
1088

Partial CR packages for the following, indicating scope of the condition and corrective actions taken (or proposed if not completed)

2000-1295, 1317, 1318, 1370, 1599, 1604, 1617, 1618, 1676, 1761, 1786
2001-0013, 0030, 0104, 0204, 0214, 0216, 0219, 0241, 0245, 0246, 0248, 0397, 0458, 0501,
0530, 0735, 0776, 0820, 0853, 0996, 1068, 1099, 1116, 1146, 1147, 1175

Information Request for Steve McCrory

Full document record for following CRs

1999-0145 1999-0460 1999-1054 1999-1083 1999-1396

1999-1673	2000-1774	2001-0151	2001-0514	2001-0859
1999-1727	2000-1776	2001-0163	2001-0524	2001-0886
2000-0021	2000-1789	2001-0179	2001-0560	2001-0954
2000-0158	2000-1807	2001-0197	2001-0561	2001-0955
2000-0232	2000-1813	2001-0198	2001-0611	2001-0963
2000-0585	2000-1817	2001-0208	2001-0680	2001-0974
2000-0972	2001-0016	2001-0212	2001-0681	2001-0977
2000-1121	2001-0052	2001-0215	2001-0721	2001-1002
2000-1284	2001-0059	2001-0231	2001-0731	2001-1022
2000-1304	2001-0065	2001-0253	2001-0750	2001-1030
2000-1334	2001-0072	2001-0293	2001-0755	2001-1063
2000-1474	2001-0082	2001-0318	2001-0801	2001-1064
2000-1484	2001-0084	2001-0353	2001-0808	2001-1072
2000-1486	2001-0089	2001-0386	2001-0809	2001-1119
2000-1493	2001-0090	2001-0395	2001-0810	2001-1149
2000-1572	2001-0091	2001-0396	2001-0830	2001-1182
2000-1607	2001-0092	2001-0448	2001-0831	2001-1196
2000-1729	2001-0093	2001-0477	2001-0833	
2000-1768	2001-0148	2001-0509	2001-0848	

CR summary list of all CRs generated during the inspection period that have human performance/error cause codes. Prefer in searchable electronic format.