

March 26, 2001

Mr. Oliver D. Kingsley, President
Exelon Nuclear
Exelon Generation Company, LLC
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: LASALLE COUNTY STATION - NRC INSPECTION
REPORT 50-373/01-04(DRS); 50-374/01-04(DRS)

Dear Mr. Kingsley:

On March 9, 2001, the NRC completed an inspection at your LaSalle County Station, Units 1 and 2. The enclosed report documents the inspection findings which were discussed on March 9, 2001, with Mr. C. Pardee and other members of your staff.

This inspection was an examination of activities conducted under your license as they relate to changes to facility structures, systems, and components, normal and emergency procedures, and the Updated Safety Analysis Report in accordance with the requirements of 10 CFR 50.59, and changes to the facility via permanent plant modifications to verify compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection consisted of a selected examination of design documents, procedures, and representative records, and interviews with personnel.

No findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Ronald N. Gardner, Chief
Electrical Engineering Branch
Division of Reactor Safety

Docket Nos. 50-373; 50-374
License Nos. NPF-11; NPF-18

Enclosure: Inspection Report 50-373/01-04(DRS);
50-374/01-04(DRS)

cc w/encl: W. Bohlke, Senior Vice President, Nuclear Services
C. Crane, Senior Vice President - Mid-West Regional
J. Cotton, Senior Vice President - Operations Support
J. Benjamin, Vice President - Licensing and Regulatory Affairs
H. Stanley, Operations Vice President
J. Skolds, Chief Operating Officer
R. Krich, Director - Licensing
R. Helfrich, Senior Counsel, Nuclear
DCD - Licensing
C. Pardee, Site Vice President
M. Schiavoni, Station Manager
W. Riffer, Regulatory Assurance Supervisor
M. Aguilar, Assistant Attorney General
Illinois Department of Nuclear Safety
State Liaison Officer
Chairman, Illinois Commerce Commission

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos: 50-373, 50-374
License Nos: NPF-11, NPF-18

Report No: 50-373/01-04(DRS); 50-374/01-04(DRS)

Licensee: Exelon Generation Company, LLC

Facility: LaSalle County Station, Units 1 and 2

Location: 2601 N. 21st Road
Marseilles, IL 61341

Dates: March 5 - 9, 2001

Inspectors: Z. Falevits, Reactor Engineer (Team Leader)
R. Daley, Reactor Engineer
K. O'Brien, Reactor Engineer
R. Winter, Reactor Engineer

Approved by: Ronald N. Gardner, Chief
Electrical Engineering Branch
Division of Reactor Safety

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas) reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety

- Initiating Events
- Mitigating Systems
- Barrier Integrity
- Emergency Preparedness

Radiation Safety

- Occupational
- Public

Safeguards

- Physical Protection

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: <http://www.nrc.gov/NRR/OVERSIGHT/index.html>

SUMMARY OF FINDINGS

IR 05000373-01-04(DRS), IR 05000374-01-04(DRS), on 03/05-03/09/2001, Exelon Generation Company, LLC, LaSalle County Station, Units 1 & 2. Evaluations of Changes, Tests, or Experiments in accordance with 10 CFR 50.59, and Permanent Plant Modifications.

The inspection was conducted by four regional inspectors from the Division of Reactor Safety.

Cornerstone: Initiating Events, Mitigating Systems, Barrier Integrity

No findings of significance were identified.

Report Details

1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems and Barrier Integrity

1R02 Evaluations of Changes, Tests or Experiments (71111.02)

.1 Review of 10 CFR 50.59 Evaluations and Screenings

a. Inspection Scope

The team reviewed 14 evaluations performed pursuant to 10 CFR 50.59. The evaluations related to permanent plant modifications, special tests, setpoint changes, procedure changes, and changes to the Updated Safety Analysis Report (USAR). The team also reviewed 21 screenings where the licensee had determined that a 10 CFR 50.59 evaluation was not necessary.

b. Findings

No findings of significance were identified.

1R17 Permanent Plant Modifications (71111.17)

.1 Review of Permanent Plant Modifications

a. Inspection Scope

The team reviewed 15 permanent plant modifications that were completed in the last several years. The modifications were chosen based upon their affecting systems that had high risk significance in the licensee's Individual Plant Evaluation or high maintenance rule safety significance. Most of the modifications involved changes to mitigating systems. The team reviewed the modifications to verify that the completed design changes were in accordance with the specified design requirements and the licensing bases and to confirm that the changes did not affect any systems' safety function. Design and post-modification testing aspects were verified to ensure the functionality of the modification, its associated system, and any support systems. The team also verified that the modifications performed did not place the plant in an increased risk configuration.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA6 Management Meetings

Exit Meeting Summary

The team presented the inspection results to Mr. C. Pardee, Site Vice President, and other members of licensee management and staff at the conclusion of the inspection on March 9, 2001. The licensee acknowledged the findings presented. No proprietary information was identified.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

C. Pardee, Site Vice President
M. Schiavoni, Station Manager
D. Bost, Site Engineering Manager
K. Bartes, Nuclear Oversight Manager
S. Stiles, Nuclear Oversight - Assessor Manager
J. Henry, Shift Operations Superintendent
P. Quealy, RP Manager
S. DuPont, Regulatory Assurance
K. Peterman, Engineering
R. McConaughay, Work Control Supervisor
F. Gogliotti, Design Engineering
M. Murskyj, Design Engineering

NRC

E. Duncan, Senior Resident Inspector
G. Wilson, Resident Inspector

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None

Closed

None

Discussed

None

LIST OF ACRONYMS USED

ADAMS	Agency Wide Documents Access and Management System
CFR	Code of Federal Regulations
CR	Condition Report
DRS	Division of Reactor Safety
NRC	Nuclear Regulatory Commission
PARS	Publicly Available Records
PERR	Public Electronic Reading Room
RHRWSW	Residual Heat Removal Service Water
TS	Technical Specifications
USAR	Updated Safety Analysis Report

LIST OF DOCUMENTS REVIEWED

The following documents were selected and reviewed by the inspectors to accomplish the objectives and scope of the inspection and to support any findings.

Condition Reports (CR)

L1998-06064	Discrepancy Found In Calculation L-001436 (GL 96-06 Relief Valve Set Pressure), December 3, 1997
L1999-02283	QRT Determined Engineering Product to be Grade 3
L1999-02020	Eng. QRT Determined Engineering Product to be Grade 3
L1999-04296	RHR 1(2) E12-F004A/B Operability During LOP-RH-07 SDC, September 11, 1999
L2000-00415	Scheduled 250Vdc Surv. Postponed
L2000 00953	Engineering Assessment DCP 9500452 50.59
L2000-01314	Engineering Rework (ERWK) UFSAR Change Overlooked in Safety Evaluation
L2000-01949	Engineering Rework (ERWK)
L2000-04131	Design Engineering Containing Improvement Plan (ERWK)
L2000-04209	Engineering Rework (ERWK) Concerning Safety Evaluation L00-0900
L2000-04673	N.O. Identified Root Cause Corrective Action not Tracked in Action Tracking
L2000-04956	Design Engineering Weekly Critique
L2000-06663	Design Engineering Weekly Critique for Week Ending November 12, 2000
L2000-06675	Weekly Critique - System Engineering
L2000-06862	(ERWK) Wiring Record Change Omitted in D/G 2A DCP 9900362
L2000 06339	Screening Missed RSAR Drawing Change
L2000 06655	Missing OAD Testing Activities for DCPs
L2000 07107	Configuration Control Logsheet w/o 10 CFR 50.59
L2000-04349	NRC Identified: Concerns Regarding Configuration Control Regarding Replacement HVAC Filters, August 3, 2000
L2000-04454	Concern Regarding HVAC Filters, August 9, 2000
L2000-06843	Extent of Condition for HVAC Filter Discrepancies, November 26, 2000
L2001-00340	Eng. Rework of Safety Evaluation L01-0040
L2001-00563	Discrepancy with Revision 13 and Pending Revision 14 of the Updated Final Safety Analysis Report, January 30, 2001
L2001-00581	Engineering Program Weekly Trending CR

Condition Reports CRs Written During this Inspection for NRC-Identified Issues

L2001-01479	Calculation L-000813 Canceled Without Updating Safety Evaluation
L2001-01504	Knowledge Deficiency in the 10 CFR 50.59 Screening Process

Calculations

L-000050	MOV Motor Terminal Voltage Calculation for System E12, Revision 0
L-002588	Loss of Voltage Relay Setpoint for 4.16 kV Buses 141Y, 142Y, 143, 241Y, 242Y, 243- Undervoltage Function 4266/19D27, 125V Div. 1 Battery Sizing, Revision 4
NED-I-EIC-0197	HPCS Discharge Pressure Minimum Flow Bypass and LPCS and LPCI Discharge ADS Permissive Error Analysis, Revision 1, January 13, 1998
L-000120	Recalibration of RHR Service Water Transmitters, January 25, 1996

- L-000711 Evaluation of RHR Service Water Flow to RHR Pump Seal Coolers, Revision 4, November 6, 2000
 - L-001212 Qualification of Replacement ECCS and RCIC Suction Strainers, Revision 0, August 28, 1997
 - L-001260 ECCS and RCIC Suppression Pool Suction Strainer Head Loss for a 50 Percent Plugged Strainer, Revision 0, August 28, 1997
 - L-001780 RHR Heat Exchanger - Cooling Water Orifice E12-D304A/B, Revision 2, April 7, 1998
- Handwheel Force Values for Closing Manual Valves 1(2) E12-F018A/B/C With The RHR Pump Operating At Shut-Off Head, March 9, 2001

Design Changes

- 9400036 DG 2A Differential Relay Replacement, September 4, 1996
- 9500088 Unit 2, Div 1, Degraded Voltage Trip Setpoint Change, December 25, 2000
- 9600347 ESF DIV 1 Molded Case Breaker Magnetic Setting, March 18, 1999
- 9600567 Modify 2B DG Voltage Regulator Circuit Wiring, September 17, 1998
- 9700027 2E12-F027A;F024A;F042A;2E21-F005 TOL Bypass Logic Changes, February 22, 1999
- 9900043 Replacement of Circuit Breaker for Valve 2E 12-F042A, March 15, 1999
- 9900315 Permanent Closure of Relief Valves 2E12-F055A & 2E12-F055B (Unit 2), March 15, 2000
- 9900338 Install Panel Wire and Banana Jacks in Panel 1H13-P628, May 19, 2000
- 9400434 Gear change for 1E12-F053A and 1E12-F053B, March 12, 1996
- 9600027 Recalibration of RHR Service Water Flow Transmitters, January 29, 1996
- 9600284 Remove U2 Main Steam Line High Radiation Monitor Scram/Trip Function, March 9, 1999
- 9600460 Removal of 10 minute timer interlock for E12-F048, April 21, 1997
- 9600616 Setpoint Change for ADS Permissive Bypass, May 6, 1998
- 9700001 Install Emergency Battery Packs for Valve 1E12-F018B, October 18, 1997
- 9700544 ECCS Suction Strainer Replacement Modification - RHR C, March 25, 1998
- 9700601 2E12-F313B Reorient RHR SW Heat Exchanger Relief Valve, October 29, 1998
- 9800055 Resize 2E12-D304A RHR SW Restricting Flow Orifice, October 27, 1998
- 9800296 2E12-C300A 2A RHR Service Water Pump Repair Modification, December 23, 1998
- 9800349 OWS01P Revise Service Water Pump Auto Trip Alarm Logic, January 18, 1999
- 9900127 MCC Circuit Breaker Setting Change for 2B DG "B10" Air Comp, May 28, 1999

10 CFR 50.59 Evaluations

- L1997-018 Safety Evaluation of the Replacement of ECCS Pump Suction Strainers for RHR, LPCS, LPCI, and HPS, Revision 1, April 1, 1998
- L98-0512 Safety Evaluation of a Design Change to Add Three Emergency Battery Pack Lights and to Revise UFSAR Tables H.4-3, H.4-30, and H.4-31 to Add Valve 1(2)E12-F018B, November 11, 1998
- L99-0910 TMOD Installing Two Temporary Power Feeds in the Turbine Building
- L99-1133 Jumpers installed to Defeat the Auto Start Capability of Unit 1 & 2 SGBT System and Bypass VR Dampers for the Purpose of Replacing Relays and Installing Banana Jacks, October 27, 1999
- L00-0007 Changes to UFSAR Chapter 8
- L00-0123 Chapter 8 & 9 UFSAR Changes for DBI, May 10, 2000

L00-0188 DG Loading Requirements (LU 1999-169)
L00-0220 Safety Evaluation of Changes to Safety Analysis Report Sections 7.3.1.2.4.6 and 7.3.1.2.5, March 8, 2000
L00-0226 Miscellaneous Changes to the UFSAR
L00-0229 Clarification and Correction to UFSAR Sections 9.2.3.2 and 9.2.9.2
L00-0281 Safety Evaluation of Changes to Safety Analysis Report Section 2.2.3.2.c, 7.3.4.3.14, 7.3.4.3.7, 9.4.1.1.3., and 9.4.1.2.3, March 20, 2000
L00-0286 DG Loading Requirements (Tech Spec Basis)
L00-0295 Permanent Closure (gagging) of RHR Heat Exchanger RCIC Steam Inlet Header Relief Valves 2E12-F055A and 2E12-F055B, March 14, 2000
L00-0438 Miscellaneous Editorial Changes to UFSAR, Revision 1
L00-0525 Conditions for DC Testing (LTS-700-5, LOP 0C-02)
L00-0593 UFSAR Sections 3.11,7.4.3, 7-7.1, 7.76 Power Uprate (LU 2000-067)
L00-0537 UFSAR Change (LU 2000-103)
L00-0640 Safety Evaluation of a Change to the Safety Analysis Report to Increase the Peak Main Cooling Lake Supply Temperature From 97 to 97.5 Degrees Fahrenheit, May 23, 2000

10 CFR 50.59 Screenings

L99-1014 Temp Power - U1 RB (TMOD1-0029-99, Revision 2)
L99-1172 Unit 1 RHR System Div 1 Relay Logic Test (LES-RH-100, Revision 13)
L99-1233 U-1 Rx Vessel Io Water Lvl 3 Scram & RHR Isolation Calibration (LIS-NB-101A, Revision 8, LIS-NB-101B, Revision 8)
L99-1368 125 Vdc System Div 2 Ground Locator and Isolation, January 21, 1999 (LOP-DC-05)
L99-1406 U1 RHR B+C (LPCI Mode) Pump Discharge Pressure ADS Permissive Cal (LIS-RH- 101B, Revision 5)
L99-1458 U-1 Rx Vessel Io Water Lvl 1 ECCS Div ½ Initiation & Lvl 2 RCIC Init (LIS-NB-104A/B, Revision 8/Revision 6)
L00-0028 Safety Screening for a Temporary Procedure Change (173-99) to Procedure LOP-RH-05, Revision 19, Operation of RHR Service Water System, January 7, 2000
L00-0103 Procedure Change to Incorporate Methodology Provided in GE SAL 350.1, January 25, 2000
L00-0179 Inspection of 4.16kV and 6.9kV ITE Circuit Breakers (LES-GM-103)
L00-0193 Safety Screening of Changes to Procedures LIS-RH-301A, Revision 1, and LIS-RH-301B, Revision 2, "Unit 1RHR A(B&C) (LPCI Mode) Pump Discharge Pressure ADS Permissive Functional Test," February 16, 2000
L00-0214 1B(2B) Diesel Generator Operability Test, February 21, 2000 (TPC #005-00 to LOS-DG-M3, Revision 42)
L00-0346 Safety Screening of Changes to Procedures LOP-RH-01M and LOP-RH-03M, "RHR Service Water System Mechanical Checklists," March 28, 2000
L00-0348 Safety Screening of Changes to Procedure LOP-RH-04, Revision 11, "Filling and Venting the RHR Service Water System," March 28, 2000
L00-0351 Safety Screening of Changes to Procedure LIS-RH-101A, Revision 6, "Unit 1 RHR A (LPCI Mode) Pump Discharge Pressure ADS Permissive Calibration," March 1, 2000

L00-0490 U2 Power Uprate (DCP 9900155)
 L00-0502 RHR System Filling and Venting (LOP-RH-01, Revision 32)
 L00-0566 Unit 1 RHR System Div 2 Relay Logic Test (LES-RH-101, Revision 13)
 L00-0583 U2 AC Power Sys Abnormal (LOA-AP-201)
 L00-0588 Install Accessible Test Points for Terminals 1 and 2 of Relay 1(2)B21C-K009A (DCP 9900338 & DCP 9900339)
 L00-0604 U1 RHR Division 1 Relay Logic Test (LES-RH-100, Revision 14)
 L00-0646 Temporary Battery Charger (LOS-RP-Q3, Revision 2) Safety Screening for a Change to Procedure LOA-FX-101, Revision 1, "Unit 1 Safe Shutdown with a Loss of Offsite Power and a Fire In The Control Room or AEER," May 28, 1997

Tests

LST-96-209 Logic Functional Test 2HS-AP060, 2414 Feed to 235X & 235Y, Revision 1
 LST-97-113 2HS-AP044 Sw Replacement Logic and Functional Test SAT Feed to Bus 241Y
 LST-97-712 RHR Logic & Function Test RHR Pump 2A, Revision 0
 LST-98-236 Unit 2 A/B RHR Heat Exchanger ASME Section XI Pressure Test
 LST-2000-010 Unit 1/2 Div I, II Ground Detector Functional Test, Revision 0

Procedures

CC-AA-102 Design Impact Screening, Revision 1
 CC-AA-103 Design Change Package, Revision 0
 CC-AA-106 Performance of Walkdowns and Control of Walkdown Information, Revision 0
 CC-AA-107 Design Change Acceptance Testing Criteria, Revision 0
 CC-AA-110 Field Change Requests, Revision 1
 CC-AA-202 Revision 0, Quality Review Team (QRT)
 RS-AA-104 10 CFR 50.59 Safety Evaluation Process, Revision 0
 RS-AA-107 UFSAR and Fire Protection Report Update Procedure, Revision 0
 NES-G-16 Guidance for Determining Design Change Package (DCP) Applicability Development and Implementation, Revision 0, June 29, 2000

Miscellaneous Documents

Evaluation 00000784, SR Part 21 Pressure Switch
 Evaluation 00004105, for Dragon Valve
 LaSalle Station OE.01: QRT Indicator
 WR 950067086, Unit 2 Div I, Degrade Voltage Trip Setpoint Change, February 28, 2000.