November 15, 2000

Mr. M. Reddemann
Site Vice President
Kewaunee and Point Beach Nuclear Plants
Wisconsin Electric Power Company
6610 Nuclear Road
Two Rivers, WI 54241

SUBJECT: POINT BEACH - NRC INSPECTION REPORT 50-266/00-15(DRS); 50-301/00-15(DRS)

Dear Mr. Reddemann:

On October 27, 2000, the NRC completed a routine inspection at the Point Beach Nuclear Plant. The enclosed report presents the results of that inspection. The results were discussed on October 27, 2000, with Mr. Cayia and other members of your staff.

The inspection was an examination of activities conducted under your license as they relate to radiation safety and to compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection consisted of a selective examination of representative records, tours of your facility and interviews with personnel.

No findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

We will gladly discuss any question you have concerning this inspection.

Sincerely,

/RA Steven K. Orth Acting for/

Gary L. Shear, Chief
Plant Support Branch
Division of Reactor Safety

Docket Nos. 50-266; 50-301
License Nos. DPR-24; DPR-27

Enclosure: Inspection Report 50-266/00-15(DRS); 50-301/00-15(DRS)

See Attached Distribution

cc w/encl: R. Grigg, President and Chief
Operating Officer, WEPCo
M. Wadley, Chief Nuclear Officer, NMC
J. Gadzala, Licensing Manager
D. Weaver, Nuclear Asset Manager
F. Cayia, Plant Manager
J. O'Neill, Jr., Shaw, Pittman,
Potts & Trowbridge
K. Duveneck, Town Chairman
Town of Two Creeks
B. Burks, P.E., Director
Bureau of Field Operations
A. Bie, Chairperson, Wisconsin
Public Service Commission
S. Jenkins, Electric Division
Wisconsin Public Service Commission
State Liaison Officer

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos:	50-266; 50-301
License Nos:	DPR-24; DPR-27
Report No:	50-266/00-15(DRS); 50-301/00-15(DRS)
Licensee:	Wisconsin Electric Power Company
Facility:	Point Beach Nuclear Plant, Units 1 and 2
Location:	6610 Nuclear Road Two Rivers, WI 54241
Dates:	October 23-27, 2000
Inspector:	D. Nelson, Radiation Specialist
Approved by:	Gary L. Shear, Chief Plant Support Branch Division of Reactor Safety

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety	Radiation Safety	Safeguards
● Initiating Events	● Occupational	● Physical Protection
● Mitigating Systems	● Public	
● Barrier Integrity		
● Emergency Preparedness		

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: http://www.nrc.gov/NRR/OVERSIGHT/index.html.

SUMMARY OF FINDINGS

IR 05000266-00-15(DRS); 05000301-00-15(DRS), on 10/23-10/27/00, Wisconsin Electric Power Company, Point Beach Nuclear Plant, Units 1 and 2.

The inspection was conducted by a regional radiation specialist. No findings of significance were identified.

Report Details

Summary of Plant Status: Unit 1 was at full power, and Unit 2 was shutdown for a refueling outage (U2R24) during the inspection period.

2. RADIATION SAFETY

Cornerstone: Occupational Radiation Safety

2OS1 Access Control to Radiologically Significant Areas

.1 Plant Walkdowns, Radiological Boundary Verifications and Observations of Radiation Worker Performance

a. Inspection Scope

The inspector conducted walkdowns of the radiologically controlled area (RCA) to verify the adequacy of radiological boundaries and postings. Specifically, the inspector walked down several radiologically significant work area boundaries (high and locked high radiation areas) in the Unit 1 and 2 Primary Auxiliary Building, the Unit 2 containment, and the Turbine Building. In addition, confirmatory radiation measurements were performed to verify that access to these areas and to selected radiation areas were properly posted and controlled in accordance with 10 CFR 20, licensee procedures, and Technical Specifications. The inspector also observed radiation workers performing the activities described in Section 2OS2.2, evaluated their awareness of radiological work conditions, and verified the implementation of radiological controls specified in applicable radiation work permits and as-low-as-is-reasonably-achievable (ALARA) plans.

b. Findings

No findings of significance were identified.

2OS2 As-Low-As-Is-Reasonably-Achievable (ALARA) Planning and Controls

.1 ALARA Planning

a. Inspection Scope

The inspector reviewed the station's collective exposure histories for 1997 to present, current exposure trends for the ongoing Unit 2 refueling outage (U2R24), and planned and completed radiological work activities for the outage to assess current performance and exposure challenges. The inspector reviewed the exposure data and the station's three-year rolling average exposure information and compared it with national pressurized water reactor industry data.

b. Findings

No findings of significance were identified.

.2 Job Site Inspections and ALARA Controls

a. Inspection Scope

The inspector selected the following high exposure or high radiation area active or recently completed job activities and evaluated the licensee's use of ALARA controls:

- 2CV-1298 Replacement/Modification (containment);
- Insulation Work/Scaffolding Construction Support;
- In Service Inspections (ISI);
- Steam Generator Eddy Current Testing and Tube Plugging; and
- Steam Generator HH Cover Removal Installation and Steam Generator Sludge Lancing.

The inspector surveyed work areas to verify that radiation levels were consistent with the licensee's survey data and verified that low dose areas were designated and appropriately used by workers. The inspector evaluated the licensee's engineering controls at selected locations and verified that the controls were consistent with those specified in the ALARA plans. The inspector also observed and questioned workers at each job location to determine that they had adequate knowledge of radiological work conditions and exposure controls.

b. Findings

No findings of significance were identified.

.3 Source Term Reduction and Control

a. Inspection Scope

The inspector reviewed the status of the licensee's source term reduction program, focusing on those initiatives taken for the outage which included shutdown chemistry controls (i.e., early boration/hydrogen peroxide addition), hydrolazing and other decontamination work, and installation of permanent and temporary shielding. The inspector also evaluated other ongoing source term reduction strategies, such as water chemistry control and hot spot reduction initiatives, and verified that a viable source term control program was in place. The inspector also performed surveys within the radiologically controlled area to verify the accuracy of the licensee's records of identified hot spots and to identify any other significant unidentified sources of radiation exposure.

b. Findings

No findings of significance were identified.

.4 Radiological Work Planning

a. Inspection Scope

The inspector selected the following outage job activities that were estimated to exceed five person-rem or were conducted in high radiation areas and assessed the adequacy of the radiological controls and work planning:

- 2CV-1298 Replacement/Modification (containment);
- Insulation Work/Scaffolding Construction Support;
- In Service Inspections (ISI);
- Steam Generator Eddy Current Testing and Tube Plugging; and
- Steam Generator HH Cover Removal Installation and Steam Generator Sludge Lancing.

For each job activity, the inspector reviewed ALARA evaluations and associated dose mitigation techniques and evaluated the licensee's exposure estimates and performance. The inspector also assessed the integration of ALARA requirements into work packages to evaluate the licensee's communication of radiological work controls.

b. Findings

No findings of significance were identified.

.5 Verification of Exposure Goals and Exposure Tracking System

a. Inspection Scope

The inspector reviewed the methodology and assumptions used for the U2R24 exposure estimates and exposure goals and compared job dose rate and man-hour estimates for accuracy. The inspector verified that job dose history files and dose reductions anticipated through lessons learned were appropriately used to forecast outage doses. The inspector also reviewed the licensee's exposure tracking system to determine if the level of exposure tracking detail, exposure report timeliness and exposure report distribution was sufficient to support control of collective exposures. The inspector verified that the licensee's dose estimates for the outage were reasonably accurate and confirmed that no outage jobs greater than five person rem exceeded respective dose estimates by more than 50 percent.

b. Findings

No findings of significance were identified.

.6 Declared Pregnant Workers

a. Inspection Scope

The inspector determined if there had been any declared pregnant workers during the current assessment period. For those individuals, the inspector reviewed the exposure results and monitoring controls employed by the licensee with respect to the requirements contained in 10 CFR Part 20 and the licensee's procedures.

b. Findings

No findings of significance were identified.

.7 Identification and Resolution of Problems

a. Inspection Scope

The inspector evaluated the effectiveness of the self-assessment process to identify, characterize, and prioritize problems and verified that previous outage related ALARA issues were adequately addressed and resulted in improved dose performance. The inspector also reviewed radiation protection assessments and outage generated condition reports to assess the adequacy of the licensee's ability to identify problems.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA6 Management Meetings

Exit Meeting Summary

The inspector presented the inspection results to Mr. Cayia and other members of licensee management and staff in an exit meeting on October 27, 2000. The licensee acknowledged the information and findings presented. No proprietary information was identified by the licensee.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

A. Cayia, Plant Manager
J. Lindsay, General Supervisor-RP
C. Onesti, Health Physicist
R. Repshas, Manager-Site Services
D. Shannon, Radiation Protection Supervisor
S. Thomas, Radiation Protection Manager

NRC

J. Lara, Senior Resident Inspector

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None

Closed

None

Discussed

None

LIST OF ACRONYMS USED

ALARA	As-Low-As-Is-Reasonably-Achievable
CFR	Code of Federal Regulations
ISI	In Service Inspections
NRC	Nuclear Regulatory Commission
PDR	Public Document Room
RCA	Radiologically Controlled Area
U2R24	Unit-2, 24th Refueling Outage

PARTIAL LIST OF DOCUMENTS REVIEWED

The following documents were selected and reviewed by the inspectors to accomplish the objectives and scope of the inspection and to support any findings.

Station Procedures

NP 4.2.1 (Revision 4)	Plant ALARA Program
NP 4.2.15 (Revision 2)	Fetal Protection Implementation Policy
NP 4.2.29 (Revision 2)	Source Term Reduction Program

ALARA Reviews with associated RWPs

ALARA Review # 2000-0003	2CV-1298 Replacement/Modification
ALARA Review # 2000-0011	Inservice Inspections
ALARA Review # 2000-0012	Insulation Work/Scaffold Construction Support
ALARA Review # 2000-0013	Steam Generator Eddy Current Testing and Tube Plugging
ALARA Review # 2000-0014	SG HH Cover Removal/Installation and S/G Secondary Sludge Lancing

Radiation Protection Assessments

NPM 1999-1347, Review of the Radiation Protection Program for the Year 1999, December 23, 1999
NPM 2000-0630, Radiation Protection Self-Assessment; Radiation Protection Planning Scheduling and RWP Processes, July 24, 2000

Condition Reports

2000-2720, 2000-2986, 2000-03202, 2000-3189

Other Documents

Dose Data, 1976(actual) - 2005(Projected)
Shielding Package Listing for U2R24, August 28, 2000
Point Beach Nuclear Plant, Strategic Plan for Radiation Dose Reduction (Draft)
Point Beach Nuclear Plant, Dose Goal Performance for the Year 2000
Listing of all U2R24 RWPs with Associated Dose Goals
Plans of the Day, Meeting Agenda, October 24-27, 2000