

November 21, 2001

Mr. Oliver D. Kingsley, President  
Exelon Nuclear  
Exelon Generation Company, LLC  
1400 Opus Place Suite 500  
Downers Grove, IL 60515

SUBJECT: BRAIDWOOD GENERATING STATION  
NRC INSPECTION REPORT 50-456/01-12(DRS); 50-457/01-12(DRS)

Dear Mr. Kingsley:

On November 2, 2001, the NRC completed an inspection at your Braidwood Generating Station. The enclosed report documents the inspection findings which were discussed on November 2, 2001, with Mr. K. Schwartz and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel. Specifically, this inspection focused on the baseline annual inspection for evaluations of changes, tests, or experiments (10 CFR 50.59) and the baseline biennial inspection for permanent plant modifications.

No findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,

*/RA/*

Ronald N. Gardner, Chief  
Electrical Engineering Branch  
Division of Reactor Safety

Docket Nos. 50-456; 50-457  
License Nos. NPF-72; NPF-77

Enclosure: Inspection Report 50-456/01-12(DRS);  
50-457/01-12(DRS)

See Attached Distribution

cc w/encl: J. Skolds, Chief Operating Officer  
W. Bohlke, Senior Vice President, Nuclear Services  
C. Crane, Senior Vice President - Mid-West Regional  
Operating Group  
J. Cotton, Senior Vice President - Operations Support  
J. Benjamin, Vice President - Licensing and Regulatory Affairs  
R. Hovey, Operations Vice President  
K. Ainger, Director - Licensing  
R. Helfrich, Senior Counsel, Nuclear  
DCD - Licensing  
J. von Suskil, Site Vice President  
K. Schwartz, Plant Manager  
A. Ferko, Regulatory Assurance Manager  
M. Aguilar, Assistant Attorney General  
Illinois Department of Nuclear Safety  
State Liaison Officer  
Chairman, Illinois Commerce Commission

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Docket Nos. 50-456; 50-457  
License Nos. NPF-72; NPF-77

Enclosure: Inspection Report 50-456/01-12(DRS);  
50-457/01-12(DRS)

See Attached Distribution

DOCUMENT NAME: G:DRS\BRA01-12DRS.WPD

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cc w/encl: J. Skolds, Chief Operating Officer  
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Operating Group  
J. Cotton, Senior Vice President - Operations Support  
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Illinois Department of Nuclear Safety  
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Chairman, Illinois Commerce Commission

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U. S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos: 50-456; 50-457  
License Nos: NPF-72; NPF-77

Report No: 50-456/01-12(DRS), 50-457/01-12(DRS)

Licensee: Exelon Generation Co., LLC

Facility: Braidwood Generating Station

Location: 35100 S. Route 53  
Suite 84  
Braceville, IL 60407-9617

Dates: October 29 through November 2, 2001

Inspectors: George M. Hausman, Senior Reactor Inspector  
Patricia V. Loughheed, Senior Reactor Inspector  
Donald C. Prevatte, Beckman and Associates, Inc.

Approved by: Ronald N. Gardner, Chief  
Electrical Engineering Branch  
Division of Reactor Safety

## SUMMARY OF FINDINGS

IR 05000456-01-12(DRS), IR 05000457-01-12(DRS), on 10/29-11/02/2001, Exelon Generation Company, LLC, Braidwood Generating Station. Evaluations of Changes, Tests, or Experiments and Permanent Plant Modifications Report.

The report covers a five day announced inspection. The inspection was conducted by two Region III based reactor inspectors and one contractor. No findings of significance were identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using IMC 0609 "Significance Determination Process" (SDP). The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described at its Reactor Oversight Process website at <http://www.nrc.gov/NRR/OVERSIGHT/index.html>. Findings for which the SDP does not apply are indicated by "No Color" or by the severity level of the applicable violations.

### A. Inspector Identified Findings

#### **Cornerstones: Initiating Events, Mitigating Systems, Barrier Integrity**

No findings of significance were identified.

## Report Details

### Summary of Plant Status

Unit 1 operated at or near full power and Unit 2 operated at or near 85 percent power throughout the inspection period.

#### **1. REACTOR SAFETY**

##### **Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity**

#### 1R02 Evaluations of Changes, Tests, or Experiments (71111.02)

##### .1 Review of 10 CFR 50.59 Evaluations and Screenings for Changes, Tests, or Experiments

###### a. Inspection Scope

The inspectors reviewed nine evaluations performed pursuant to 10 CFR 50.59. The evaluations related to permanent plant modifications, setpoint changes, procedure changes, conditions adverse to quality, and changes to the updated final safety analysis report. The inspectors confirmed that the evaluations were thorough and that prior NRC approval was obtained as appropriate. The inspectors also reviewed 12 screenings where the licensee had determined that a 10 CFR 50.59 evaluation was not necessary. In regard to the changes reviewed where no 10 CFR 50.59 evaluation was performed, the inspectors verified that the changes did not meet the threshold to require a 10 CFR 50.59 evaluation. These evaluations and screenings were chosen based on risk significance of samples from the different cornerstones.

###### b. Findings

No findings of significance were identified.

#### 1R17 Permanent Plant Modifications (71111.17)

##### .1 Review of Recent Permanent Plant Modifications

###### a. Inspection Scope

The inspectors reviewed 10 permanent plant modifications that were installed in the last two years. The modifications were chosen based upon their affecting systems that had high probabilistic risk analysis (PRA) significance in the licensee's Individual Plant Evaluation (IPE) or high maintenance rule safety significance. The inspectors reviewed the modifications to verify that the completed design changes were in accordance with the specified design requirements and the licensing bases and to confirm that the changes did not affect any systems' safety function. Design and post-modification testing aspects were verified to ensure the functionality of the modification, its

associated system, and any support systems. The inspectors also verified that the modifications performed did not place the plant in an increased risk configuration.

b. Findings

No findings of significance were identified.

**4. OTHER ACTIVITIES (OA)**

4OA2 Identification and Resolution of Problems

a. Inspection Scope

The inspectors reviewed a selected sample of condition reports associated with Bradwood's permanent plant modifications and concerning 10 CFR 50.59 evaluations and screenings to verify that the licensee had an appropriate threshold for identifying issues. The inspectors evaluated the effectiveness of the corrective actions for the identified issues.

b. Findings

No findings of significance were identified.

4OA6 Meetings

Exit Meeting

The inspectors presented the inspection results to Mr. K. Schwartz and other members of licensee management at the conclusion of the inspection on November 2, 2001. The licensee acknowledged the findings presented. No proprietary information was identified.



## KEY POINTS OF CONTACT

### Licensee

J. Bailey, Regulatory Assurance  
R. Belair, Rapid Response Team Engineer  
S. Butler, Regulatory Assurance  
G. Dudek, Operations Manager  
C. Dunn, Site Engineering Director  
A. Ferko, Regulatory Assurance Manager  
J. Harvey, Nuclear Oversight Manager  
F. Lentine, Design Engineering Manager  
J. Panici, Mechanical Design Engineer  
D. Riedinger, Electrical Design Engineer  
K. Schwartz, Station Manager  
J. Steele, Engineering Admin. Supervisor  
B. Viehl, Mechanical Design Supervisor  
R. Wolen, Rapid Response Team Supervisor

### NRC

C. Phillips, Senior Resident Inspector  
N. Shah, Resident Inspector

## LIST OF ACRONYMS USED

ADAMS	Agencywide Documents Access and Management System
CFR	Code of Federal Regulations
CR	Condition Report
DCP	Design Change Package
DPR	Demonstration Power Reactor
DRS	Division of Reactor Safety
IMC	Inspection Manual Chapter
IPE	Individual Plant Evaluation
IR	Inspection Report
LLC	Limited Liability Company
MCC	Motor Control Center
MOV	Motor Operated Valve
NEI	Nuclear Energy Institute
NRC	United States Nuclear Regulatory Commission
NRR	Office of Nuclear Reactor Regulation
OA	Other Activities
OAD	Operational Analysis Division
PARS	Publicly Available Records
PRA	Probabilistic Risk Analysis
QC	Quality Control
RG	Regulatory Guide
SDP	Significance Determination Process
TS	Technical Specification
UFSAR	Updated Final Safety Analysis Report
V	Volt

## LIST OF DOCUMENTS REVIEWED

The following is a list of licensee documents reviewed during the inspection, including documents prepared by others for the licensee. Inclusion on this list does not imply that NRC inspectors reviewed the documents in their entirety, but rather that selected sections or portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document in this list does not imply NRC acceptance of the document, unless specifically stated in the inspection report.

### **CALCULATIONS**

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<b><u>Number</u></b>	<b><u>Description</u></b>	<b><u>Rev/Date</u></b>
BRW-98-0006-E	Calculation for Voltage Drop in Breaker Closing Control Circuits for Various 4.16kV and 480V Loads	1
BRW-00-0010-M/BYR2000 -007	Spent Fuel Pool Temperature Analysis	000A
L-VE-403	Battery Room and Miscellaneous Electrical Equipment Room Division 12 & 22 Time Temperature Profile	2
SM-VD105	Diesel Generator Vent Fan (2)1VD03CB Differential Pressure Switch	D

### **CONDITION REPORTS (CRs) GENERATED PRIOR TO INSPECTION**

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<b><u>Number</u></b>	<b><u>Description</u></b>	<b><u>Rev/Date</u></b>
00003786	AIR 20-2-89-015; Replacement of All AR-3 Relays	March 5, 1999
A1999-03967	Numerous Plant Modifications Forced into Work Schedule	December 17, 1999
A2000-00838	IST Testing Discrepancy for Check Valves (2)1AF014A-H	February 24, 2000
A2000-01005	Drawing 20E-1-4206C Does Not Reflect Current Plant Configuration	March 1, 2000
A2000-02020	Problems Identified During Testing of Modification D20-0-98-307, RSH Level Indication Mod	April 25, 2000
A2000-02419	Design Change Testing Not Ready to Support Mod Package Completion	June 2, 2000
A2000-02439	D20-0-99-332 Modification Installation Put on Hold Due to Procedure Updates	June 5, 2000
A2000-02692	QC Identified Incorrect Hardware During Installation of DC Contactor	June 26, 2000

**CONDITION REPORTS (CRs) GENERATED PRIOR TO INSPECTION**

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<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
A2000-02944	Boron 10 Depletion Effects	July 18, 2000
A2000-03502	Nuclear Oversight Identifies 3 of 3 T-Mods Not Being Aggressively Monitored and Driven to Closure	September 5, 2000
A2000-03749	Procedure Revisions Needed as a Result of DG Modifications	September 27, 2000
A2001-00812	Approved UFSAR Change Package Not in Place for Installed Modification	March 13, 2001

**CONDITION REPORTS (CRs) GENERATED DURING INSPECTION**

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<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
00081275	UFSAR Figure and Drawing for RWST Vent Are Not Accurate	November 2, 2001
00081388	Deficient 10 CFR 50.59 Screening - Change to RCP Starting Procedure	November 2, 2001
00081432	Proposed DRP 9-049 Needs Clarification	November 2, 2001
00081439	Lab Ventilation (VL) Intake Mod/50.59 Deficiencies	November 2, 2001

**DRAWINGS**

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<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
20E-1-4008AB	Tabulation of Trip Settings 480V Auxiliary Building ESF MCC 132X4 (1AP28E) & 132X4A (1AP28EA) Part 1	AA
20E-1-4030AF02	Schematic Diagram Auxiliary Feedwater Pump 1B 1AF01PB	Y1
20E-1-4030SX01	Schematic Diagram Essential Service Water Pump 1A 1SX01PA	U
20E-1-4030SX02	Schematic Diagram Essential Service Water Pump 1B 1SX01PB	U
20E-1-4031RC01A	Loop Schematic Diagram $\Delta T/T$ Average Protection I Loop "1A" Protection Cabinet I (1PA01J) 1TE-0410A1, 0411A1, 0410A2, 0411A2, 0410A3, 0411A3, 0410B, 0411B) Part 1 of 4	D

**MODIFICATIONS:**

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<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
0000041391	Install Permanent Cooling to Unit 2 Rod Drive Cabinets	July 23, 1998
0000041408	Evaluate New High dp Trip and Alarm Setpoint for 2VD03CB	January 25, 1999
0000041461	Increase Water Removal Capacity for the Essential Service Water Pump Room Sumps	March 22, 2001
0000041529	AR3 Relay Replacement - 1SX01PA/B	September 17, 1999
0000041550	Add Manual Pressure Controllers of Main Steam Isolation Valve Accumulators	October 29, 1998
0000041587	Install Forged Blades on Auxiliary Building Ventilation (VA) Fans	December 7, 1999
0000042402	Permatize TMOD 99-1-013 for RTD 1TE-0411B	August 31, 1999
0000042509	Increase Speed of Fuel Handling Equipment	May 30, 2000
DCP 9700156	CNMT Charcoal Filter Unit Fan 1VP02CA Will Not Obtain 3.85"	September 5, 1998
DCP 9700219	Install Relief Devices on Containment Penetration	October 28, 1998
DCP 9800070	1DO020A-D Diesel Oil Transfer Pump Discharge Relief Valves	August 27, 2001
DCP 9800278	Replace SI Relief Valves	August 24, 1999
DCP 9900192	Upgrade Motor Operator for SI Valves	November 15, 1999
DCP 9900351	Change Diesel Generator 2A Starting Air Compressor Operating Range 02/15/1999	December 24, 2000
DCP DO-0-97-001	Redirect Diesel Exhaust Fumes Away From VL & VC Intakes	February 15, 1999

**MODIFICATION TESTS**

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<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
E20-1-96-288-001-01	1MS001A Accumulator Relief Modification Test	0

**MODIFICATION TESTS**

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<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
E20-1-96-288-001-02	1MS001D Accumulator Relief Modification Test	0
E20-0-96-302-007-1	Verify Proper Operation of Modification Including New Forged Blade Assembly, Inner-fairing Cover Plate and New Screen Fasteners	0
E-20-2-96-238-2	Rod Drive Spot Cooler Performance Verification Test	0

**PROCEDURES**

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<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
BwFP FH-12T2	Spent Fuel Pool Bridge Crane Outage Operational Checklist	9
BwMP 3215-001	Periodic Inspections and Repairs of Main Steam Isolation Valve Accumulators	7
BwOP MS-5	Main Steam Isolation Valve Accumulator Operability Check	16
BwOP SX-M2	Operating Mechanical Lineup for Essential Service Water System, Unit 2	16
CC-AA-102	Design Impact Screening	2
CC-AA-103	Design Change Package	1
CC-AA-107	Design Change Acceptance Testing Criteria	1
ECTP-19	OAD Electrical Construction Test Procedure for Modifications at Nuclear Stations #19 - Control Circuits	2
LS-AA-104	10 CFR 50.59 Safety Evaluation Process	1
LS-AA-104	Exelon 50.59 Review Process	2
LS-AA-104-1000	Exelon 50.59 Resource Manual	March 8, 2001
NES-E/I&C 10.01A	Molded Case Circuit Breaker Selection and Setting Design Standard	0
NES-E/I&C 10.01B	Molded Case Circuit Breaker Selection and Setting Design Standard	0
RS-AA-104	10 CFR 50.59 Safety Evaluation Process	0

## REFERENCES

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<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
IP 71111.02	NRC Inspection Procedure - Evaluation of Changes, Tests, or Experiments	September 12, 2000
IP 71111.17	NRC Inspection Procedure - Permanent Plant Modifications	September 12, 2000
NEI 96-07	Guidelines for 10 CFR 50.59 Implementation	January 1, 1900
Part 9900	NRC Inspection Manual - 10 CFR Guidance	March 13, 2001
RG 1.187	Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiments	November 2000

## SAFETY EVALUATIONS

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<u>Number</u>	<u>Description</u>	<u>Rev/Date</u>
BG 99-0128	Byron Evaluation of Modification to Operation of Non-Safety Related Containment Refueling Machine and Spent Fuel Pool Bridge Crane	October 18, 1999
BRW-E-2001-0311	Increase ESF Recirculation Loop Leakage Outside Containment	May 10, 2001
BRW-E-2001-0439	Change Incore Decay Time for A1R09	August 28, 2001
BRW-SE-1999-1150	AR3 Relay Replacement - 1SX01PA/B	September 21, 1999
BRW-SE-1999-1392	Revise Procedure BwOP RH-6 to Permit Operation Through the Cross-tie Header with the Associated Cold Leg MOV Closed	January 7, 2000
BRW-SE-2000-0011	Unit 1 Diesel Driven Auxiliary Feedwater Pump Main Control Board Control Switch Modification Test	January 4, 2000
BRW-SE-2000-0269	Remove Boron Dilution Protection System and Associated TS	April 1, 2000
BRW-SE-2000-1026	Aux. Bldg. Ventilation System - 4 fan operation	November 15, 2000
BRW-SE-2000-1258	UFSAR Change Package 9-024, Update NPSH Discussion for RH and CS Pumps	December 5, 2000

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## **SAFETY EVALUATIONS**

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<b><u>Number</u></b>	<b><u>Description</u></b>	<b><u>Rev/Date</u></b>
BRW-SE-2001-0214	Revised TS Bases to Clarify Accumulators' DB Assumptions	March 13, 2001
BRW-SESV-2000-0553	Braidwood Validation of Byron Evaluation on Spent Fuel Pool Bridge Crane	May 18, 2000

## **SAFETY SCREENINGS**

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<b><u>Number</u></b>	<b><u>Description</u></b>	<b><u>Rev/Date</u></b>
BRW-FCS-1999-0075	Change High dp Trip Setpoint for Fan 2VD03CB from 3.2" w.c. to 4.2" w.c.	January 21, 1999
BRW-FCS-1999-1100	Permatize TMOD 99-1-013 for RTD 1TE-0411B	August 31, 1999
BRW-PTES-2000-0006	Revision to SI ASME Procedure to Change Flow Acceptance Criteria	January 6, 2000
BRW-PTES-2000-0089	Revision to 1BwVSR 5.5.8.SX.2 to Reduce the Lower ASME Acceptance Criteria	January 24, 2000
BRW-PTES-2000-0426	Add Instruction to DG Room Temp BwAR to Provide Portable Ventilation and Open Rollup Door If Room Temperature Exceeds 122°	April 19, 2000
BRW-PTES-2000-1197	Removing Fire Protection System Battery Specific Gravity Requirements	November 30, 2000
BRW-PTES-2001-0106	Power Uprate Revised Containment Leak Rate Surveillances	February 1, 2001
BRW-S-2001-0297	Eliminate Requirement to Alternate Hot and Cold Leg Recirculation	April 26, 2001
BRW-S-2001-0328	Removal of Control Rod Drive (RD) System Stationary and Moveable Gripper Blocking Diodes	12:00 AM
BRW-S-2001-0376	Technical Specification Bases Change for Section B3.8.1	June 28, 2001
BRW-S-2001-0398	Bases Change Request #01-011 for Auxiliary Feedwater System	July 5, 2001
BRW-S-2001-0401	Remove Main Steam Safety Valve Lifting Levers	July 17, 2001



## **SAFETY SCREENINGS**

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<b><u>Number</u></b>	<b><u>Description</u></b>	<b><u>Rev/Date</u></b>
BRW-S-2001-0462	Install Taps and Isolation Valves to Enable Cross-Tie from Fire Protection System to the Essential Service Water System	August 9, 2001
BRW-S-2001-0497	Change Breaker Setting for 1SI8804B	September 11, 2001
BRW-S-2001-0513	Change UFSAR Section 9.3.3.2, Auxiliary Building Safety-Related Components Area Flood Analysis	September 18, 2001

## **SPECIAL TESTS**

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<b><u>Number</u></b>	<b><u>Description</u></b>	<b><u>Rev/Date</u></b>
SPP 00-21	Isolation and Draining of the Unit 2 Component Cooling Heat Exchanger	January 15, 2001

## **WORK REQUESTS**

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<b><u>Number</u></b>	<b><u>Description</u></b>	<b><u>Rev/Date</u></b>
990119774-01	Increase Speed of Fuel Handling Equipment per D20-099-364	October 11, 2000
00357926	1SI8804B Breaker Trips When Trying to Open the Valve	September 10, 2001

INITIAL DOCUMENT REQUEST  
(Sent to Licensee on 10/12/2001)

**Inspection Report:** 50-456-01-12; 50-457-01-12

**Inspection Dates:** October 29 - November 2, 2001

**Inspection Procedures:** IP 71111-02, "Changes, Tests, or Experiments"  
IP 71111-17, "Permanent Plant Modifications"

**Lead Inspector:** George M. Hausman, 630-829-9743

***Information Needed for In-Office Preparation Week***

The following information is needed by October 19, 2001, or sooner, to facilitate the selection of specific items that will be reviewed during the onsite inspection week. The team will select specific items from the information requested below and submit a list to your staff by October 24, 2001. We request that the specific items selected from the lists be available and ready for review on the first day of inspection. If you have any questions regarding this information, please call the team leader as soon as possible. All lists requested should cover a two year span. All information should be sent electronically (as PDF or Word documents) if at all possible.

- (1) List of "permanent plant modifications" to risk significant systems, structures or components. (For the purpose of this inspection, permanent plant modifications include permanent: plant changes, design changes, set point changes, procedure changes, equivalency evaluations, suitability analyses, calculations, and commercial grade dedications.)
- (2) List of all 10 CFR 50.59 completed evaluations involving: (a) changes to facility (modifications); (b) procedure revisions; (c) tests or non-routine operating configurations; (d) changes to the USFAR; (e) calculations.
- (3) List of all 10 CFR 50.59 screenings for changes that have been screened out as not requiring a full evaluation involving: (a) changes to facility (modifications); (b) procedure revisions; (c) tests or non-routine operating configurations; (d) changes to the USFAR; (e) calculations.
- (4) List of all 10 CFR 50.59 validations or applicability reviews done in lieu of a full 10 CFR 50.59 evaluation.
- (5) List of corrective action documents (open and closed) that address plant permanent modification or 10 CFR 50.59 issues or concerns.
- (6) List of special tests or experiments and non-routine operating configurations.
- (7) Copies of modification procedure(s), post modification testing procedure(s) and 10 CFR 50.59 procedures.

INITIAL DOCUMENT REQUEST  
(Sent to Licensee on 10/12/2001)

- (8) Name(s) and phone numbers for the regulatory and technical contacts.

***Information to be Available on First Day of Inspection (October 29, 2001)***

- (1) Copies of all selected items need to be ready for the inspectors when they arrive on site. Selected items will be relayed to the NRC coordinator/ contact person by October 24<sup>th</sup>.
- (2) The following information needs to be available to the team once they arrive onsite. Copies of these documents do not need to be solely available to the team as long as the inspectors have easy and unrestrained access to them.
- Updated Final Safety Analysis Report
  - Technical Specifications
  - Latest IPE/PRA Report
  - Vendor Manuals
  - The Latest 10 CFR 50.59 FSAR Update Submittal
- (3) Please ensure that all supporting documents for the selected items have been located and are readily retrievable as the team will likely be requesting these documents during the inspections. Examples of supporting documents are:
- Calculations supporting the selected modifications or 10 CFR 50.59 evaluations
  - Post-modification tests for the selected modifications
  - Copies of other evaluations or validations referred to by a modification or 10 CFR 50.59 evaluation
  - Drawings supporting the modifications
  - Procedures affected by the modifications
  - UFSAR change papers generated due to a 10 CFR 50.59 evaluation

***Information to be provided throughout the inspection***

- (1) Copies of any Condition Report generated as a result of the team's questions or queries during this inspection.
- (2) Copies of the list of questions submitted by the team members and the status/resolution of the information requested (provide daily during the inspection to each team member).