



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 231B5
ATLANTA, GEORGIA 30303-8931**

July 23, 2001

Mr. Dale E. Young,
Vice President,
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing &
Regulatory Programs
15760 West Power Line Street
Crystal River, FL 34428-6708

SUBJECT: ASSESSMENT FOLLOWUP - CRYSTAL RIVER 3 NUCLEAR PLANT

Dear Young:

On July 19, 2001, the NRC staff completed a review of the performance indicators of Crystal River 3 Nuclear Plant for the second quarter of 2001. The review identified that you had crossed the threshold from green to white for the Reactor Coolant System (RCS) leakage performance indicator. The increase in leakage, reported as a percentage of the Technical Specification limiting value, was identified by your staff while the plant was shutdown to correct a feedwater valve problem. A modification was completed to Decay Heat Valve (DHV)-3 which reduced the RCS leakage prior to the plant being restarted. The performance indicator was White for about one day.

In accordance with the Regulatory Response Column of the NRC's Reactor Oversight Process Agency Action Matrix, we plan to perform supplemental Inspection Procedure 95001 at your facility during the week of August 6, 2001, to review your actions to address the RCS leakage issue. We conducted baseline inspections to review some aspects of your corrective actions as they were implemented. An onsite meeting with your staff has been scheduled for August 9, 2001, to discuss the results of the inspections, the cause of the PI going white, and your plans for additional corrective actions.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Please contact me at 404-562-4540 with any questions you may have regarding this letter.

Sincerely,

/RA/

Leonard D. Wert, Chief
Reactor Projects Branch 3

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Division of Reactor Projects

Docket No. 50-302
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