

The potential environmental impacts of the proposed action that will be identified and evaluated in detail in the comprehensive environmental evaluation include:

- Physical disturbance to the snow and ice environment
  - Air emissions
  - Releases to the snow and environment
  - Impacts to Amundsen-Scott Station operations
  - Impacts to other science at the South Pole or in other areas of the USAP
- Selected mitigating measures, representing specific actions or options that would be taken to reduce or avoid impacts to the environment, have already been incorporated into the design of the proposed Project. These mitigating measures will be identified in the comprehensive environmental evaluation, as well as additional measures that will be under consideration during the implementation of the Project activities.

The public is invited to comment on any aspect of the proposal. The comment period on the draft comprehensive environmental evaluation will be a minimum of 90 days from the date the Environmental Protection Agency publishes the notice of availability in the **Federal Register**.

**Polly A. Penhale,**

*Program Manager.*

[FR Doc. 03-23856 Filed 9-17-03; 8:45 am]

**BILLING CODE 7555-01-M**

## NUCLEAR REGULATORY COMMISSION

[Docket No. 50-400]

### Carolina Power & Light Company; Notice of Partial Withdrawal of Application for Amendment to Facility Operating License

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Carolina Power & Light Company, *et al.* (the licensee) to withdraw a portion of its August 28, 2002, application for proposed amendment to Facility Operating License No. NPF-63 for the Shearon Harris Nuclear Power Plant, Unit 1, located in Wake and Chatham Counties, North Carolina.

The withdrawn portion of the proposed amendment would have revised Technical Specification 6.9.1.6.2 by including Topical Report EMF-2310(P)(A), "SRP [Standard Review Plan] Chapter 15 Non-LOCA [loss-of-coolant accident] Methodology for Pressurized-Water Reactors," as a

reference methodology used to determine core operating limits at Shearon Harris Nuclear Power Plant, Unit 1.

The other portion of the amendment application, which requested approval of topical report EMF-2328(P)(A), "PWR [pressurized-water reactor] Small-Break LOCA Evaluation Model, S-RELAP5-Based," as a reference in the TS, was approved and issued as Amendment No. 114 on March 28, 2003 (68 FR 18291, April 15, 2003).

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the **Federal Register** on October 15, 2002 (67 FR 63691). However, by letter dated August 28, 2003, the licensee withdrew the portion of the proposed change described above.

For further details with respect to this action, see the application for amendment dated August 28, 2002, and the licensee's letter dated August 28, 2003, which withdrew this portion of the application for license amendment. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Electronic Reading Room on the internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams/html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff by telephone at 1-800-397-4209 or 301-415-4737, or by email to [pdrc@nrc.gov](mailto:pdrc@nrc.gov).

Dated at Rockville, Maryland, this 11th day of September, 2003.

For the Nuclear Regulatory Commission.

**Chandu P. Patel,**

*Project Manager, Section 2, Project Directorate II, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.*

[FR Doc. 03-23839 Filed 9-17-03; 8:45 am]

**BILLING CODE 7590-01-U**

## NUCLEAR REGULATORY COMMISSION

[Docket No. 50-390]

### Tennessee Valley Authority; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-90 issued to Tennessee Valley Authority (the licensee) for operation of the Watts Bar Nuclear Plant, Unit 1 (WBN), located in Rhea County, Tennessee.

The proposed amendment would revise the Updated Final Safety Analysis Report to change the postulated primary-to-secondary leakage from a faulted steam generator in the main steamline break (MSLB) accident analysis.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in Title 10 of the Code of Federal Regulations (10 CFR), Section 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

No. The postulated MSLB outside of containment but upstream of the main steam isolation valves is the limiting accident relative to the voltage based alternate repair criteria for axial outside diameter stress corrosion cracking (ODSCC). It is the credible accident for determining the radiological consequences of increasing the postulated primary-to-secondary leakage. The leakage is an input parameter and does not physically alter any equipment, system performance, or operator actions required to mitigate the radiological consequences of an accident.