

September 25, 2000

Mr. M. Reddemann
Site Vice President
Kewaunee and Point Beach Nuclear Plants
Wisconsin Electric Power Company
6610 Nuclear Road
Two Rivers, WI 54241

SUBJECT: POINT BEACH - NRC INSPECTION REPORT 50-266/2000011(DRS);
50-301/2000011(DRS)

Dear Mr. Reddemann:

On September 14, 2000, the NRC completed a routine inspection at the Point Beach Nuclear Plant. The enclosed report presents the results of that inspection. The results were discussed on September 14, 2000, with Mr. Mende and other members of your staff.

The inspection was an examination of activities conducted under your license as they relate to radiation safety and to compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection consisted of a selective examination of representative records, tours of your facility and interviews with personnel.

Based on the results of this inspection, no findings were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

We will gladly discuss any question you have concerning this inspection.

Sincerely,

/RA/

Gary L. Shear, Chief
Plant Support Branch
Division of Reactor Safety

Docket Nos. 50-266; 50-301
License Nos. DPR-24; DPR-27

Enclosure: Inspection Report 50-266/2000011(DRS);
50-301/2000011(DRS)

cc w/encl: R. Grigg, President and Chief
Operating Officer, WEPCo
M. Wadley, Chief Nuclear Officer, NMC
J. Gadzala, Licensing Manager
S. Cartwright, Nuclear Asset Manager
F. Cayia, Plant Manager
J. O'Neill, Jr., Shaw, Pittman,
Potts & Trowbridge
K. Duveneck, Town Chairman
Town of Two Creeks
B. Burks, P.E., Director
Bureau of Field Operations
A. Bie, Chairperson, Wisconsin
Public Service Commission
S. Jenkins, Electric Division
Wisconsin Public Service Commission
State Liaison Officer

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos: 50-266; 50-301
License Nos: DPR-24; DPR-27

Report No: 50-266/2000011(DRS); 50-301/2000011(DRS)

Licensee: Wisconsin Electric Power Company

Facility: Point Beach Nuclear Plant, Units 1 and 2

Location: 6610 Nuclear Road
Two Rivers, WI 54241

Dates: September 11-14, 2000

Inspector: D. Nelson, Radiation Specialist

Approved by: Gary L. Shear, Chief
Plant Support Branch
Division of Reactor Safety

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

- | Reactor Safety | Radiation Safety | Safeguards |
|---|---|---|
| <ul style="list-style-type: none">● Initiating Events● Mitigating Systems● Barrier Integrity● Emergency Preparedness | <ul style="list-style-type: none">● Occupational● Public | <ul style="list-style-type: none">● Physical Protection |

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: <http://www.nrc.gov/NRR/OVERSIGHT/index.html>.

SUMMARY OF FINDINGS

IR 50-266/2000011(DRS); 50-301/2000011(DRS), on 09/11-09/14/00; Wisconsin Electric Power Company; Point Beach Nuclear Plant; Units 1 and 2.

The inspection was conducted by a regional radiation specialist. There were no findings identified.

Report Details

Summary of Plant Status: The plant was at full power during the inspection period.

2. RADIATION SAFETY

Cornerstone: Occupational Radiation Safety

2OS3 Radiation Monitoring Instrumentation

.1 Source Tests and Calibration of Radiological Instrumentation

a. Inspection Scope

The inspector verified that radiological instruments associated with transient high and very high radiation areas (area radiation monitors (ARM)), and instruments used for coverage of high radiation work and/or for air monitoring for jobs with the potential for workers to receive greater than 100 millirem committed effective dose equivalent (CEDE), had been properly calibrated and their alarm set-points (if applicable) properly set. The inspector verified that selected ARMs (spent fuel pool high range, containment high range, and primary side sample room high range) had been appropriately calibrated, and function and operation tested in 1999 and 2000. The inspector reviewed the calibration procedures and year 2000 calibration records to verify that selected portable radiation survey instruments (RSO-50), selected portable continuous air monitors (AMS-3), and selected portable teledosimeters (Teletector) had been properly calibrated. The inspector also reviewed the calibration procedures and year 2000 calibration records for the whole body counter (Canberra Fastscan WBC), the small articles monitor (SAM-9/11) and selected contamination monitors (PCM-1B) to verify that they had been properly calibrated. The inspector observed the calibration of an RSO-50 portable survey instrument, a Teletector Model 6112, a Rados RAD-51, and an area monitor (RE-110) to verify that the instruments were calibrated in compliance with the appropriate procedures. The inspector also observed the functional check of a PCM-1B personnel contamination monitor to verify compliance with the appropriate procedure. In addition, the inspector observed several radiation protection (RP) technicians source check portable radiation survey instruments to verify compliance with procedures.

b. Findings

There were no findings identified.

.2 Radiation Protection Technician Instrument Use

a. Inspection Scope

The inspector verified compliance with NP 4.2.3.1, "Portable Survey Instrument and Exposure Device Control Program," by observing several RP technicians' selection and operational checks of portable radiation survey instruments used for radiation protection (RP) technician job coverage.

b. Findings

There were no findings identified.

.3 Self-Contained Breathing Apparatus (SCBA) Program

a. Inspection Scope

The inspector reviewed EPMP 1.1, "Routine Check, Maintenance, Calibration, and Inventory Schedule for Radiation Protection Emergency Preparedness Equipment," Revision 1, and verified the adequacy of the program to provide SCBA for unknown or emerging conditions. The inspector walked down the available equipment, reviewed the status and surveillance records of SCBA staged for use in the plant, verified the licensee's capability for refilling and transporting SCBA bottles to the control room and support locations in the plant, verified the training, medical, fit test, and qualification records of selected individuals in 2000, and reviewed the licensee's actions relative to Information Notices 98-20 and 99-05 (no specific response was required by the notices).

b. Findings

There were no findings identified.

.4 Identification and Resolution of Problems

a. Inspection Scope

The inspector reviewed all year 2000 Condition Reports (CR) that addressed radiation instrument deficiencies. The review determined if any significant radiological incidents involving radiation instrument deficiencies had occurred during the year 2000. The review was conducted to verify that the licensee had effectively implemented the corrective action program.

b. Findings

There were no findings identified.

4. OTHER ACTIVITIES

4OA6 Management Meetings

Exit Meeting Summary

The inspector presented the inspection results to Mr. Mende, and other members of licensee management and staff, in an exit meeting on September 14, 2000. The licensee acknowledged the information and findings presented. No proprietary information was identified by the licensee.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

A. Cayia, Manager, Site Services & Assessment
F. Flentje, Senior Regulatory Compliance Specialist
R. Mende, Plant Manager
D. Shannon, Radiation Protection Supervisor
S. Thomas, Radiation Protection Manager
J. Walsh, Manager, Procedures Programs

NRC

R. Powell, Resident Inspector

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None

Closed

None

Discussed

None

LIST OF ACRONYMS USED

| | |
|------|------------------------------------|
| ARM | Area Radiation Monitor |
| CR | Condition Report |
| ED | Electronic Dosimeter |
| RP | Radiation Protection |
| SCBA | Self-Contained Breathing Apparatus |

LIST OF DOCUMENTS REVIEWED

The following is a list of licensee documents reviewed during the inspection. Inclusion on this list does not imply that NRC inspectors reviewed the documents in their entirety, but, rather that selected sections or portions of the documents were evaluated as part of the overall inspection effort.

Condition Reports

00-0006, 00-0594, 00-0742, 00-0779, 00-0783, 00-0846, 00-1076, 00-1837, 00-2056, 00-2258, 00-2666, 00-2720.

Station Procedures

| | |
|-------------------------|--|
| Canberra ACK#61292 | Calibration of the Canberra Fastscan WBC System at the Point Beach Nuclear Plant, May 18, 2000 |
| EPMP 1.1 (Revision 26) | Routine Check, Maintenance, Calibration and Inventory Schedule for Radiation Protection Emergency, July 26, 2000 |
| HPCAL 1.10 (Revision 7) | Operation of J. L. Shepherd Model 89 Calibrator, April 29, 1999 |
| HPCAL 1.11 (Revision 9) | Calibration of the Teletector Model 6112 Portable Survey Instrument, October 19, 1993 |
| HPCAL 1.27 (Revision 7) | Calibration of Bicron RSO-50 Ion Chamber, February 25, 1999 |
| HPCAL 1.35 (Revision 3) | Calibration of the Rados RAD-51 Dosimeter, May 28, 1998 |
| HPCAL 2.15 (Revision 7) | Small Articles Monitor Type SAM-9/11 Calibration and Efficiency, May 19, 2000 |
| HPCAL 2.8 (Revision 12) | Personnel Contamination Monitor PCM-1B Calibration Procedure, September 3, 1998 |
| HPCAL 3.2 (Revision 16) | Area Monitor Calibration Procedure DA1-1 and DA 1-6 Detector Assemblies, July 7, 2000 |
| HPCAL 3.11 (Revision 9) | Containment High Range Detector Response Check, November 11, 1999 |
| HPIP 3.57 (Revision 8) | Calibration of the Eberline Beta Air Monitor Model AMS-3, March 26, 1998 |
| HPIP 7.52 (Revision 10) | Personnel Contamination Monitor (PCM-1B) Functional Check, January 27, 2000 |
| NP 1.2.3 (Revision 7) | Temporary Procedure Changes, November 8, 1999 |
| NP 4.2.3.1 (Revision 1) | Portable Survey Instrument and Exposure Monitoring Device Control Program, July 26, 2000 |
| NP 10.3.1 (Revision 12) | Authorization of Changes, Tests, and Experiments (10 CFR 50.59 and 72.48 Reviews, July 14, 1999) |

Calibration Records

| | |
|-----------------------------------|---------------------------------------|
| SAM-9/11 | Instru No 2 (May 31, 2000) |
| Teletector Model 6112 | Instru No 2858 (September 12, 2000) |
| PCM-1B | Instru No 7739 (May 8, 2000) |
| PCM-1B | Instru No 7738 (April 12, 2000) |
| Bicron RSO-50 | Instru No 7859 (September 12, 2000) |
| Rados RAD-51 Electronic Dosimeter | Instru No 951267 (September 12, 2000) |
| Spent Fuel Pool Area Monitor | Instru No RE-135 (February 19, 1999) |
| Spent Fuel Pool Area Monitor | Instru No RE-135 (February 21, 1998) |
| U1 Sample Room Monitor | Instru No RE-136 (February 20, 1999) |
| U1 Sample Room Monitor | Instru No RE-136 (April 10, 2000) |
| U2 Sample Room Monitor | Instru No RE-136 (February 20, 1999) |
| U2 Sample Room Monitor | Instru No RE-136 (April 11, 1999) |
| U1Cont High Range Monitor | Instru No 1RE-126 (April 17, 1998) |
| U1Cont High Range Monitor | Instru No 1RE-126 (October 21, 1999) |
| U2 Cont High Range Monitor | Instru No 2RE-126 (December 19, 2000) |
| U2 Cont High Range Monitor | Instru No 2RE-126 (January 22, 1999) |
| SI Pump Room Low Range | Instru No RE-110 (September 13, 2000) |
| Canberra FASTSCAN | May 16, 2000 |

Forms

| | |
|-------------------------|--|
| EPMP-1.1a (Revision 12) | Emergency Preparedness Respiratory Monthly Visual Inspection Checklist, March 3, 2000 |
| EPMP-1.1b (Revision 13) | Radiation Protection Preparedness Quarterly Checklist, July 26, 2000 |
| PBF-4004 (Revision 1) | Portable Instrument Calibration Worksheet |

Other Documents

Spreadsheet Printouts for SCBA Training, Medical and Fit Test Records

Source #10208 Dose Rates

Database Printouts for Active RP Callups (Calibration Due Dates and Calibration Completion Dates)