

May 3, 2002

Mr. John L. Skolds, President
Exelon Nuclear
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: BYRON STATION, UNITS 1 AND 2
NRC INSPECTION REPORT 50-454/02-04(DRS); 50-455/02-04(DRS)

Dear Mr. Skolds:

On April 19, 2002, the NRC completed an inspection at your Byron Station, Units 1 and 2. The enclosed report documents the inspection findings which were discussed on April 19, 2002, with Mr. Lopriore and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel. Specifically, this inspection focused on the baseline annual inspection for evaluations of changes, tests, or experiments (10 CFR 50.59) and the baseline biennial inspection for permanent plant modifications.

No findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Ronald N. Gardner, Chief
Electrical Engineering Branch
Division of Reactor Safety

Docket Nos. 50-454; 50-455
License Nos. NPF-37; NPF-66

Enclosure: Inspection Report 50-454/02-04(DRS);
50-455/02-04(DRS)

See Attached Distribution

J. Skolds

-2-

cc w/encl: Site Vice President - Byron
Byron Station Plant Manager
Regulatory Assurance Manager - Byron
Chief Operating Officer
Senior Vice President - Nuclear Services
Senior Vice President - Mid-West Regional
Operating Group
Vice President - Mid-West Operations Support
Vice President - Licensing and Regulatory Affairs
Director Licensing - Mid-West Regional Operating Group
Manager Licensing - Braidwood and Byron
Senior Counsel, Nuclear, Mid-West Regional
Operating Group
Document Control Desk - Licensing
M. Aguilar, Assistant Attorney General
Illinois Department of Nuclear Safety
State Liaison Officer
State Liaison Officer, State of Wisconsin
Chairman, Illinois Commerce Commission

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50-455/02-04(DRS)

See Attached Distribution

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cc w/encl: Site Vice President - Byron
Byron Station Plant Manager
Regulatory Assurance Manager - Byron
Chief Operating Officer
Senior Vice President - Nuclear Services
Senior Vice President - Mid-West Regional
Operating Group
Vice President - Mid-West Operations Support
Vice President - Licensing and Regulatory Affairs
Director Licensing - Mid-West Regional Operating Group
Manager Licensing - Braidwood and Byron
Senior Counsel, Nuclear, Mid-West Regional
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REGION III

Docket Nos: 50-454; 50-455
License Nos: NPF-37; NPF-66

Report No: 50-454/02-04(DRS); 50-455/02-04(DRS)

Licensee: Exelon Generation Company, LLC

Facility: Byron Station, Units 1 and 2

Location: 4450 N. German Church Road
Byron, IL 61010

Dates: April 15 through 19, 2002

Inspectors: George M. Hausman, Senior Reactor Inspector (Lead)
Kenneth G. O'Brien, Senior Reactor Inspector
Darrell L. Schrum, Reactor Inspector
Hershell A. Walker, Senior Reactor Inspector

Observer: James E. Neurauter, Reactor Inspector (Trainee)

Approved by: Ronald N. Gardner, Chief
Electrical Engineering Branch
Division of Reactor Safety

SUMMARY OF FINDINGS

IR 05000454-02-04(DRS), IR 05000455-02-04(DRS), on 04/15-19/2002; Exelon Generation Company, LLC; Byron Station, Units 1 and 2. Evaluations of Changes, Tests, or Experiments and Permanent Plant Modifications Report.

The report covers a five day announced inspection. The inspection was conducted by four Region III based reactor inspectors and one observer in training. No findings of significance were identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using IMC 0609 "Significance Determination Process" (SDP). The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described at its Reactor Oversight Process website at <http://www.nrc.gov/NRR/OVERSIGHT/index.html>. Findings for which the SDP does not apply are indicated by "No Color" or by the severity level of the applicable violations.

A. Inspector Identified Findings

Cornerstones: Initiating Events, Mitigating Systems, Barrier Integrity

No findings of significance were identified.

B. Licensee identified Findings

No findings of significance were identified.

Report Details

Summary of Plant Status

Units 1 and 2 were operated at or near full power throughout the inspection period.

1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity

1R02 Evaluations of Changes, Tests, or Experiments (71111.02)

.1 Review of 10 CFR 50.59 Evaluations and Screenings for Changes, Tests, or Experiments

a. Inspection Scope

The inspectors reviewed ten evaluations performed pursuant to 10 CFR 50.59. The evaluations related to permanent plant modifications, set-point changes, procedure changes, conditions adverse to quality, and changes to the updated final safety analysis report. The inspectors reviewed the evaluations to verify that the evaluations were thorough and that prior NRC approval was obtained as appropriate. The inspectors also reviewed fifteen screenings where the licensee had determined that a 10 CFR 50.59 evaluation was not necessary. In regard to the changes reviewed where no 10 CFR 50.59 evaluation was performed, the inspectors verified that the changes did not meet the threshold to require a 10 CFR 50.59 evaluation. These evaluations and screenings were chosen based on risk significance of samples from the different cornerstones.

b. Findings

No findings of significance were identified.

1R17 Permanent Plant Modifications (71111.17)

.1 Review of Recent Permanent Plant Modifications

a. Inspection Scope

The inspectors reviewed ten permanent plant modifications that were installed in the last two years. The modifications were chosen based upon their affecting systems that had high probabilistic risk analysis (PRA) significance in the licensee's Individual Plant Evaluation (IPE) or high maintenance rule safety significance. The inspectors reviewed the modifications to verify that the completed design changes were in accordance with the specified design requirements and the licensing bases and to confirm that the changes did not affect any systems' safety function. Design and post-modification testing aspects were reviewed to ensure the functionality of the modification, its

associated system, and any support systems. The inspectors also reviewed the modifications performed to verify the modification did not place the plant in an increased risk configuration.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES (OA)

4OA2 Identification and Resolution of Problems

a. Inspection Scope

The inspectors reviewed a selected sample of corrective action documents associated with the licensee's permanent plant modifications and concerning 10 CFR 50.59 evaluations and screenings to verify that the licensee had an appropriate threshold for identifying issues. The inspectors evaluated the effectiveness of the corrective actions for the identified issues.

b. Findings

No findings of significance were identified.

4OA6 Meetings

Exit Meeting

The inspectors presented the inspection results to Mr. Lopriore and other members of licensee management at the conclusion of the inspection on April 19, 2002. The licensee acknowledged the findings presented. No proprietary information was identified.

KEY POINTS OF CONTACT

Excelon Nuclear, LLC

B. Adams, Engineering
K. Ainger, Mid-West Regional Operating Group - Regulatory Affairs
B. Altman, Maintenance Manager
E. Blondin, Design Engineering
R. Choinard, Regulatory Assurance
G. DeBoo, Design Engineering (Corporate)
D. Drawbaugh, NRC Coordinator
D. Flowers, Engineering
B. Grundmann, Regulatory Assurance Manager
K. Hansing, Nuclear Oversight Manager
M. Heinzer, Nuclear Oversight
D. Hoots, Operations Manager
D. Kerr, Engineering
S. Kerr, Chemistry Manager
B. Kolo, Work Control Manager
R. Lopriore, Site Vice President
J. Lubbat, Design Engineering
R. Mancini, Engineering
P. O'Neill, Material Management Supervisor
K. Passmore, System Engineering
B. Perchiazzi, Engineering
D. Prisby, Procurement Engineering
R. Randels, Design Engineering Manager
T. Roberts, Engineering Manager
D. Robinson, Fire Protection Engineer
B. Sambito, Radiation Protection
R. Schmidt, Design Engineering
M. Shah, Design Engineering

NRC

R. Skokowski, Senior Resident Inspector
P. Snyder, Resident Inspector

LIST OF ACRONYMS USED

ADAMS	Agency-wide Documents Access and Management System
A/E	Architect/Engineer
AFW	Auxiliary Feedwater
ANSI	American National Standards Institute
AOP	Abnormal Operating Procedure
AOT	Allowed Outage Time
ASME	The American Society of Mechanical Engineers
ATWS	Anticipated Transient Without Scram
BAR	Byron Alarm Response
BTP	Branch Technical Position
CCW	Component Cooling Water
CFR	Code of Federal Regulations
CMEB	Chemical Engineering Branch
CV	Chemical and Volume Control
CVCS	Chemical and Volume Control System
dc	Direct Current
DPR	Demonstration Power Reactor
DRS	Division of Reactor Safety
EC	Engineering Change
ECR	Engineering Change Request
EDG	Emergency Diesel Generator
EOP	Emergency Operating Procedure
EPN	Equipment Part Number
EQ	Environmental Qualification
ESFAS	Engineered Safety Features Actuation System
F	Fahrenheit
FSAR	Final Safety Analysis Report
FW	Feedwater
GL	Generic Letter
gov	Government
gpm	Gallons Per Minute
HELB	High Energy Line Break
html	Hypertext Markup Language
http	Hypertext Transfer Protocol
HVAC	Heating Ventilation and Air Conditioning
HX	Heat Exchanger
IA	Instrument Air
IEEE	The Institute of Electrical and Electronics Engineers
IMC	Inspection Manual Chapter
IP	Inspection Procedure
IPE	Individual Plant Evaluation
IR	Inspection Report
IST	In-Service Testing
LLC	Limited Liability Company
LOV	Loss of Voltage

LIST OF ACRONYMS USED

MCC	Motor Control Center
MFW	Main Feedwater
MOV	Motor Operated Valve
MWROG	Mid-West Regional Operating Group
NEI	Nuclear Energy Institute
NON	Nuclear Operations Notification
NRC	United States Nuclear Regulatory Commission
NRR	Office of Nuclear Reactor Regulation
OA	Other Activities
PARS	Publicly Available Records
PRA	Probabilistic Risk Analysis
QA	Quality Assurance
QC	Quality Control
QL-1	Quality Level 1
RG	Regulatory Guide
RHR	Residual Heat Removal
RPS	Reactor Protection System
RCP	Reactor Coolant Pump
RCS	Reactor Coolant System
RWST	Refueling Water Storage Tank
SD	Shutdown
SDP	Significance Determination Process
SE	Safety Evaluation
SER	Safety Evaluation Report
SFP	Spent Fuel Pool
SI	Safety Injection
SSFI	Safety Systems Functional Inspection
SW	Service Water
SX	Essential Service Water
TDAFW	Turbine Driven Auxiliary Feedwater
TMOD	Temporary Modification
TRM	Technical Requirements Manual
TS	Technical Specification
USAR	Updated Final Safety Analysis Report
USI	Unresolved Safety Issue
V	Volt
VCT	Volume Control Tank
WO	Work Order
wpd	WordPerfect Document
WS	Service Water
www	World Wide Web

LIST OF DOCUMENTS REVIEWED

The following is a list of licensee documents reviewed during the inspection, including documents prepared by others for the licensee. Inclusion on this list does not imply that NRC inspectors reviewed the documents in their entirety, but rather that selected sections or portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document in this list does not imply NRC acceptance of the document, unless specifically stated in the inspection report.

CALCULATIONS

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
7.16.10- BMW-98-0136	To Design the Anchorage Details for the Spare Battery Racks 1/2DC10EA and 1/2DC11EA	February 18, 1998
BMW-96-398-E	Endurance Evaluation of Chemical and Volume Control Pumps	September 30, 1996

CONDITION REPORTS (CRs) GENERATED DURING INSPECTION

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
00103518	EC Supporting ECR Was Not Requested for Installed EPNs	April 12, 2002
00103770	EC 332388 Screening 6E-01-0175 Needs Editorial Clarification	April 15, 2002
00103781	EC Configuration Control Concerns/Enhancements	April 15, 2002
00104005	Safety Evaluation 6G-97-0277 Rev 1, NRC Identified Concern	April 16, 2002
00104149	Potentially Non-Conservative Methodology Used to Support EC	April 17, 2002
00104342	Identified Concerns for 50.59 Screening for Procedure BARs	April 18, 2002
00104368	Discrepancy Noted for Screening 6D-00-0716	April 18, 2002
00104393	Proceduralized TCC Controls Not Incorporated in WOs	April 18, 2002
00104470	CV Pump Potential Seal Concern During Postulated High Temps	April 19, 2002

CONDITION REPORTS (CRs) GENERATED PRIOR TO INSPECTION

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
AR00076696	Ventilation Barrier is Beyond Design Accident	September 27, 2001
AR00078478	Unauthorized Temporary Modification Installed at Air Dampers	October 10, 2001
AR00080266	Unauthorized Temporary Modification Installed on Door 0DSSD171	October 24, 2001
AR00083623	2A Diesel Generator Prints Not Updated 6 Months after Governor Modification Was Installed	November 21, 2001
AR00086148	Problems Experienced with TMOD Implementation for U1 Essential Service Water	December 12, 2001

CONDITION REPORTS (CRs) GENERATED PRIOR TO INSPECTION

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
AR00087306	Inadequate 50.59 Screening Documentation for 1BVSR SX-8 Revision	December 18, 2001
AR00093890	Unapproved Temporary Modification Installed at 0VS03C Plenum Doors	February 5, 2002
AR00095601	A/E Used Incorrect Design Criteria and Engineering Analysis for Calculation	February 18, 2002
AR00096206	An Installed Concrete Expansion Anchor Was Torqued to the Wrong Torque Value for the Application	February 21, 2002
AR00096406	Problems Encountered with the Installation of the Electrical Modification for the CV/RH Letdown Booster Pump	February 22, 2002
AR00096933	Wrong Spring Hanger Was Installed on 1HD53018V	February 27, 2002
AR00097242	The Grouting of the Equipment Base for Pump 1CV03P Was Not Complete	February 28, 2002
AR00098118	ACE to Document Apparent Cause of 1MS01031R and 1MS01032 Support Damage	April 2, 2002
AR00100447	Potential Unqualified Coatings in Unit 1 Containment	March 22, 2002
B2000-01501	Improper Use of the 50.59 and out of Service Processes	May 24, 2000
B2000-01769	Calculation Error	June 22, 2001
B2000-01798	Positive Displacement Pump(s) Referenced as an Available Source in Flood Calculation 3C8-0787-001	June 26, 2000
B2000-02299	Valve 1SX2199A Was Installed in a Different Orientation From the Engineering Design	August 21, 2000
B2000-02426	High Energy Line Break Design Basis for Auxiliary Feedwater Diesel Not in Updated Final Safety Analysis Report	August 28, 2000
B2000-02914	Lack of Control for Spent Fuel Pool Temporary Pumps	October 2, 2000
B2000-02915	Failure to Follow Procedures When Transferring Spent Fuel Pool Water	October 3, 2000
B2000-03270	2B DG "B" Air Dryer Failed to Meet the Dew Point Acceptance Criteria in the Post Modification Test	October 23, 2000
B2001-01370	Concern That the Effects of a Freeze Seal Failure Are Not Being Performed or Verified as Completed	March 26, 2001
B2001-01633	Unexpected Response on 2A Diesel Generator During Cooldown While Executing SPP 00-026	April 21, 2001
B2001-03111	Questionable Auxiliary Feed Pump Operability Due to Low Temperatures	July 16, 2001

DRAWINGS

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
6E1-4-3-CC04	Component Cooling Water from Reactor Coolant Pumps Thermal Barrier Isolation Valves 1CC9438 and 1CC685	September 11, 2000

ENGINEERING CHANGES (ECs)

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
078346	Replace Westinghouse Source Range System with Gamma-Metrics (DCP 9800195 & 9800196)	7
332356	Installation of Letdown Booster Pump, Unit 1	0
332388	Allow Station Auxiliary Diesel (Security Diesel) Ventilation Damper/louver to Open upon a Demanded Start Signal	August 29, 2001
334199	Temporary Staging of Lead Shielding Blankets Inside Unit 1 Containment Prior to B1R11	March 1, 2002
334978	Authorizes the Temporary Removal of up to One-Half of the Flange Bolts for Valves 1/2SX001A and B with the SX Train in Service	February 7, 2002
335840	Authorizes the Temporary Removal of up to One-Half of the Flange Bolts for Valve 1SX143B (Every Other Bolt Removed) with the SX Train in Service	March 8, 2002
335856	Redesign Support 1MS01031R	1

MODIFICATIONS (DCPs):

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
9700622	Convert MOV 1CC685 from a Torque Switch Close to a Limit Switch Close Design	1
9900020	Change the Orientation of Relief Valve 0CC9432 to Provide Protection to the CC - RHR Return Header	0
9900207	Alternate Cooling/Seal Water Supply to the Service Water Pumps	July 29, 1999
9900266	BDPS Replacement with VCT Level Monitoring	0
9900281	Increase the Speed of the Fuel Handling Equipment	October 19, 1999
9900291	Drill Hole in Valve 1CV8804A Disc to Prevent Pressure Lock	January 26, 2000
9900292	Drill Hole in Valve 2CV8804A Disc to Prevent Pressure Lock	January 26, 2000
9900375	Provide Alternate Cooling to 2A CV Charging Pump for Diesel Generator Allowed Outage Time	September 11, 2000
9900460	U-1 Add Two-Cell Battery Rack to Battery Room 111	April 7, 2000
9900510	Replacement of PSA Mechanical Snubber 1CC36030S with LISEGA Hydraulic Snubber	0
9900529	Replacement of Four CC Snubbers with Hydraulic Snubbers	0

MODIFICATIONS (DCPs):

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
9900560	Removal and Replacement of Level Control Valve 1HD026C Between Feedwater Heater 12C and Flash Tank 1HD02TC	July 10, 2000
9900604	Abandonment of Fire Protection Foam System to Outdoor Diesel Oil Storage Tanks.	October 6, 2000
9900789	Auxiliary Feedwater to Steam Generator Flow Control for Power Up-Rate	0
9900827	Isolate Instrument Air to Essential Service Water Valves 2SX173 and 2SX178 During Normal Operation	0
9900828	Isolate Instrument Air to Essential Service Water Valves 1SX173 and 1SX178 During Normal Operation	0

PROCEDURES

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
1B0A PRI-7	Essential Service Water Malfunction Unit 1	100
BAR 0-37-A4	Unit 1 Area Fire Alarm Response	3, 4, & 6
BAR 0-39-A4	Unit 2 Area Fire Alarm Response	3, 4, & 6
BOP RC-16	Operation of the Loop Stop Bypass Valves	4
BOP CV-17	Establishing and Securing Normal and RH Letdown Flow	12
BOP RH-9	Pump Down of the Refueling Cavity to the Reactor Water Storage Tank	18
CC-AA-10	Configuration Control Process Description	1
CC-AA-102	Design Impact Screening	2
CC-AA-103	Design Change Package	1
CC-AA-104	Configuration Change Closeout	2
CC-AA-107	Design Change Acceptance Testing Criteria	1
CC-AA-112	Temporary Configuration Changes	5
CC-AA-309	Control of Design Analyses	1
LS-AA-104	Exelon 50.59 Review Process	2
LS-AA-104-1000	Exelon 50.59 Resource Manual	1
LS-AA-104-1001	50.59 Review Cover Sheet Form	0
LS-AA-104-1002	50.59 Applicability Review Form	0
LS-AA-104-1003	50.59 Screening Form	0
LS-AA-104-1004	50.59 Evaluation Form	1
LS-AA-104-1005	50.59 Validation Form	0
LS-AA-104-1006	Exelon 50.59 Training and Qualification	0
SM-AA-300	Procurement Engineering Support Activities	0
SM-AA-300-1001	Procurement Engineering Process and Responsibilities	0
SM-AA-300-1001	Procurement Engineering Process and Responsibilities	1

PROCEDURES

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
SPP 01-027	Steam Generator Tube Flush	1
SPP 01-029	Byron Unit 1, CV Booster Pump Operability Testing	0

REFERENCES

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
--	ASME Boiler and Pressure Vessel Code, Section III, Appendix XI, Appendix XIII and Subsection ND	1977
--	Byron Site Organization Chart	February 28, 2002
--	Maintenance Rule Risk Significant Systems	March 15, 2002
--	Valve 1CC685 Rising Stem MOV Data Sheet	April 15, 1996
--	Valve 1CC685 Rising Stem MOV Data Sheet	September 14, 2000
--	Licensee's 10 CFR 50.59 Self-Assessment Final Draft	October 26, 2001
Action Tracking Number 40605	Licensee's Permanent Plant Modification Self-Assessment	March 1, 2002
BTP CMEB 9.5.1	Fire Protection Program Requirements	Amendment 13
DIT BYR-01-052	Temporary Staging of Lead Shielding Blankets Inside Unit 1 Containment Prior to B1R11	February 22, 2002
IEEE 308	IEEE Standard Criteria for Class IE Power Systems for Nuclear Power Generating Stations	1974
IEEE 317	IEEE Standard for Electric Penetration Assemblies in Containment Structures for Nuclear Power Generating Stations	1972
Memo	Speed of Fuel Handling Equipment	July 15, 1999
RG 1.63, Revision 2	Electric Penetration Assemblies in Containment Structures for Light-Water-Cooled Nuclear Power Plants	July 1978
RG 1.63, Revision 3	Electric Penetration Assemblies in Containment Structures for Nuclear Power Plants	February 1987
Vendor Document	IL-B/S In-Line Separators - Effective Solids from Liquids Filtration Equipment for Applications with Low Flow Rates	January 1996

WORK ORDERS

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
367096-01	Replace Valve Stud Bolts and Nuts for 1SX001A	February 27, 2002

50.59 EVALUATIONS(SEs)

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
6G-00-0022	Drill Hole in Valves 1CV8804A and 2CV8804A to Prevent Pressure Lock	January 26, 2000
6G-00-0026	Modify CC Piping to Install OCC9432 in the Proper Direction	February 1, 2000

50.59 EVALUATIONS(SEs)

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
6G-00-0121	Replacement of PSA Mechanical Snubber 1CC36030S with LISEGA Hydraulic Snubber	1
6G-00-0161	Increase Chemical and Volume Control System Letdown Flow up to a Maximum of 150 Gallons per Minute for Reactor Coolant System Radioactive Material Cleanup	December 28, 2000
6G-01-0033	Install Bypass and Check Valve of Air Solenoid Valve on AF Pump Re-Circulation Valves 1/2AF024	December 13, 2001
6G-02-0001	Installation of Letdown Booster Pumps, Unit 1 and Unit 2	0
6G-02-0003	Revise UFSAR Section 6.3.5.4 for Using Environmental Qualification Containment Floor Water Level Instrumentations for Re-Circulation Switch over from Refueling Water Storage Tank	February 12, 2002
6H-00-0154	Validation of Westinghouse Safety Evaluation SECL-00-144, "Primary Side Loose Parts," Revision 1	October 5, 2000
6H-00-0167	Validation of Safety Evaluation 6G-98-0177, "SX Blowdown Lines and New Blowdown Restriction Orifices," Revision 1	December 9, 2000
Passport Evaluation 8280	Provide Quality Level 1 Procurement Requirements for 1/2SX04P Mechanical Seal, Cat Id 15594-1 and Document Seal Material Changes	November 21, 2000

50.59 SCREENS(SCRs)

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
6D-00-0044	Revise Procedures BOP AF-1, Revision 15; BOP AF-7, Revision 18; BOP AF-8, Revision 15; and 2BOSR 7.5.3-2 Following Plant Modification	January 13, 2000
6D-00-0064	Revise Step 1.4,1.5 and Note Following Step 1.5 to Agree with New ITS Full Load Requirements	January 21, 2000
6D-00-0183	Transfer of In-Service Testing Acceptance Criteria	March 15, 2000
6D-00-0494	Revision of BARs 0-37-A4, Revision 4 and 0-39-A4, Revision 4	August 1, 2000
6D-00-0521	Perform a Differential Pressure Test on 1SI8807B	0 & 1
6D-00-0713	Clarify Precaution with Requirement for Reactor Coolant Pumps in Operation and Rod Control Capable of Rod Withdrawal for Procedure BOP RC-16, Revision 4	August 24, 2000
6D-00-0716	Revise Procedure BOP RC-9, Revision 18 to Match Commitments	August 24, 2000
6E-01-0142	Replacement of Four CC Snubbers with Hydraulic Snubbers (EC 79315)	0
6D-02-0014	Special Test Procedure SPP 01-027, "Steam Generator Tube Flush (Primary Side)	January 1, 1900

50.59 SCREENS(SCRs)

<u>Number</u>	<u>Description</u>	<u>Revision/Date</u>
6D-02-0018	Draining of Essential Service Water System	January 4, 1900
6E-02-0032	Replace Valve 1/2SX001A Stud Bolts and Nuts	0
6E-02-0048	Removal of Two Minute Time Delay for Closing of MS009 Valves on Turbine Trip	0
6E-02-0053	Valve 1SX143B Flange Bolt Removal While In-Service	0
6E-02-0056	Redesign Support 1MS01031R	0

LIST OF INFORMATION REQUESTED

INITIAL DOCUMENT REQUEST

(Sent to Licensee on March 21, 2002)

Inspection Report: 50-454/02-04(DRS); 50-455/02-04(DRS)

Inspection Dates: April 15 -19, 2002

Inspection Procedures: IP 71111.02, "Evaluations of Changes, Tests, or Experiments"
IP 71111.17, "Permanent Plant Modifications"

Lead Inspector: George M. Hausman, (630) 829-9743

I. Information Needed for In-Office Preparation Week

The following information is needed by April 8, 2002, or sooner, to facilitate the selection of specific items that will be reviewed during the on-site inspection week. The team will select specific items from the information requested below and submit a list to your staff by April 10, 2002. We request that the specific items selected from the lists be available and ready for review on the first day of inspection. If you have any questions regarding this information, please call the team leader as soon as possible. All lists requested should cover a two year span. All information should be sent electronically (as a PDF or WordPerfect/Word document) if at all possible.

- a. List of permanent plant modifications to risk significant systems, structures, or components. For the purpose of this inspection, permanent plant modifications include permanent: (i) plant changes, (ii) design changes, (iii) set point changes, (iv) procedure changes, (v) equivalency evaluations, (vi) suitability analyses, (vii) calculations, and (viii) commercial grade dedications.
- b. List of all 10 CFR 50.59 completed evaluations involving: (i) changes to facility (modifications), (ii) procedure revisions, (iii) tests or non-routine operating configurations, (iv) changes to the UFSAR, and (v) calculations.
- c. List of all 10 CFR 50.59 screenings for changes that have been screened out as not requiring a full evaluation involving: (i) changes to facility (modifications), (ii) procedure revisions, (iii) tests or non-routine operating configurations, (iv) changes to the UFSAR, and (v) calculations.
- d. List of all 10 CFR 50.59 validations or applicability reviews done in lieu of a full 10 CFR 50.59 evaluation.
- e. List of corrective action documents (open and closed) that address the plant permanent modification process and 10 CFR 50.59 issues or concerns.

LIST OF INFORMATION REQUESTED

- f. List of special tests or experiments and non-routine operating configurations.
- g. Copies of the modification procedure(s), post-modification testing procedure(s), 10 CFR 50.59 procedure(s) and any other pertinent modification process procedures (e.g., generating calculations, etc.).
- h. Name(s) and phone numbers for the regulatory and technical contacts.
- i. Latest Organizational Chart.

II. **Information to be Available on First Day of Inspection (April 15, 2002)**

- a. Copies of all selected items need to be ready for the inspectors when they arrive on-site. Selected items will be relayed to the NRC coordinator/contact person by April 10th.
- b. The following information needs to be available to the team once they arrive on-site. Copies of these documents do not need to be solely available to the team as long as the inspectors have easy and unrestrained access to them.
 - Updated Final Safety Analysis Report
 - Technical Specifications
 - Latest IPE/PRA Report
 - Vendor Manuals
 - The Latest 10 CFR 50.59 FSAR Update Submittal
- c. Please ensure that all supporting documents for the selected items have been located and are readily retrievable as the team will likely be requesting these documents during the inspections. Examples of supporting documents are:
 - Calculations supporting the selected modifications or 10 CFR 50.59 evaluations
 - Post-modification tests for the selected modifications
 - Copies of other evaluations or validations referred to by a modification or 10 CFR 50.59 evaluation.
 - Drawings supporting the modifications.
 - Procedures affected by the modifications.

LIST OF INFORMATION REQUESTED

- UFSAR change papers generated due to a 10 CFR 50.59 evaluation.
- Condition Reports associated with each selected modification or 10 CFR 50.59 evaluation.

III. Information to Be Provided Throughout the Inspection

- a. Copies of any Condition Report generated as a result of the team's questions or queries during this inspection.
- b. Copies of the list of questions submitted by the team members and the status/resolution of the information requested (provided daily during the inspection to each team member).