

May 2, 2002

MEMORANDUM TO: Robert A. Gramm, Chief, Section 1  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

FROM: David H. Jaffe, Senior Project Manager, Section 1  
Project Directorate IV */RA/*  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

SUBJECT: SUMMARY OF THE APRIL 10, 2002, TELEPHONE CONFERENCE  
CALL REGARDING TXU GENERATION COMPANY LP POST-  
INSPECTION RESULTS OF THE COMANCHE PEAK STEAM  
ELECTRIC STATION (CPSES), UNIT 2, REACTOR PRESSURE  
VESSEL HEAD (TAC NO. MB4538)

On April 10, 2002, a conference call was held with members of the U.S. Nuclear Regulatory Commission (NRC) staff and representatives from TXU Generation Company LP, the licensee for CPSES, Unit 2. The purpose of this call was to discuss the licensee's preliminary results of their inspection of the reactor pressure vessel head.

The licensee performed a 100% effective visual inspection of the reactor pressure vessel head. No evidence of leakage from the vessel head penetration nozzles was observed. The licensee noted that the head outer surface was coated with a thin layer of gray, easily dislodged, powder that did not obstruct the view of the metal. No boric acid material was found and there was no wastage of the vessel head. Some of the peripheral tubes showed a brown discoloration which was attributed to wetting of the surfaces during a previous incident. No other anomalous conditions were noted. The NRC staff expressed some concern that the gray power might be boric acid residue and requested that the licensee determine the composition of this material; the licensee's initial impression was that the material was "dust."

Following the conference call, the licensee e-mailed to the NRC staff, the following analysis of the gray powder:

Calcium as Calcium Carbonate = 55.6%  
Magnesium as Magnesium Carbonate = 42.4%  
Aluminum as Aluminum Oxide = 2.03%  
Boron as Boric Acid < 0.00196%

The above analysis, especially the low boric acid content and easily dislodged form, seems to indicate that whatever the source, the material is benign.

Based on the preliminary findings from the Davis-Besse Nuclear Power Station, the inspections performed by the licensee and the CPSES, Unit 2, low susceptibility to nozzle cracking, the

R. A. Gramm

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NRC staff did not identify any issues that needed additional follow-up prior to plant restart. The NRC staff will document its formal review after receiving the licensee's Bulletin 2002-01 response.

Docket No. 50-446

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