

December 27, 2000

Mr. William O'Connor, Jr.
Vice President
Nuclear Generation
Detroit Edison Company
6400 North Dixie Highway
Newport, MI 48166

SUBJECT: FERMIL 2 NUCLEAR POWER PLANT - NRC INSPECTION REPORT
50-341/00-15(DRS)

Dear Mr. O'Connor:

On December 15, 2000, the NRC completed a baseline inspection at your Fermi 2 Nuclear Power Plant. The results of this inspection were discussed on December 15, 2000, with Mr. P. Fessler and members of your staff. The enclosed report presents the results of that inspection.

The inspection was an examination of activities conducted under your license as they relate to implementation of your maintenance rule program and to compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas the inspection consisted of a selective examination of procedures and representative records, observations of activities, and interviews with personnel. Based on the results of this inspection, no findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

We will gladly discuss any question you have concerning this inspection.

Sincerely,

/RA/

Ronald N. Gardner, Chief
Electrical Engineering Branch
Division of Reactor Safety

Docket No. 50-341
License No. NPF-43

Enclosure: Inspection Report 50-341/00-15(DRS)

See Attached Distribution

W. O'Connor

-2-

cc w/encl: N. Peterson, Director, Nuclear Licensing
P. Marquardt, Corporate Legal Department
Compliance Supervisor
R. Whale, Michigan Public Service Commission
Michigan Department of Environmental Quality
Monroe County, Emergency Management Division
Emergency Management Division
MI Department of State Police

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cc w/encl: N. Peterson, Director, Nuclear Licensing
P. Marquardt, Corporate Legal Department
Compliance Supervisor
R. Whale, Michigan Public Service Commission
Michigan Department of Environmental Quality
Monroe County, Emergency Management Division
Emergency Management Division
MI Department of State Police

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DSH (Project Mgr.)

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket No: 50-341
License No: NPF-43

Report No: 50-341/00-15(DRS)

Licensee: Detroit Edison Company

Facility: Enrico Fermi, Unit 2

Location: 6400 N. Dixie Highway
Newport, MI 48166

Dates: December 13 - 15, 2000

Inspectors: M. Farber, Reactor Engineer

Approved by: Ronald N. Gardner, Chief,
Electrical Engineering Branch
Division of Reactor Safety

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety

- Initiating Events
- Mitigating Systems
- Barrier Integrity
- Emergency Preparedness

Radiation Safety

- Occupational
- Public

Safeguards

- Physical Protection

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: <http://www.nrc.gov/NRR/OVERSIGHT/index.html>.

SUMMARY OF FINDINGS

IR 05000341-00-15(DRS), on 12/13 -15/2000, Detroit Edison Company, Enrico Fermi, Unit 2. Maintenance Rule Implementation.

The report covers a three day period of announced inspection by a regional reactor engineer. This inspection focused on the Reactor Safety strategic area, and included the following.

REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems, Barrier Integrity

No findings of significance were identified.

Report Details

1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems, Barrier Integrity

1R12 Maintenance Rule Implementation (Inspection Procedure 71111.12)

a. Inspection Scope

The objective of the inspection was to:

- Verify that the periodic evaluation was completed within the time restraints defined in the maintenance rule (once per refueling cycle, not to exceed two years), ensuring that the licensee reviewed its goals, monitoring, preventive maintenance activities, industry operating experience, and made appropriate adjustments as a result of that review;
- Verify that the licensee balanced reliability and unavailability during the previous refueling cycle, including a review of safety significant structures, systems, and components, (SSC);
- Verify that (a)(1) goals were met, corrective action was appropriate to correct the defective condition including the use of industry operating experience, and (a)(1) activities and related goals were adjusted as needed; and
- Verify that the licensee has established (a)(2) performance criteria, examined any SSCs that failed to meet their performance criteria, or reviewed any SSCs that have suffered repeated maintenance preventable functional failures including a verification that failed SSCs were considered for (a)(1).

The inspector examined the current periodic evaluation, "Maintenance Rule Periodic Assessment Report - September 1997 through April 1999," dated September 27, 1999. To evaluate the effectiveness of (a)(1) and (a)(2) activities the inspector examined more than 90 Corrective Action Resolution Documents (CARD) (contained in the list of documents at the end of this report) associated with the following systems:

- a. Nuclear Boiler
- b. Reactor Protection
- c. Control Rod Drive Hydraulic Control
- d. Residual Heat Removal
- e. Residual Heat Removal Service Water
- f. DC Battery
- g. Reactor Building Closed Cooling Water
- h. Stator Cooling Water
- i. Hydrogen Seal Oil

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA2 Problem Identification and Resolution

a. Inspection Scope

The objective of the inspection was to review the licensee's problem identification and resolution of maintenance rule-related issues. The inspector reviewed the following CARDS identifying problems with Maintenance Rule implementation:

- 00-12134, OOS Hours Not Tracked for Several Risk Significant Containment Isolation Valves, dated January 7, 2000;
- 00-13501, Maintenance Rule Monthly Reporting Errors, dated March 9, 2000;
- 00-19223, PMT Failure Maintenance Rule Functional Failure Evaluation Required per MWC(02), dated October 12, 2000;
- 00-24758, Maintenance Rule Availability Inconsistently Applied for Surveillances, dated December 3, 2000;
- 00-20740, Self Assessment Action Item, dated October 12, 2000;
- 00-25408, Bogus Data Added to EDG OOS Hours in CPEP by an ISO Administrator, dated November 29, 2000; and
- 00-25410, The RBCCW Temperature Control Function is Not Adequately Described in the Maintenance Rule Program, dated November 29, 2000.

b. Findings

No findings of significance were identified.

4OA6 Management Meetings

Exit Meeting Summary

The inspector presented the inspection results to Mr. P. Fessler and other members of licensee management and staff on December 15, 2000. The licensee acknowledged the information presented and did not identify any as proprietary.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

S. Booker, Work Control
D. Cobb, Maintenance
J. Davis, Outage Management
G. DePalma, System Engineering
P. Fessler, Plant Manager
S. Hassoun, Nuclear Licensing
M. Kramer, Operations
A. Lim, Plant Support Engineering
N. Peterson, Nuclear Licensing
W. Miller, Engineering Projects
S. Stasek, Nuclear Assessment
J. Tibai, Maintenance Rule Principal Engineer
D. Williams, Radiation Protection
R. Zemenski, Nuclear Quality Assurance

NRC

S. Campbell, Senior Resident Inspector
G. Larizza, Resident Inspector

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None

Closed

None

Discussed

None

LIST OF ACRONYMS USED

CARD	Condition Assessment Resolution Document
CFR	Code of Federal Regulations
DRS	Division of Reactor Safety
NRC	Nuclear Regulatory Commission
OA	Other Activities
SSC	Structures, Systems, and Components

INSPECTION PROCEDURES USED

71111.12 - Maintenance Rule Implementation

LIST OF DOCUMENTS REVIEWED

Procedures

- Engineering Support Conduct Manual, MES40, Maintenance Rule Program Description, Revision 7, dated June 2, 2000
- Maintenance Rule Program Manual (MRPM) MR01, Maintenance Rule Program Description, Revision 5, dated July 24, 1998
- MRPM, MR02, Expert Panel, Revision 9, dated November 15, 2000
- MRPM, MR03, Scoping, Revision 2, dated January 12, 1998
- MRPM, MR04, Determination of Risk Significance, Revision 1, dated January 12, 1998
- MRPM, MR05, Determination of SSC Functions, Revision 3, dated April 3, 1998
- MRPM, MR06, Establishing Performance Criteria, Revision 5, dated February 2, 1998
- MRPM, MR07, Historical Review of SSCs, Revision 0, dated June 24, 1996
- MRPM, MR08, SSC Classification, Revision 5, dated July 24, 1998
- MRPM, MR09, Establishment of Get Well Plans, Revision 14, dated October 15, 1999
- MRPM, MR10, Monitoring, Revision 7, dated March 28, 2000
- MRPM, MR11, Periodic Assessment, Revision 2, dated September 20, 1999
- MRPM, MR12, Equipment Out of Service Risk Management, Revision 9, dated November 28, 2000
- MRPM, MR13, Critical Performance Evaluation Program (CPEP), Revision 3, dated January 24, 2000
- MRPM, MR14, Structures Monitoring, Revision 2, January 12, 1998
- MRPM, MR15, Self Assessment of Practices and Processes Effectiveness, Revision 0, dated June 2, 2000
- MRPM, MR APP A, Appendix A, Terms and Definitions, Revision 3, dated March 6, 1998
- MRPM, MR APP B, Appendix B, References, Revision 2, dated January 12, 1998
- MRPM, MR APP C, Appendix C, Position Papers, Revision 7, dated January 24, 2000
- MRPM, MR APP D, Appendix D, Guidelines for Determining Functional Failures (FFs) and Maintenance Preventable Functional Failures (MPFFs), Revision 9, dated March 28, 2000
- MRPM, MR APP E, Appendix E, Attachment Cross Reference List, Revision 6, dated January 24, 2000
- MRPM, MR APP F, Appendix F, Maintenance Rule Scoping Summary Report, Revision 2, dated December 30, 1998
- MRPM, MR APP G, Appendix G, Maintenance Rule SSC Specific Functions, Revision 7, dated June 5, 2000
- MRPM, MR APP H, Appendix H, Maintenance Rule Performance Criteria, Revision 7, dated September 26, 2000
- MRPM, MR APP I, Appendix I, On-line Risk Maintenance Risk Matrix, Revision 5, dated November 21, 2000
- MRPM, MR APP J, Appendix J, Risk Significant Systems, Revision 0, dated July 1, 1998

- MRPM, MR APP K, Appendix K, PSA Model Maintenance & Configuration Control, Revision 0, dated July 1, 1998

Miscellaneous

- Maintenance Rule Periodic Assessment Report - September 1997 through April 1999, dated September 27, 1999
- (a)(1)/(c)(1) System Performance Report, dated November 10, 2000

Condition Assessment Resolution Document

- 98-22550, RHRSW Pumps A & C Differential Pressure in Alert Range Low, dated November 10, 1998
- 98-22900, C1103D023 Frequent Water Alarms, dated November 15, 1998
- 98-22420, Div I ESF Battery Cell 28 Low Voltage Outside Tech Spec Allowable Value, dated November 5, 1998
- 98-23059, CRD Suction Pressure Reg. Not Maintaining Suction Pressure during a Downpower causing C11F212 to Open, dated November 22, 1998
- 98-23132, Reactor Building Containment Isolation Check Valves, dated November 18, 1998
- 98-23291, Control Rod Withdrawal Not Working, dated December 12, 1998
- 98-23726, RHR Pump 'B' Discharge Check Valve Leaks by its Seat, dated December 23, 1998
- 99-00434, N3032D002 N. H₂ Seal Oil Turb End Str, dated April 11, 1999
- 99-02606, 2C-2 Battery Voltage Reading Low, dated December 17, 1999
- 99-11116, Recurring Packing Leaks (6 Times in 8 Years), dated March 11, 1999
- 99-11353, MCC 2PC-1 Pos. 5B - Wrong Fuses Found Installed, dated March 9, 1999
- 99-11705, Discharge Pressure Lowered Unexpectedly When Shifting to N H₂ Seal Oil Pump, dated February 7, 1999
- 99-11722, BOP 130VDC Battery Charger 2C-1 Tripped on High Voltage, dated February 17, 2000
- 99-11745, H₂ Seals Degrading, dated March 29, 1999
- 99-11928, Control Rod Withdrawal Block Relay is Cycling Causing Intermittent Control Rod Blocks, dated April 24, 1999
- 99-11976, Control Rod 06-35 Single Rod Scram, dated April 10, 1999
- 99-12664, Lower than Expected Voltage on Div 2 24/48 VDC Battery Cells 1 - 12, dated March 18, 1999
- 99-13197, Crossed Cables on Scram Solenoid Pilot Valves, dated April 10, 1999
- 99-13489, Possible Mixed Oil in RHR Pump 'C' Upper Motor Bearing, dated May 5, 1999
- 99-13580, Rod 54-39 Was Slow on Insert during CRD Operability, dated May 15, 1999
- 99-13630, Loss of SRV-G Control on Dedicated Shutdown Panel H21P623 due to High Impedance Faults, dated May 21, 1999
- 99-13769, Loss of '48' Position Causing Control Rod Drift Alarm (Rod 46-07), dated May 23, 1999
- 99-14207, RHRSW Crosstie Valve Failed in Mid-position, dated June 1, 1999
- 99-14662, Incorrect Fuses Installed, dated August 2, 1999

- 99-15631, 2C-1 Battery Charger Tripped on High Voltage Shutdown, 2C1-2 Battery Charger (spare) Maintained Voltage Too Low When Placed in Service, dated, July 3, 1999
- 99-15633, CRD 58-19 Difficult to Withdraw during 24.106.001, CRD Operability Surveillance, dated July 3, 1999
- 99-15917, RHRSW 'C' Hi Thrust Bearing Temp, dated August 9, 1999
- 99-16097, Frequent CRD Hydraulic Temperature High Alarm, dated September 9, 1999
- 99-17244, Debris Found in 24/48VDC Batteries, dated September 16, 1999
- 99-17385, Battery 2C-2 Voltage Reading Low, dated October 4, 1999
- 99-18177, Cooling Water Diff Press Ind. Reading High, dated October 24, 1999
- 99-18464, Improper Stroke Length of Valve E11F400A, dated December 2, 1999
- 99-18999, Div 2 Battery Electrolyte Level Low, dated December 20, 1999
- 99-19030, Div 1 Btry 21A Low on Specific Gravity, dated November 18, 1999
- 99-19158, Scoring Found in SRV (Target Rock Valve) Pilot Base #389 in Ring Seating Area, dated December 12, 1999
- 99-19233, Div 1 Battery Corroded Cells, dated December 9, 1999
- 99-19329, E1102C002B Axial Vibration Level in ALERT range, dated December 21, 1999
- 00-00566, Div I Battery Charger Amps High OOS, dated July 13, 2000
- 00-01036, West Stator Cooling Water Pump Seal Leaking, dated May 14, 2000
- 00-01039, West Stator Cooling Water Pump Leaking Discharge Flange, dated May 16, 2000
- 00-01365, RBCCW TCW Linkage Loose, dated October 29, 2000
- 00-01477, N0332D002 H₂ Seal Oil Turbine End Strainer, dated August 16, 2000
- 00-04087, Leak from W. SWC Pump Discharge Valve, dated November 27, 2000
- 00-10538, Incorrect Labeling of Control Wires to Pos. 11A and 11B Results in Wrong Circuits Red-Tagged, dated January 13, 2000
- 00-10592, C11-N016B - 3 Consecutive Failures on Calibration, dated January 22, 2000
- 00-10595, Testing Shows Possible Motor Degradation, dated February 9, 2000
- 00-10659, C11-N016A - 3 Consecutive Out of Tolerance As-found Values, dated February 7, 2000
- 00-10863, Blown Div 2 RHR Logic Fuse due to Shorting of Limit Switch Contacts, dated April 27, 2000
- 00-11109, HCU 22-15 Neutral Wire for the C11F112 Solenoid Sparked to Ground, dated January 8, 2000
- 00-11116, Recurring Packing Leaks (Six Times in Eight Years), dated March 11, 1999
- 00-11292, E1150F006D Improper Fuses Installed, dated February 19, 2000
- 00-11306, NS4-Main Steam Line Tunnel Temperature Trip System B Channel D B21-N612D MTU Gross Fail High, dated February 24, 2000
- 00-11323, High Temperature on CRD 30-23, dated March 21, 2000
- 00-11393, Unexpected Delay When ATWS/ARI/RPT Logic Channel 'C' Reset, dated March 13, 2000
- 00-11432, Mechanical Binding of Directional Valve for CRD 18-39, dated April 22, 2000
- 00-11445, RHR Pump B Discharge Check Valve or Min Flow Check Valve Leaks By, dated May 5, 2000
- 00-12134, OOS Hours Not Tracked for Several Risk Significant Containment Isolation Valves, dated January 7, 2000
- 00-12882, Closing Min.Flow Valve Isolations during Failure of RHR MFV to Close When Going in SDC, dated September 16, 2000

- 00-13018, Loop Failure 3 Consecutive Times, dated March 1, 2000
- 00-13024, 3 Consecutive Out of Tolerance Values, dated March 1, 2000
- 00-13501, Maintenance Rule Monthly Reporting Errors, dated March 9, 2000
- 00-14356, Limit Switch Mounting Bolt Broken Off and Switches Loose on the Bracket, dated April 23, 2000
- 00-14463, Valve Was Inadvertently Stalled into the Backseat, dated April 24, 2000
- 00-14613, Install Spare 24VDC Battery Charger, dated April 5, 2000
- 00-14711, B2100F011A Will Not Fully Open, dated April 27, 2000
- 00-15105, RPS Shorting Link Terminal @ H11-P611 Terminal AA-113 (north) is Damaged, dated April 24, 2000
- 00-15108, CRDM 34-35 Difficult to Move out of Position 00, dated April 23, 2000
- 00-15146, Loss of Shutdown Cooling, dated April 22, 2000
- 00-15213, Foreign Material Found in Pump, dated April 12, 2000
- 00-15305, E1100F050A Seat Failure, dated April 11, 2000
- 00-15396, Feedwater Check Valve Found with Missing Cotter Pin and Disc Nut Loose and Backed off Approx. ½ " to 9/16", dated April 15, 2000
- 00-16045, E1100M045B Unsat Electrical Connection, dated April 25, 2000
- 00-16220, West Stator Cooling Water Pump Seal Leaks, dated May 13, 2000
- 00-16250, CRD Flow Controller Operates Erratically, dated May 5, 2000
- 00-16526, SRV Solenoid Deficiencies & Corrective Actions, dated May 3, 2000
- 00-16623, Increase in Vibration Levels on Stator Cooling Water Pump, dated May 18, 2000
- 00-16812, Less than Expected Flow, dated May 15, 2000
- 00-17225, Relays Dropped Out with 'D' MSIVs Open, dated May 18, 2000
- 00-17279, Control Rod 26-55 is Difficult to Withdraw, dated May 17, 2000
- 00-17349, SRV 'B' Reopened After Close PB was Depressed, dated May 5, 2000
- 00-17459, Degrading Bearing on East Stator Water Cooling Pump, dated June 27, 2000
- 00-17609, 200 DPM Leak from Seal, dated June 11, 2000
- 00-17441, 'D' RHRSW Pump Differential Pressure in Alert Range Low, dated June 20, 2000
- 00-18588, Isolation Valves Leaking, dated June 21, 2000
- 00-18907, Investigate Reason for H₂ Seal Oil Temperature Control Valve Being Open, dated July 27, 2000
- 00-18942, Apparent Failure of Relay A71B-K2A during Surveillance, dated August 3, 2000
- 00-18988, Hydrogen Seal Oil Cooler Possibly Fouled, dated August 24, 2000
- 00-19163, Main Generator Stator Winding Cooling Water Flow Restriction, Characteristic of Copper Oxide Fouling, dated September 14, 2000
- 00-19223, PMT Failure Maintenance Rule Functional Failure Evaluation Required per MWC(02), dated October 12, 2000
- 00-19481, Performance Criteria Exceeded for RHR System, dated September 21, 2000
- 00-19495, Riley Temperature Switch Model Number Discrepancies, dated October 3, 2000
- 00-19680, Significant Degradation of Motor Actuator Has Caused High Out of Spec Stroke Time, dated September 19, 2000
- 00-19686, E1150F024B Flow Inconsistencies in the Fully Open Position, dated September 23, 2000
- 00-24758, Maintenance Rule Availability Inconsistently Applied for Surveillances, dated December 3, 2000

- 00-18997, Failure of P42F400, dated August 22, 2000
- 00-20740, Self Assessment Action Item, dated October 12, 2000
- 00-24717, Repeat Failure of RBCCW TCV Causing ESF Actuation, dated November 24, 2000
- 00-25408, Bogus Data Added to EDG OOS Hours in CPEP by an ISO Administrator, dated November 29, 2000
- 00-25410, The RBCCW Temperature Control Function is Not Adequately Described in the Maintenance Rule Program, dated November 29, 2000
- 00-25500, Replace In-service Battery with a Capacity Tested Spare, dated November 15, 2000