UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
611 RYAN PLAZA DRIVE, SUITE 400
ARLINGTON, TEXAS 76011-4005

July 27, 2005

Randall K. Edington, Vice
President-Nuclear and CNO
Nebraska Public Power District
P.O. Box 98
Brownville, NE 68321

SUBJECT: COOPER NUCLEAR STATION - NRC RADIATION SAFETY TEAM INSPECTION REPORT 05000298/2005011

Dear Mr. Edington:

On June 9, 2005, the U.S. Nuclear Regulatory Commission (NRC) completed an onsite inspection at your Cooper Plant. The enclosed report documents the inspection findings, which were discussed at the conclusion of the inspection with Mr. S. Minahan, General Manager of Plant Operations, and other members of your staff. On July 21, 2005, a telephonic exit was conducted with Mr. T. Chard, Radiation Protection Manager concerning a radioactive material shipment and disposal finding.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The team reviewed selected procedures and records, observed activities, and interviewed personnel. Specifically, the team evaluated the inspection areas within the Radiation Protection Strategic Performance Area that are scheduled for review every two years. These areas are:

- Radiation Monitoring Instrumentation
- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems
- Radioactive Material Processing and Transportation
- Radiological Environmental Monitoring Program and Radioactive Material Control Program

This inspection report documents one self-revealing, non-cited violation of very low safety significance (Green). However, because the finding was of very low safety significance and it was entered into your corrective action program, the NRC is treating this finding as a non-cited violation consistent with Section VI.A of the NRC Enforcement Policy. If you contest the non-cited violation in this report, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington DC 20555-0001; with copies to the Regional Administrator, U.S. Nuclear Regulatory Commission Region IV, 611 Ryan Plaza Drive,

Suite 400, Arlington, Texas 76011-4005; the Director, Office of Enforcement, U.S. Nuclear Regulatory Commission, Washington DC 20555-0001; and the NRC Resident Inspector at the Cooper Nuclear Generating Station.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Sincerely,

//RA//

Michael P. Shannon, Chief
Plant Support Branch
Division of Reactor Safety

Docket: 50-298
License: DPR-46

Enclosure:
NRC Inspection Report 05000298/2005011
w/attachment: Supplemental Information

cc w/enclosure:
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Electronic distribution by RIV:
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Team Leader, DRP/TSS **(RLN1)**
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SISP Review Completed: Yes ADAMS: Yes ☐ No Initials: _LC2____
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RIV:PSB\SHP	PSB\HP	PSB\HP	PSB\HP	C:PSB
LCCarson II	BDBaca	DRCarter	BKTharakan	MPShannon
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DRP\B	C:PSB			
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/RA/	/RA/			
7/ 27 /05	7/ 27 /05			

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U.S. NUCLEAR REGULATORY COMMISSION
REGION IV

Docket.:	50-298
License:	DPR-46
Report:	05000298/2005011
Licensee:	Nebraska Public Power District
Facility:	Cooper Nuclear Station
Location:	P.O. Box 98 Brownville, Nebraska
Dates:	June 6 through July 21, 2005
Inspectors:	Louis C. Carson II, Senior Health Physicist, Plant Support Branch Bernadette D. Baca, Health Physicist, Plant Support Branch Daniel R. Carter, Health Physicist, Plant Support Branch Binesh K. Tharakan, Health Physicist, Plant Support Branch
Approved By:	Michael P. Shannon, Chief, Plant Support Branch Division of Reactor Safety

Enclosure

SUMMARY OF FINDINGS

Cooper Nuclear Station
NRC Inspection Report 05000298/2005011

IR 05000298/2005011; Nebraska Public Power District; 06/06/05-07/15/2005; Cooper Nuclear Station; Radiation Safety Team Inspection

The report covered a six week period of inspection by a team of four region-based health physics inspectors. One finding of very low safety significance (Green) was identified. The significance of most findings is indicated by their color (Green, White, Yellow, or Red) using Inspection Manual Chapter (IMC) 609 and was determined by the Significance Determination Process (SDP) in IMC 0609. Findings for which the SDP does not apply may be "Green" or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

A. NRC-Identified and Self-Revealing Findings

Cornerstone: Public Radiation Safety (PS)

- Green. The team reviewed a self-revealing non-cited violation of 10 CFR 30.41(b)(5) because the licensee failed to correctly ship byproduct material. Specifically, on July 8, 2005, the licensee was notified by Chem-Nuclear, LLC, of the Barnwell Waste Management Facility (Barnwell) that the licensee's radioactive waste shipment (05-10) contained loose radioactive material in the Type B shipping cask, which is prohibited by the Barnwell license.

  The failure to correctly ship radioactive material is a performance deficiency. The finding is greater than minor because it was associated with the Public Radiation Safety cornerstone attribute of Transportation Packaging, and it affected the cornerstone objective to ensure adequate protection of public health and safety from exposure to radioactive materials. This finding was processed through the Public Radiation Safety Significance Determination Process because the finding involved an occurrence in the licensee's radioactive material transportation program that is contrary to NRC regulations. The finding was determined to be of very low safety significance (Green) because: (1) it is a finding in the transportation program, (2) there were no radiation dose limits exceeded, (3) there was no breach of package during transportation, (4) it was not a Certificate of Compliance finding, (5) it was a low-level waste burial Ground Nonconformance; however, (6) access was not denied and (7) the waste was not under-classified. The finding was entered into the licensee's corrective action program as CR-CNS-2005-04886 (Section 2PS2).

Report Details

2. RADIATION SAFETY

Cornerstone: Occupational Radiation Safety [OS]

2OS3 Radiation Monitoring Instrumentation and Protective Equipment (71121.03)

a. Inspection Scope

This area was inspected to determine the accuracy and operability of radiation monitoring instruments that are used for the protection of occupational workers and the adequacy of the program to provide self-contained breathing apparatus (SCBA) to workers. The team used the requirements in 10 CFR Part 20 and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- Calibration of area radiation monitors associated with transient high and very high radiation areas and post-accident monitors used for remote emergency assessment
- Calibration of portable radiation detection instrumentation, electronic alarming dosimetry, and continuous air monitors used for job coverage
- Calibration of whole body counting equipment and radiation detection instruments utilized for personnel and material release from the radiologically controlled area
- Self-assessments and audits
- Corrective action program reports since the last inspection
- Licensee action in cases of repetitive deficiencies or significant individual deficiencies
- Calibration expiration and source response check currency on radiation detection instruments staged for use
- The licensee's capability for refilling and transporting SCBA air bottles to and from the control room and operations support center during emergency conditions, status of SCBA staged and ready for use in the plant and associated surveillance records, and personnel qualification and training
- Qualification documentation for onsite personnel designated to perform maintenance on the vendor-designated vital components, and the vital component maintenance records for SCBA units

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

- Licensee Event Reports

The inspector completed 9 of the required 9 samples.

b. Findings

No findings of significance were identified.

Cornerstone: Public Radiation Safety [PS]

2PS1 Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems (71122.01)

a. Inspection Scope

This area was inspected to ensure that the gaseous and liquid effluent processing systems are maintained so that radiological releases are properly mitigated, monitored, and evaluated with respect to public exposure. The team used the requirements in 10 CFR Part 20, 10 CFR Part 50 Appendices A and I, the Offsite Dose Assessment Manual, and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- The most current radiological effluent release reports, changes to radiation monitor setpoint calculation methodology, anomalous sampling results, effluent radiological occurrence performance indicator incidents, self-assessments, and audits
- Gaseous and liquid release system component configurations
- Routine processing, sample collection, sample analysis, and release of radioactive gaseous effluent and radioactive liquid effluent release permits with dose projections to members of the public
- Abnormal releases
- Changes made by the licensee to the Offsite Dose Assessment Manual, the liquid or gaseous radioactive waste system design, procedures, or operation since the last inspection
- Monthly, quarterly, and annual dose calculations
- Surveillance test results involving air cleaning systems and stack or vent flow rates

- Instrument calibrations of discharge effluent radiation monitors and flow measurement devices, effluent monitoring system modifications, effluent radiation monitor alarm setpoint values, and counting room instrumentation calibration and quality control program

- Interlaboratory comparison program results

- Special reports, audits, self-assessments and corrective action reports performed since the last inspection

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

- Licensee Event Reports

The inspector completed 10 of the required 10 samples.

b. Findings

No findings of significance were identified.

2PS2 Radioactive Material Processing and Transportation (71122.02)

a. Inspection Scope

This area was inspected to verify that the licensee's radioactive material processing and transportation program complies with the requirements of 10 CFR Parts 20, 61, and 71 and Department of Transportation regulations contained in 49 CFR Parts 171-180. The team interviewed licensee personnel and reviewed:

- The radioactive waste system description, recent radiological effluent release reports, and the scope of the licensee's audit program

- Liquid and solid radioactive waste processing systems configurations, the status and control of any radioactive waste process equipment that is not operational or is abandoned in place, changes made to the radioactive waste processing systems since the last inspection, and current processes for transferring radioactive waste resin and sludge discharges

- Radio-chemical sample analysis results for radioactive waste streams and use of scaling factors and calculations to account for difficult-to-measure radionuclides

- Shipment packaging, surveying, labeling, marking, placarding, vehicle checking, driver instructing, and disposal manifesting

- Shipping records for non-excepted package shipments

- Licensee event reports, special reports, audits, state agency reports, self-assessments and corrective action reports performed since the last inspection

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

- Licensee event reports and special reports

The inspector completed 6 of the required 6 samples.

b. Findings

Introduction: The team reviewed a green self-revealing non-cited violation of 10 CFR 30.41(b)(5) because the licensee failed to correctly ship radioactive material. Specifically, on July 8, 2005, the licensee was notified by Chem-Nuclear, LLC, of the Barnwell Waste Management Facility (Barnwell) that the licensee's radioactive waste shipment contained loose radioactive material in the Type B shipping cask, which is prohibited by the Barnwell license.

Description: On June 24, 2005, the licensee sent a shipment, number 05-10, of low-level radioactive waste in a Type B Cask, model number TN-RAM to Barnwell for disposal. On July 8, 2005, during the offloading process at Barnwell, a quarter-inch by quarter-inch piece of metal debris was discovered between the outer wall of the inner TN-RAM liner and the inner wall of the TN-RAM cask. The radiation level measured on contact with the metal debris was 70 Roentgen per hour. The metal debris was safely removed from the shipping cask and stored for final disposal.

Analysis: The failure to correctly ship byproduct material is a performance deficiency. The finding is greater than minor because it was associated with the Public Radiation Safety cornerstone attribute of Transportation Packaging, and it affected the cornerstone objective to ensure adequate protection of public health and safety from exposure to radioactive materials. This finding was processed through the Public Radiation Safety Significance Determination Process because the finding involved an occurrence in the licensee's radioactive material transportation program that is contrary to NRC regulations. The finding was determined to be of very low safety significance (Green) because: (1) it is a finding in the transportation program, (2) there were no radiation dose limits exceeded, (3) there was no breach of package during transportation, (4) it was not a Certificate of Compliance finding, (5) it was a low-level waste burial Ground Nonconformance; however, (6) access was not denied and (7) the waste was not under-classified.

Enforcement: 10 CFR 30.41(b)(5), states, in part, that any licensee may transfer byproduct material to a person authorized to receive such byproduct material under terms of a specific license or general license issued by the Commission or an Agreement State. Barnwell License Number 97, Condition 61, states, in part, that loose radioactive material and solidification residuals within shipping casks are prohibited. On

July 8, 2005, the licensee was notified that their shipment, number 05-10, contained loose radioactive material which did not meet the requirements of Barnwell license. Because the finding is of very low safety significance and has been entered into the licensee's corrective action program as CR-CNS-2005-04886, this violation is being treated as a non-cited violation in accordance with Section VI.A of the Enforcement Policy: NCV 05000298/2005011-01, Failure to Correctly Ship Radioactive Material.

2PS3 Radiological Environmental Monitoring Program (REMP) And Radioactive Material Control Program (71122.03)

a. Inspection Scope

This area was inspected to ensure that the REMP verifies the impact of radioactive effluent releases to the environment and sufficiently validates the integrity of the radioactive gaseous and liquid effluent release program; and that the licensee's surveys and controls are adequate to prevent the inadvertent release of licensed materials into the public domain. The team used the requirements in 10 CFR Part 20, Appendix I of 10 CFR Part 50, the Offsite Dose Assessment Manual, and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- Annual environmental monitoring reports
- A sampling of air sampling stations and thermoluminescence dosimeter monitoring stations
- Collection and preparation of environmental samples
- Operability, calibration, and maintenance of meteorological instruments
- Each event documented in the Annual Radiological Environmental Monitoring Report which involved a missed sample, inoperable sampler, lost thermoluminescence dosimeter, or anomalous measurement
- Significant changes to the Off-Site Dose Assessment Manual as the result of changes to the land census or sampler station modifications since the last inspection
- Calibration and maintenance records for air samplers, composite water samplers, and environmental sample radiation measurement instrumentation, quality control program, inter-laboratory comparison program results, and vendor audits
- Locations where the licensee monitors potentially contaminated material leaving the radiological controlled area and the methods used for control, survey, and release from these areas

- Types of radiation monitoring instrumentation used to monitor items released, survey and release criteria of potentially contaminated material, radiation detection sensitivities, procedural guidance, and material release records

- Audits, self-assessments and corrective action reports performed since the last inspection

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

- Licensee event reports

The inspector completed 10 of the required 10 samples.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA2 Problem Identification and Resolution

l. Inspection Scope

The team evaluated the effectiveness of the licensee's problem identification and resolution process with respect to the following inspection areas:

- Radiation Monitoring Instrumentation (Section 2OS3)

- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems (Section 2PS1)

- Radioactive Material Processing and Transportation (Section 2PS2)

- Radiological Environmental Monitoring Program and Radioactive Material Control Program (Section 2PS3)

b. Findings and Observations

No findings of significance were identified.

4OA5 Other

TI 2515/161 - Transportation of Reactor Control Rod Drives in Type A Packages

a. Inspection Scope

This area was inspected to verify that the licensee's radioactive material transportation program complies with specific requirements of 10 CFR Parts 20, 71, and Department of Transportation regulations contained in 49 CFR Part 173. The inspector interviewed licensee personnel and determined the licensee had undergone refueling/defueling activities between January 1, 2002, and present, but it had not shipped irradiated control rod drives in Department of Transportation Specification 7A Type A packages.

b. Findings and Observations

No findings of significance were identified.

4OA6 Meetings

Exit Meeting Summary

The team presented the results of the inspection to Mr. S. Minahan, General Manager of Plant Operation, and other members of licensee management during an exit meeting conducted on June 9, 2005. The licensee acknowledged the findings presented.

On July 21, 2005, the team conducted an exit interview by telephone, and presented additional radioactive material control findings to Mr. Timothy Chard, Radiation Protection Manager and Mr. Paul Fleming, Manager, Licensing. The licensee acknowledged the findings presented

The team asked the licensee whether or not any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

SUPPLEMENTAL INFORMATION

PARTIAL LIST OF PERSONS CONTACTED

Licensee
G. Armknecht, Radiological Specialist, Radiation Protection
J. Bedner, Manager, Emergency Preparedness
J. Cass, Engineer, Reactor and Containment Engineering
T. Chard, Radiation Protection Manager, Radiation Protection
B. Duncan, Radiological Protection Technician, Radiation Protection
P. Fleming, Manager, Risk and Regulatory Affairs
S. Freiling, Radiological Specialist, Radiation Protection
J. Gren, Engineer, System Engineering
G. Kahnk, Engineer, Reactor and Containment Engineering
J. Kuttler, Staff Health Physicist, Radiation Protection
L. Maine, Technician, Chemistry
R. McDonald, Staff Health Physicist, Radiation Protection
S. Minahan, General Manager, Plant Operation
K. Ohrablo, Technician, Chemistry
J. Roberts, Director, Nuclear Safety Assurance
E. Rotkvic, Staff Health Physicist, Radiological Protection
C. Stipp, Environmental Coordinator, Corporate Environmental
K. Tanner, Supervisor, Radiation Protection
J. Teten, Supervisor, Chemistry

NRC
S. Schwind, Senior Resident Inspector
S. Cochrum, Resident Inspector

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened and Closed During this Inspection

05000298/2005011-01 NCV Failure to Correctly Ship Radioactive Material (Section 2PS2)

LIST OF DOCUMENTS REVIEWED

Section 2OS3: Radiation Monitoring Instrumentation and Protective Equipment (71121.03)

Procedures

EPIP5.7.12 Emergency Radiation Exposure Control, Revision 14
9.INST.10 Eberline Model PM-7 Portal Monitor, Revision 2
9.INST.20 Calibration of the Canberra Whole Body Counter, Revision 1
9.INST.41 SAIC Model PD-1 Electronic Dosimeter System, Revision 5
9.INST.47 Eberline Personnel Contamination Monitor Model PCM-2, Revision 2
9.INST.52 Extender Model 2000W GM Survey Instrument, Revision 1
9.INST.53 Ion Chamber Survey Instrument Eberline Models RO-2, RO-2A, and RO-20, Revision 1
9.INST.57 Friskers, Revision 0
9.INST.61 Merlin Gerin AMP-100/AMP-200, Revision 1
9.INST.65 Constant Air Monitor Eberline Model AMS-4, Revision 1

Condition Reports

CR-CNS-2005-04218, CR-CNS-2005-04222, CR-CNS-2005-04265

Audit and Assessments

Cooper Nuclear Station Quality Assurance Surveillance Report QAD20050031, May 17, 2005
Quality Assurance Audit 03-08, Off-Site Dose Assessment Manual, September 8-25, 2003
Cooper Nuclear Station 2004 Environmental Assessment Report, June 7-15, 2004
Cooper Nuclear Station 2003 Environmental Assessment Report, September 8-17, 2003
NUPIC Audit/Survey of Teledyne Brown Engineering Environmental Services, Audit No. 18668, March 25-27, 2003

Calibration Records

Identification Number 377, Eberline PCM-2, April 18, 2005
Identification Number 378, Eberline PCM-2, April 18, 2005
Identification Number 401, Eberline PCM-2, April 18, 2005
Identification Number 553, Eberline PCM-2, April 18, 2005
Identification Number 7799-021 MGP Instruments AMP-200, December 28, 2004
Identification Number 7799-021 MGP Instruments AMP-200, June 7, 2005
Identification Number 15704, Extender Low/High Range Detector March 25, 2005
Identification Number 375-376, SAM-11 Tool Contamination Monitor, March 28, 2005
Identification Number 266, RO-2 Dose Rate Meter, August 11, 2004
Identification Number 266, RO-2 Dose Rate Meter, February 28, 2005
Identification Number 1676, RO-20 Dose Rate Meter, February 28, 2005
Identification Number 498, RML-2 Frisker, March 24, 2005
Identification Number 609, RML-2 Frisker, March 24, 2005

Surveillance Procedures

6.PRM. 302 Off-Gas Radiation Monitor Linearity Test and Efficiency Determination, Revision 6
6 PRM. 312 Reactor Building Kaman Monitor Channel Functional Test, Revision 5
6.PRM. 331 SW Radiation Monitor - A Calibration Check and Instrument Channel Test Revision 4
6.PRM. 332 SW Radiation Monitor - B Calibration Check and Instrument Channel Test Revision 5

Section 2PS1: Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems (71122.01)

Audits, Cross-Checks, and Self-Assessments

2003 and 2004 ANALYTICS Cross-Check Program Results
2003 and 2004 Chemistry Department Cross-Check Program Results
NUPIC Audit 18899, Eberline Services dated March 14, 2005
QAD20030050, QA Audit 03-08, Offsite Dose Assessment Manual
QAD20050031, Surveillance Number S407-0503
Snap Shot Assessments: SS04-007, SS04-220, CR-CNS-2002-5695 CA-0033, CR-CNS-2002-5695 CA-0035, and LO-CNSLO-2005-0008 CA-0001

Effluent Monitor Calibration

Covered under 71121.03 inspection material

Liquid and Gaseous Discharge Permits and Dose Calculations

Liquid Waste Discharge Authorization Numbers: 03-01, 03-02, 03-03, 03-04, and 03-05
2003 to 2005 Gaseous EFFECTS Results

Notifications and Condition Reports

10245964, 10248714, 10249996, 10251470, 10289532, 10311652, 10315124, CR-CNS-2003-2752, CR-CNS-2003-3358, CR-CNS-2003-4682, CR-CNS-2003-5516, CR-CNS-2003-6333, CR-CNS-2004-0302, CR-CNS-2004-0869, CR-CNS-2004-2150, CR-CNS-2004-2476, CR-CNS-2004-3042, CR-CNS-2004-3330, CR-CNS-2004-5508, CR-CNS-2004-6358, CR-CNS-2004-6426, CR-CNS-2005-0240, CR-CNS-2005-0438, CR-CNS-2005-0462, CR-CNS-2005-0694, CR-CNS-2005-0984, CR-CNS-2005-1123, CR-CNS-2005-4117

Procedures

4.15 Elevated Release Point and Building Radiation Monitoring Systems, Revision 35
4.15.1 Elevated Release Point Radiation Monitoring System, Revision 10
4.15.2 Turbine Building Ventilation Radiation Monitoring System, Revision 6
4.15.5 Multipurpose Facility Ventilation Radiation Monitoring System, Revision 9

8.1	Chemistry Quality Control Program and General Laboratory Instructions, Revision 17
8.2.1	Chemistry Analysis Schedule, Revision 46
8.2.2	Instrument Performance Monitoring and Calibration Schedule, Revision 6
8.3	Control Parameters and Limits, Revision 43
8.5.2.1	Canberra System Operation, Revision 9
8.8.11	Liquid Radioactive Waste Discharge Authorization, Revision 26
8.11.1	EFFECTS Program, Revision 11
6.1SGT.501	SGT A Carbon Sample, Carbon Adsorber and HEPA Filter In-Place Leak Test and Component Leak Test (Division I), Revision 9
6.2SGT.501	SGT B Carbon Sample, Carbon Adsorber and HEPA Filter In-Place Leak Test and Component Leak Test (Division II), Revision 9
6.HV.104	Control Room Emergency Fan Charcoal and HEPA Filter Leak Test, Fan Capacity Test, and Charcoal Sampling, Revision 11

Miscellaneous

2003 and 2004 Cooper Nuclear Station Radioactive Effluent Release Report
2003 through 2005 Testing results for procedures 6.1SGT.501, 6.2SGT.501, and 6.HV.104
Chemistry Logbook
Chemistry On-line Monitor Unavailability Logbook
Control Room Logbook

Section 2PS2: Radioactive Material Processing and Transportation (71122.02)

Audits and Self Assessments

QA Audit 04-08 Radioactive Material Processing and Shipping

Condition Reports

2003-7535, 2004-5554, 2004-5559, 2005-0700, 2005-0806, 2005-0825, 2005-1182, 2005-3317, 2005-04886

Procedures

0.PCP.1	Process Control Program, Revision 0
9.RW.1	Radioactive Shipments, Revision 14
9.RW.2	Condensate Waste Resins, Spent Resins, RWCU Resins, and Waste Sludge Classification and Listing, Revision 10
9.RW.3	Dry Radioactive Waste Classification/Listing and Radioactive Material Shipments, Revision 3
9.RW.7	Waste Stream Sampling, Revision 7
9.RW.9	Filling Containers with Waste/Radioactive Material, Revision 8
2.5.4.4	NUPAC Dewatering System, Revision 8
9.RADOP.8	Receipt of Radioactive Materials, Revision 12

Shipping Paper Work and Manifests

03-12, 04-14, 05-02, 05-02N, 05-04, 05-05, 05-10

Certificates of Compliance

9208, 9233

Licenses

287-02, R-I2005-K01, R-86011-K01, R-73006-A02, R-73006-F13, South Carolina Department of Health and Environmental Control No. 097

Procedures

9.RESP.1	Respiratory Protection Program, Revision 7
9.RESP.2	Self-Contained Breathing Apparatus, Revision 11
9.RESP.4	Bauer FS-9 Air Compressor Air Quality Monitoring, Revision 5

Section 2PS3: Radiological Environmental Monitoring Program and Radioactive Material Control Program (71122.03)

Procedures

9.ENN-RP-106-1	Radiation and Contamination Surveys, Revision 0
9.ENV.8	Administering the CNS Meteorological Program (CNS MET), Revision 0
9.RADOP.2	Radiation Safety Standards and Limits, Revision 8
14.MET.301	Meteorological Maintenance Procedure for 10-Meter Tower, System A, Revision 1
14.MET.302	Meteorological Maintenance Procedure for 100-Meter Tower, System A, Revision 3

CNS Radiological Environmental Monitoring Program Administration, July 2004
Sampling Manual for the CNS Radiological Environmental Monitoring Program, July 2004
CNS Environmental Air Pump Calibration and Maintenance, July 2002
Action Levels for Environmental Samples, July 2002
Annual CNS Land Use Census, July 2002

Condition Reports

CR-CNS-2003-06205, CR-CNS-2003-06791, CR-CNS-2003-06927, CR-CNS-2003-06928, CR-CNS-2004-01951, CR-CNS-2004-06570, CR-CNS-2004-07699, CR-CNS-2005-00042, CR-CNS-2005-00054, CR-CNS-2005-00061, CR-CNS-2005-02384, CR-CNS-2005-02389, CR-CNS-2005-04231

Audit and Assessments

Cooper Nuclear Station Quality Assurance Surveillance Report QAD20050031, May 17, 2005
Quality Assurance Audit 03-08, Off-Site Dose Assessment Manual, September 8-25, 2003
Cooper Nuclear Station 2004 Environmental Assessment Report, June 7-15, 2004
Cooper Nuclear Station 2003 Environmental Assessment Report, September 8-17, 2003
NUPIC Audit/Survey of Teledyne Brown Engineering Environmental Services, Audit No. 18668, March 25-27, 2003

Calibration Records

Environmental Air Pump Station 1, Pump Serial # 6164, April 7, 2005
Environmental Air Pump Station 1, Pump Serial # 6164, March 16, 2005
Environmental Air Pump Station 2, Pump Serial # 6154, April 7, 2005
Environmental Air Pump Station 2, Pump Serial # 6154, October 19, 2004
Environmental Air Pump Station 10, Pump Serial # 6155, April 7, 2005
Environmental Air Pump Station 10, Pump Serial # 6155, October 19, 2004

Miscellaneous

2003 Annual Radiological Environmental Operating Report
2004 Annual Radiological Environmental Operating Report
Off-Site Dose Assessment Manual