



## A Framework for the Analysis of Localized Corrosion at the Proposed Yucca Mountain Repository

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## Introduction

- Corrosion is a primary determinant of waste package performance at the proposed Yucca Mountain Repository
  - > The most likely degradation process
  - Controls the delay time for radionuclide transport from the waste package
  - Determines when packages will be penetrated and the shape size and distribution of those penetrations
- In this presentation a framework for the analysis of localized corrosion is presented and demonstrated for a scenario
  - > Water chemistry of mixed salt solutions (sodium chloride-potassium nitrate)
  - > Time-temperature-relative humidity profiles for a hot, mid and cool temperature waste package



#### Methodology for Determination of Materials Performance

- Materials performance at the proposed Yucca Mountain Repository is amenable to a familiar and effective analytical methodology
  - > Widely accepted in the energy, transportation and other industries
- Three components comprise the analysis
  - > Definition of the performance requirements
  - Determination of the operating conditions to which materials will be exposed
  - Selection of materials of construction that perform well in those conditions
- A special feature of the proposed Repository is the extremely long time frame of interest, i.e. 10,000's of years and longer
  - > Time evolution of the environment in contact with waste package surfaces
  - > Time evolution of corrosion damage that may result



## The Proposed Yucca Mountain Repository

#### **Repository Reference Design Concept**





## The Proposed Yucca Mountain Repository







- Unsaturated zone, i.e. fractures and pores in rock are partially filled with water
- Desert area with about 18 cm of rain per year
- Atmospheric pressure
- Ambient waters are dilute and near neutral pH
- Concentrated waters can form by condensation, deliquescence and evaporation



## **Proposed Emplacement Drift**





## **Background on Ni-Cr-Mo Alloys**

- Alloy 22 belongs to a family of Ni-Cr-Mo alloys
  - Earlier alloys include C-276 and C-4 and later alloys include Inconel 686, Alloy 59, Hastelloy C-2000 and MAT-21
  - Alloy 22 (N06022) is a solid solution of Ni, Cr, Mo and W as the main alloying elements
  - Cr-Mo-W in Alloy 22 act synergistically to provide resistance to localized corrosion such as crevice corrosion
- Large industrial equipment in service for many years in harsh environments without corrosion
  - Alloy 22 has great toughness and over 50% elongation before failure
  - > Can be hot or cold formed and is weldable by many methods
  - > Can be fabricated into large structures and components



#### Corrosion Resistance is Crucial to Waste Package Performance

- Radionuclides are fully isolated if there are no penetrations
  - > Even penetrated package can limit radionuclide movement
- Corrosion rates of passive metals are extremely low
  - Realistic rates are less than 1 µm/yr (a millionth of a meter per year) and much less
  - > Alloy 22 layer is 2-cm thick (a stack of 12 U.S. quarters)
- Corrosion rates of approximately 0.01 µm/year are measured in exposures of over 5-years at the Long Term Test Facility at Lawrence Livermore National Laboratory



16,000 to 160,000 years to penetrate the thickness of one U.S. quarter for a corrosion rate of 0.1 to 0.01 μm/yr



#### Attributes of the Proposed Yucca Mountain Repository



- One long, slow heating/cooling cycle
  - Packages cool to ambient over several thousands of years
- Waste packages on support pallets
  - > No immersion in waters
- No moving parts
- Low heat fluxes, slow heating and cooling, and modest thermal gradients
- Radiation effects at waste package surface negligible after a few hundred years
- Limited amount of water moving through the rock
- Limited salts and minerals carried into drifts by incoming water and dust



## **Relevant Time Periods for Corrosion**



- This scenario is for Temp-RH shown above
  - Waste Package at 101°C when Drift Wall cooled to 96°C
  - > Critical Corrosion Temp 90°C

- I Emplacement of waste packages and preclosure
  - > Start to Year 50
- II Heat Up after closure
  - > Year 50 to ~65
- III Cool down/Thermal Barrier (drift wall above boiling temperature)
  - > Year ~65 to 750
- IV Cool Down/Dripping and Seepage Possible
  - > Year 750 to 1375
- V Waste Packages below Critical Temp for Corrosion
  - > Year 1375 and beyond
- Periods are determined by
  - > Temperature-RH conditions
  - > Time when drift wall reaches 96°C
  - > Critical Corrosion Temp for Alloy 22



## **Period IV-Dripping and Seepage Possible**

- When drift wall is below boiling temperature (96°C), dripping/seepage can occur
- Dripping/seepage can contact waste package surface
  - > Where both capillary barrier and drip shield are inoperative
  - > And dripping location is in aligned with drip shield penetration
- When these conditions are met
  - If waste package temperature above critical corrosion temperature
  - <u>Then</u> follow decision-tree analysis for local corrosion damage evolution



- Drift wall is below boiling at year 750
  - > Waste Package at 101°C
  - > Relative humidity 65%
- Waste Package is at 90°C at year 1375
  - > Relative Humidity 84%



#### Period IV Conditions for Mid, Hot and Cool Waste Packages





Drift Wall 96°C	Year	Waste Package Temp <sup>°</sup> C	Relative Humidity	Waste Package at 90°C
Mid WP	700	101	65	1325
Hot WP	1850	99	56	3000
Cool WP	62	102	72	125



# Categories of Waters



- Ambient Waters:
  - > Dilute solutions
  - > Na-Ca-Mg-HCO<sub>3</sub>-CO<sub>3</sub>-CI-NO<sub>3</sub>-SO<sub>4</sub>
  - > Near neutral pH
- Waters can be concentrated
  - > Modified during movement
  - > Thermal-chemical processes
- Modifications on waste package surface
- Chemical and electrochemical processes



## **Solution Chemistry Principles**



J. Payer-Advisory Committee on Nuclear Waste, Nuclear Regulatory Commission, Rockville MD, March 23, 2006

#### Constraints on Water Compositions for Sodium and Potassium Salts



J. Paver-Advisory Committee on Nuclear Waste, Nuclear Regulatory Commission, Rockville MD, March 23, 2006

#### Water Chemistry Scenarios for Waste Package

 T-RH Profiles Related to Brine Solution Compositions for Sodium and Potassium Base Salts





#### **Period IV Analysis of T-RH-Solution Composition**



Waste Package at 101°C; Relative Humidity 65%

Critical Corrosion Temp 90°C at year 1375; Relative Humidity 85%



The Temp-RH at any time fixes the possible waters. Can follow the trajectory with time

Number of non-corrosive solutions; Sodium chloride with low nitrate solutions can be corrosive



## **Decision-Tree Analysis**

- A decision-tree for localized corrosion
  - > Are environments and crevices present to induce localized corrosion?
    - >> Consider conditions in moist layers of particulate and deposits
  - > If localized corrosion initiates, will it persist?
    - Consider stifling and arrest processes as the corrosion proceeds
  - > What amount of metal penetration occurs?
  - > What is the size and distribution of corrosion sites?



#### **Decision-Tree Analysis**

#### A decision-tree for localized corrosion





## Summary

- Presented a framework for the analysis of localized corrosion
- Demonstrated the analysis for a scenario
  - > Water chemistry of mixed salt solutions
  - > Time-temperature-relative humidity profiles for waste packages
- Localized corrosion on waste packages is restricted to finite time periods
  - > Corrosion conditions at key time periods in proposed Repository
  - > Corrosion analysis during period IV-cool down/dripping and seepage
- Decision-tree analysis for corrosion damage evolution
  - > For those time periods when localized corrosion can be supported
  - > Based upon the temperature and possible water chemistries
  - Apply decision-tree analysis to determine the evolution of corrosion damage

