

ACRS MEETING WITH
THE U.S. NUCLEAR
REGULATORY
COMMISSION

October 20, 2006

OVERVIEW

GRAHAM B. WALLIS

Accomplishments

- Since our last meeting with the Commission on December 8, 2005, we issued 37 Reports:
- Topics included:
 - Early site permit applications for the Grand Gulf and Clinton sites

- Generic Safety Issue-191,
"Assessment of Debris
Accumulation on PWR Sump
Performance"
- Regulatory Guide 1.97, Rev. 4,
"Criteria for Accident
Monitoring Instrumentation for
Nuclear Power Plants"

- Regulatory Guide 1.205, "Risk-Informed, Performance-Based Fire Protection for Existing Light-Water Nuclear Power Plants"
- Proposed approach to enhance the Reactor Oversight Process to address safety culture issues

- Report on the NRC Safety Research Program (NUREG-1635, Vol. 7)
- Application of the TRACG computer code to evaluate the stability of the ESBWR

- SRP Section 14.2.1, "Generic Guidelines for Extended Power Uprate Testing Programs"
- Ongoing security-related activities
- Digital instrumentation and control system research

License Renewal

Since December 2005:

- Completed timely review of four applications (Browns Ferry, Brunswick, Nine Mile Point, and Monticello)
- Reviewed Generic Letter 2006-xx, "Inaccessible or Underground Cable Failures That Disable Accident Mitigation Systems"

- Reviewed interim results of the study to determine the need for establishing limits for phosphate ion concentration in groundwater to protect concrete structures of plants
- Will perform final review of Palisades application on an expedited schedule
- Performing interim review of Oyster Creek application

Power Upgrades

- Reviewed three extended power upgrade applications (Vermont Yankee, R.E. Ginna, and Beaver Valley)
- Will review the extended power upgrade application for Browns Ferry Units 1, 2, and 3 after receiving complete safety evaluation report

Risk-Informing 10 CFR 50.46

- Will review draft final rule to risk inform 10 CFR 50.46 at the November meeting
- At the request of the staff, deferred review of Regulatory Guide and draft final NUREG on 10 CFR 50.46 LOCA Break Frequency Evaluations

Ongoing/Future Activities

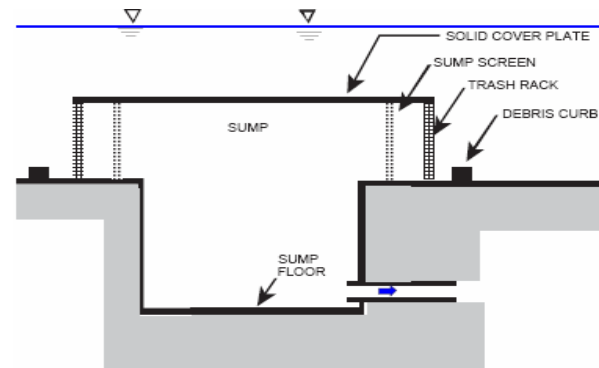
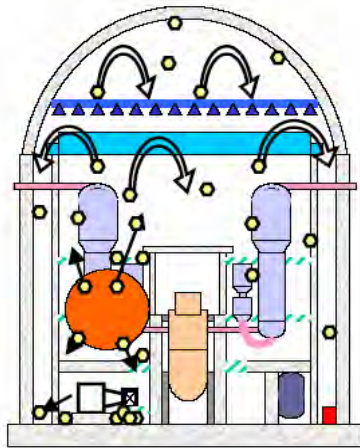
- Advanced reactor design certifications
- Assessment of research quality
- Combined license applications
- Digital instrumentation and control system matters
- Early site permit applications
- Extended core power uprates
- Fire protection

- High-burnup fuel and cladding issues
- Human reliability analysis
- License renewal applications
- Operating plant issues
- Report on the NRC Safety Research Program
- Resolution of GSI-191, "Assessment of Debris Accumulation on PWR Sump Performance"

- Revisions to Regulatory Guides and SRP Sections to support new reactor licensing
- Risk-informing 10 CFR 50.46
- SPAR models
- Safeguards and security matters
- Technology-neutral regulatory framework
- Thermal-hydraulic codes

PWR SUMP PERFORMANCE

Graham B. Wallis



ACRS Interactions With The Staff And Industry

- February and June 2006 – NRC Staff and NEI
- August 2006 - Strainer Vendors and PWR Owners Group
- September 2006 – Observation of Vendor Tests

Recent NRC Research

- Chemical effects
- Debris bed structure
- Coatings transport and head loss
- Head loss correlations
- Downstream effects

Industry Activities

- Prototypical full-scale tests of strainer modules
- Chemical effects testing
- Head-loss correlations
- Coatings
- Debris transport
- Downstream effects

The Path Forward

- ACRS letter April 10, 2006
 - PWR licensees are installing larger sump screens
 - Screen performance validated by “proof tests”
 - Debris formation and transport treated conservatively
 - Industry is performing additional needed research

- ACRS is questioning how the NRC will determine the adequacy of plant-specific solutions
- ACRS Letter August 1, 2006
 - Research efforts and development of predictive models should continue

ACRS 2006 REPORT ON NRC SAFETY RESEARCH PROGRAM

Mario V. Bonaca

Scope

- Research projects dealing with the safety of existing plants and advanced LWR designs submitted for certification

Considerations

- Justification for the research
- Technical approach
- Progress of the work

General Observation

- NRC Safety Research Program is focused on addressing near term regulatory needs of the agency with an appropriate concentration in three disciplines:
 - Materials and Metallurgy
 - Probabilistic Risk Assessment
 - Thermal Hydraulics

Notable RES Accomplishments

- Multidisciplinary development of technical basis for revising the PTS screening criterion in the PTS Rule
- Performance of high-burnup fuel during reactivity transients
- Embrittlement of zirconium alloy cladding when taken to high burnup

International Collaboration

- The NRC has developed effective collaborations with other countries on reactor safety research
- More collaboration with Asian countries having active nuclear power programs should be pursued

Support For Future Licensing Activities

- NRC's resources for advanced reactor safety research are appropriately focused on addressing issues associated with ESBWR and EPR
- NRC needs to provide clear guidance on its expectations for the experimental validation of computer models to be used in the licensing of advanced reactors that do not use familiar technologies

Opportunities For Independent Research

- Focusing NRC research entirely on the immediate needs of the line organizations reduces the opportunities for independent research by RES that could improve the regulatory process

A Vision For Future

- A portion of research program needs to be devoted to the development of a regulatory infrastructure to deal with future regulatory demands
- The ACRS can foresee, for example, a time when regulatory staff has routine access to superior tools for system analysis, phenomenological analysis, and risk assessment

LESSONS LEARNED FROM THE REVIEW OF EARLY SITE PERMIT APPLICATIONS

William J. Shack

Early Site Permit Applications

- North Anna
- Grand Gulf
- Clinton

Generic Lessons

- Common understanding of expectations between staff and applicants
- Internet Data
- Electronic submissions
- Criteria for permit conditions and COL action items
- Performance-based seismic hazard analysis

Other Lessons

- Definitions of pertinent site characteristics
- Guidance on plant parameter envelope
- Guidance on major features of emergency plans
- Calculation of probable maximum flood

- High frequency ground motion
- Weather cycles
- Lessons learned should aid considerably preparation and processing of COL applications

FUTURE PLANT DESIGN
ACTIVITIES AND
COORDINATION WITH
STAFF ON THE MASTER
INTEGRATED SCHEDULE

Thomas S. Kress

ACRS Activities on New Reactor Designs

- Early Site Permit Applications
- Design Certifications
- Coordination on the development of Master Integrated Schedule
- COL Applications
- Revisions to Reg. Guides and SRP Sections
- 10 CFR Part 52

Early Site Permit Applications

- Completed review of ESP applications for North Anna, Grand Gulf, and Clinton
- Issued a lessons learned report
- Will review Vogtle ESP application after receiving staff's SER

ESBWR Design

- Reviewed application of TRACG Code to analyze ESBWR stability
- ACRS letter dated April 26, 2006:
 - Recommended that the staff approve the use of TRACG to analyze the stability of ESBWR during normal operation, anticipated operational occurrences, and the low-power phase of reactor startup

- Reviewed Draft SER related to ESBWR pre-application
- Agreed with the staff's decision to approve the TRACG code for use in analyzing ESBWR in response to LOCA scenarios
- Identified issues that should be considered when using TRACG for design certification purpose
- Reviewing ESBWR PRA
- Will review staff's SER for ESBWR design certification in 2007

Master Integrated Schedule

- Memorandum from former Chairman Diaz recommending ACRS coordination in the development of Master Integrated Schedule
- Provided ACRS planning assumptions for reviewing ESP, Design Certification, and COL applications

- Discussed the staff's proposed Master Project Management Plan at the October ACRS meeting
- Will continue to coordinate with the staff in the development and refinement of the Master Integrated Schedule and the Master Project Management Plan

COL Applications

- Established design-specific COL Subcommittees
 - R-COL
 - S-COL
- Review to focus on COL action items and on plant and site-specific features
- Review of topic-specific issues

Guidance For New Reactor Licensing

- COL application guidance
(DG-1145)
- RG revisions
- SRP revisions

ACRS Recommendations Related To Proposed Revisions To Part 52

- Level-3 PRA at the ESP stage should not be required
- Support a requirement for COL holders to maintain an up-to-date PRA
- Major features of the site emergency plan
- Operation up to 5 percent power even with deficiencies in emergency preparedness

ABBREVIATIONS

ACRS	Advisory Committee on Reactor Safeguards
CFR	Code of <i>Federal Regulations</i>
COL	Combined license
CY	Calendar year
DG	Draft guide
EPR	Evolutionary Power Reactor
ESBWR	Economic Simplified Boiling Water Reactor
ESP	Early site permit
GSI	Generic Safety Issue
LWR	Light-water reactor
LOCA	Loss-of-coolant accident
NEI	Nuclear Energy Institute
NRC	Nuclear Regulatory Commission
PRA	Probabilistic risk assessment
PTS	Pressurized thermal shock
PWR	Pressurized water reactor
R-COL	Reference combined license
RES	Office of Nuclear Regulatory Research
RG	Regulatory guide
S-COL	Subsequent combined license
SER	Safety evaluation report
SPAR	Standardized Plant Analysis Risk Model
SRP	Standard Review Plan
U.S.	United States