

Regulatory Guides Not Planned To Be Updated by December 2010

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1.112	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light-Water-Cooled Power Reactors
1.121	Bases for Plugging Degraded PWR Steam Generator Tubes
1.132	Site Investigations for Foundations of Nuclear Power Plants
1.138	Laboratory Investigations of Soils and Rocks for Engineering Analysis and Design of Nuclear Power Plants
1.14	Reactor Coolant Pump Flywheel Integrity
1.140	Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Normal Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants
1.142	Safety-Related Concrete Structures for Nuclear Power Plants (Other than Reactor Vessels and Containments)
1.152	Criteria for Digital Computers in Safety Systems of Nuclear Power Plants
1.161	Evaluation of Reactor Pressure Vessels with Charpy Upper-Shelf Energy Less Than 50 Ft-Lb
1.166	Pre-Earthquake Planning and Immediate Nuclear Power Plant Operator Postearthquake Actions
1.181	Content of the Updated Final Safety Analysis Report in Accordance with 10 CFR 50.71(e)
1.184	Decommissioning of Nuclear Power Reactors
1.185	Standard Format and Content for Post-Shutdown Decommissioning Activities Report
1.188	Standard Format and Content for Applications To Renew Nuclear Power Plant Operating Licenses
1.190	Calculational and Dosimetry Methods for Determining Pressure Vessel Neutron Fluence
1.191	Fire Protection Program for Nuclear Power Plants During Decommissioning and Permanent Shutdown
1.192	Operation and Maintenance Code Case Acceptability, ASME OM Code
1.195	Methods and Assumptions for Evaluating Radiological Consequences of Design Basis Accidents at Light-Water Nuclear Power Reactors
1.197	Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors
1.198	Procedures and Criteria for Assessing Seismic Soil Liquefaction at Nuclear Power Plant Sites
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1.204	Guidelines for Lightning Protection of Nuclear Power Plants
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1.25	Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors
1.26	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants
1.29	Seismic Design Classification
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1.31	Control of Ferrite Content in Stainless Steel Weld Metal
1.32	Criteria for Power Systems for Nuclear Power Plants
1.35	Inservice Inspection of UngROUTed Tendons in Prestressed Concrete Containments
1.35.1	Determining Prestressing Forces for Inspection of Prestressed Concrete Containments
1.36	Nonmetallic Thermal Insulation for Austenitic Stainless Steel
1.41	Preoperational Testing of Redundant On-Site Electric Power Systems To Verify Proper Load Group Assignments
1.5	Assumptions Used for Evaluating the Potential Radiological Consequences of a Steam Line Break Accident for Boiling Water Reactors
1.52	Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Post-Accident Engineered-Safety-Feature Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants
1.53	Application of the Single-Failure Criterion to Nuclear Power Plant Protection Systems
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