

Byron 1

2Q/2007 Plant Inspection Findings

Initiating Events

Significance:  Jun 30, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO FIRE PROOF STRUCTURAL STEEL BEAMS TO ACHIEVE A 3-HOUR FIRE RATING.

The inspectors identified an NCV of Byron Station's Operating License Condition 2.C.6 for failure to maintain a 3-hour rated firewall in the control room heating, ventilation and air conditioning (HVAC) equipment room.

Specifically, the walls between the upper cable spreading rooms and the control room HVAC equipment were not fireproofed to achieve a 3-hour rating as required by the fire protection report and applicable plant drawings. The licensee entered this issue into its corrective action program for resolution and implemented compensatory measures that included hourly fire watches.

This finding was more than minor because it was associated with the external factor attribute of the Initiating Events cornerstone related to fire and it affected the cornerstone's objective to limit the likelihood of fire that upset plant stability and challenge critical safety functions during shutdown as well as power operations. This finding was of very low safety significance because there were no fire ignition source scenarios that would have caused the structural steel beams to weaken and the ceiling to collapse.

Inspection Report# : [2007003](#) (*pdf*)

Significance:  Jun 30, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

DESIGN BASIS ANALYSIS FOR THE POSTULATED DROP OF A REACTOR VESSEL HEAD DURING REFUELING WAS NOT UP-TO-DATE.

The inspectors identified a finding of very low safety significance and associated NCV for a failure to establish measures to assure that regulatory requirements and the design basis were correctly translated into procedures as required by 10 CFR 50 Appendix B Criterion III. Specifically, the procedures related to the reactor vessel head lift did not correctly reflect in a non conservative direction the design lift height. As immediate corrective actions, the licensee incorporated compensatory measures to lower reactor cavity water level during the head lift to ensure the actual airdrop distance was bounded by the analysis.

It was more than minor because it involved the equipment performance attribute of the Initiating Events Cornerstone Objective. The finding was determined to be of very low safety significance (Green) because as part of the additional corrective actions, the licensee's subsequent calculations showed the lift height was acceptable due to margin gained from the much heavier head weight assumed in their analysis.

This finding has a cross-cutting aspect in the area of Problem Identification and Resolution, operating experience because the licensee failed to implement and institutionalize operating experience through changes to their procedures (P.2(b)).

Inspection Report# : [2007003](#) (*pdf*)

Mitigating Systems

Significance:  Jun 30, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

ALTERNATIVE SHUTDOWN USING THE REMOTE SHUTDOWN PANEL.

The inspectors identified a finding of very low safety significance and an associated NCV of 10 CFR Part 50, Appendix B, Criterion XI, "Test Control," for failure to ensure that all testing necessary to demonstrate that the Unit 1 and 2 remote shutdown panels (RSPs) will perform satisfactorily in-service be identified and conducted. Specifically, the licensee failed to periodically test applicable (i.e., important to safety) components (e.g., control switches) on the RSPs to ensure the operability and functional performance of the RSP components and the operability of their associated systems as a whole. The licensee's corrective actions were to immediately begin testing of the instrumentation and controls located at the RSP and to continue the testing in accordance with a schedule that would allow timely completion.

The finding was more than minor because the finding was associated with the equipment performance attribute of the Mitigating Systems Cornerstone and affected the cornerstone objective to ensure the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences (i.e., core damage). The finding was of very low safety significance because the finding did not represent an actual loss of the instrumentation indications and control functions at the RSP, since the 1BOA PRI-5 and 2BOA PRI-5 procedures' "Response Not Obtained" column provided an alternative shutdown capability method using local manual actions and the Fire Hazards Panel.

Inspection Report# : [2007003](#) (*pdf*)

Significance:  Mar 31, 2007

Identified By: NRC

Item Type: FIN Finding

INADEQUATE SETPOINT CONTROL OF THE OIL LEVEL TO SAFETY RELATED PUMPS.

The inspectors identified a finding for the licensee's failure to maintain setpoint control of the constant level oilers. Specifically, the licensee did not incorporate the vendor's recommendation on setting the oil level for the essential service water pumps. This condition increased the challenges to the proper functioning of the lubricating oil and thus to the bearings of the safety-related pumps. The licensee subsequently reset the oil level for the pumps to the recommended setting and entered this issue into their corrective action program.

this finding is more than minor because of the potential for degradation of oil and bearings to safety related components, which could adversely affect their availability and reliability. This finding is of very low safety significance because no bearings had been damaged due to the high oil levels despite operating in this condition for many years and no significant oil degradation had occurred. The inspectors did not identify a violation of regulatory requirements. However, the cause of the finding is related to the cross-cutting element of problem identification and resolution, particularly the thoroughness of the extent of condition review. (Section 1R04.2)

Inspection Report# : [2007002](#) (*pdf*)

Significance:  Mar 31, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

ADEQUACY OF SAFE SHUTDOWN PROCEDURES TO ADDRESS DRAINING OF THE RWST.

The inspectors identified a Non-Cited Violation (NCV) of the Byron Station Operating License for the failure to have adequate alternate safe shutdown procedure. Specifically, licensee's procedure BOP FR-1, "Fire Response Guidelines," did not include adequate steps and instructions to prevent the draining of the refueling water storage tank (RWST) into the containment sump in the event of a fire in the auxiliary electrical equipment room (AEER) or the control room. The licensee implemented appropriate procedure changes for both the AEER and control room fire zones to isolate all potential RWST drain paths.

The finding is greater than minor because it affected the attribute of procedure quality for protection against external factors and it impacted the objective of the mitigating systems cornerstone. The failure to provide adequate instructions in the alternate shutdown procedure to promptly prevent the draining of the RWST to the containment sump could have adversely impacted the operators' ability to promptly take appropriate actions and could have complicated safe shutdown in the event of a fire. The finding was of very low safety significance based on Phase 2

and Phase 3 SDP evaluations completed by the Region III senior reactor analyst (SRA) in accordance with IMC 0609, Appendix F, "Fire Protection Significance Determination Process." (Section 1R05.2)

Inspection Report# : [2007002](#) (*pdf*)

Significance:  Jan 16, 2007

Identified By: NRC

Item Type: FIN Finding

FAILURE TO HAVE SETPOINT CONTROL OF THE CONSTANT LEVEL OILERS ON SAFETY-RELATED PUMPS

The inspectors identified a finding of very low safety significance associated with the failure to maintain control of the setpoints for constant level oilers. This condition increased the challenges to the proper functioning of the lubricating oil and thus to the bearings to the safety-related pumps.

This finding was considered more than minor because of the potential for the degradation of oil/bearings to safety-related components which would increase their unavailability and unreliability. This finding was of very low safety significance because no bearings had been damaged due to the high or low oil levels despite operating in this condition for many years and the oil had only been moderately impacted. The licensee's corrective actions included assessing the setpoints of other safety related and non-safety related pumps, verifying no pumps had been damaged, and revising the work order template to include the reference to the corporate procedure for the setting of constant level oilers. No violation of NRC requirements occurred.

Inspection Report# : [2006005](#) (*pdf*)

Significance:  Dec 01, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Non-Conservative Errors in Unverified Design Input Data Used to Determine the Impact of Core Power Uprate on Medium Voltage Loads

Green. The team identified a NCV of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," having very low safety significance. The power uprate electrical loading calculation used incorrect design input for the 4160 Vac engineered safety features (ESF) distribution system load analysis. Specifically, the licensee's contract engineering organization failed to adequately verify design input data used to determine brake horsepower loading. The incorrect horsepower values were subsequently used in revising the 4160 Vac ESF distribution system power analysis. The licensee's acceptance review did not identify the problem. Using corrected values, the licensee determined that the reduction in load margin was acceptable based on a revised loading calculation prepared informally during the inspection.

The finding was more than minor because failing to correctly identify, verify, and input the correct design data into the electrical power analysis program resulted in the load conditions not being adequately evaluated, resulting in inaccurate and non-conservative determination of loading and load margin for the 4160 Vac ESF buses. The finding was of very low safety significance based on the results of the licensee's analysis and screened as Green using the SDP Phase 1 screening worksheet.

Inspection Report# : [2006009](#) (*pdf*)

Significance:  Dec 01, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Acceptance Criteria for Safety Injection Pump NPSH Not Met

Green. The team identified a NCV of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," having very low safety significance. Specifically, the calculation for evaluating the net positive suction head (NPSH) for the safety injection pump contained errors and failed to demonstrate that the acceptance criteria was met. To demonstrate operability, the licensee performed a preliminary calculation, using a less conservative pump flow value along with correcting the identified errors.

The finding was more than minor because the calculation of record was not adequate and failed to demonstrate that

the NPSH available met design basis requirements. The finding was of very low safety significance based on the results of the licensee's corrected analysis and screened as Green using the SDP Phase 1 screening worksheet. The cause of the finding was related to the cross-cutting aspect of human performance.

Inspection Report# : [2006009](#) (*pdf*)

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Significance: Sep 30, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO MAINTAIN FIRE BARRIERS IN ACCORDANCE WITH FIRE PROTECTION PROGRAM.

The inspectors identified a Non-Cited Violation of Byron Facility Operating License Nos. NPF-37 and NPF-66, Condition 2.c.6, for failing to maintain the firewall separating the Auxiliary Building from the penetration area in accordance with the approved fire protection program. Fire seals were required to be provided in this firewall, except where an evaluation had been performed and approved to allow a deviation. Two sleeves containing fire seals had pulling ropes embedded in the fire seals in the firewall separating the Auxiliary Building General Area 401 from the Unit 1 piping penetration area; also, no evaluation or exemption existed to justify this configuration. The licensee entered the issue into its corrective action program for resolution and implemented compensatory measures that included hourly fire watches.

This finding was more than minor because it affected the Mitigating Systems Cornerstone objective to ensure that external factors (i.e., fire, flood, etc) do not impact the availability, reliability, and capability of systems that respond to initiating events. The finding was of very low safety significance because the fire seals were in small diameter sleeves that traveled a distance of 45 feet and had two 90 degree bends and the location of combustibles were positioned such that the piping penetration end of the fire seals would not be subject to direct flame impingement.

Inspection Report# : [2006004](#) (*pdf*)

Barrier Integrity

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Significance: Jun 30, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO PROPERLY REINSTALL A FLOOD SEAL HATCH / HIGH ENERGY LINE BREAK BARRIER.

The inspectors identified a finding of very low safety significance and an associated NCV of Technical Specification (TS) 5.4.1 regarding procedure adherence when a flood seal hatch, which also served as a High Energy Line Break (HELB) barrier, was found to have been improperly reinstalled following maintenance.

The finding was more than minor because it involved the attribute of procedure adherence and could have affected containment isolation valves necessary to ensure isolation of the secondary side of the steam generators. As such, this finding affected the containment barrier cornerstone. The finding was of very low safety significance because the finding did not represent an actual loss of the Unit 1 Auxiliary Feedwater System Containment Isolation Valves. Subsequently, the licensee repaired the hatch to ensure a proper seal.

This finding has a cross-cutting aspect in the area of human performance, work practices, supervisory oversight because the licensee's planned work activities did not effectively ensure the protection of a safety system in a mild environment from the effects of a harsh environment (H.4(c)).

Inspection Report# : [2007003](#) (*pdf*)

Emergency Preparedness

Occupational Radiation Safety

Significance:  Sep 30, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO POST AND CONTROL A HIGH RADIATION AREA.

An inspector-identified finding of very low safety significance and two associated Non-Cited Violations of NRC requirements were identified for the failure to post and control access to High Radiation Areas, as required by 10 CFR Part 20, to notify individuals of the radiological hazard present and to prevent the unauthorized entry to such areas. Specifically, the entrance to the Unit 1 Filter Valve Aisle located on the 383' elevation of the Auxiliary Building, a high radiation area with a radiation dose rate of approximately 135 millirem in one hour, was not posted or controlled by any of the methods described in 10 CFR 20.1902, 10 CFR 20.1601, or Technical Specification 5.7.1.

The issue was more than minor because the issue was associated with the Program/Process attribute of the Occupational Radiation Safety Cornerstone and affected the cornerstone objective to ensure adequate protection of the worker health and safety from exposure to radiation from radioactive material during routine civilian nuclear reactor operation. The issue represents a finding of very low safety significance because the finding did not constitute an ALARA or work control issue, did not result in an overexposure or the substantial potential for an overexposure, and did not compromise the licensee's ability to assess dose. Non-Cited Violations of 10 CFR 20.1902 and 10 CFR 20.1601 were identified for the failure to post and control access to high radiation areas. Corrective actions taken by the licensee for this finding included establishing control through postings and barricades. The cause of this finding is related to the cross-cutting element of human performance.

Inspection Report# : [2006004](#) (*pdf*)

Public Radiation Safety

Significance:  Sep 30, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO EVALUATE THE POTENTIAL RADIOLOGICAL HAZARD ASSOCIATED WITH THE LEAKAGE OF WATER FROM THE VACUUM BREAKER VALVE VAULT.

An inspector-identified finding of very low safety significance and an associated Non-Cited Violation of NRC requirements were identified for the failure to perform surveys that are necessary to comply with the regulations in 10 CFR Part 20 and that are reasonable under the circumstances to evaluate the extent of radiation levels, concentrations or quantities of radioactive materials, and the potential radiological hazards that could be present prior to pumping liquids from blowdown line vacuum breaker valve vaults to the environment. Specifically, the conditions found at OCW276 (vault No. 6) on July 7, 2005, were outside the parameters of the original assessment, and the licensee did not evaluate the change of conditions for the potential radiological hazards to ensure compliance with 10 CFR 20.1301, which limits radiation exposure to a member of the public to 0.1 rem.

The issue was more than minor because the issue was associated with the Program/Process attribute of the Public Radiation Safety Cornerstone and affected the cornerstone objective to ensure adequate protection of public health and safety from exposure to radioactive materials released into the public domain as a result of routine civilian nuclear reactor operation. Since the releases were limited to licensee owned property, the licensee has not measured any licensed material beyond its property line, and the licensee's REMP has a monitoring well in the vicinity of the blowdown lines, the finding did not represent a failure to assess dose nor a failure to assess the environmental impact. Consequently, the finding was determined to be of very low safety significance. A Non-Cited Violation of 10 CFR 20.1501 was identified for the failure to make surveys to ensure compliance with 10 CFR 20.1301, which limits radiation exposure to a member of the public to 0.1 rem. Corrective actions taken by the licensee for this finding included performing surveys of the soil surrounding the vacuum breaker vault for radionuclides, establishing additional groundwater monitoring wells, sealing the vacuum breaker vaults, and installing of an automated leak detection system. The cause of this finding is related to the cross-cutting element of problem identification and resolution.

Physical Protection

Although the NRC is actively overseeing the Security cornerstone, the Commission has decided that certain findings pertaining to security cornerstone will not be publicly available to ensure that potentially useful information is not provided to a possible adversary. Therefore, the [cover letters](#) to security inspection reports may be viewed.

Miscellaneous

Last modified : August 24, 2007