Clinton 3Q/2005 Plant Inspection Findings

Initiating Events

Significance: G

Sep 30, 2005

Identified By: NRC Item Type: FIN Finding

PERFORMANCE OF WORK IN THE OFF-GAS SYSTEM THAT RESULTED IN A SUBSEQUENT LOSS IN OFF-GAS SYSTEM FLOW AND THE OPERATORS PERFORMING A RAPID POWER REDUCTION.

On August 29, 2005, a finding of very low safety significance was self revealed following the performance of work in the off-gas system that resulted in a subsequent loss in off-gas system flow and the operators performing a rapid power reduction. The finding involved the failure to stroke a gas dryer inlet valve to ensure the valve would operate following a packing adjustment. This issue was caused by poor work practices and communication by licensee personnel.

The issue was more than minor because it affected the Reactor Safety/Initiating Event cornerstone objective of limiting the likelihood of those events that upset plant stability. The finding was of very low safety significance because it would not affect the availability of mitigating systems or functions even if it had resulted in a plant trip. No violation of NRC requirements occurred. The finding also affected the cross cutting area of Human Performance.

Inspection Report#: 2005008(pdf)

Significance: G

Sep 30, 2005

Identified By: NRC Item Type: FIN Finding

THE LICENSEE FAILED TO TAKE PROMPT ACTION TO CORRECT A PROBLEM WITHIN THE ELECTRO-HYDRAULIC CONTROL SYSTEM.

On July 17, 2005, a finding of very low safety significance was identified by the inspectors when the licensee failed to take prompt action to correct a problem within the electro-hydraulic control system. In April 2005, one main turbine combined intermediate valve went shut at power due to a clogged servo valve strainer, causing a plant transient. The licensee identified that other main turbine valves were susceptible to the same failure, but did not take action to correct the problem until after a second combined intermediate valve went shut three months later, causing a second plant transient.

The issue was more than minor becuse the licensee knew of the degraded condition and associated risks and failed to correct the problem before it resulted in a second plant transient requiring operators to respond. The finding was of very low safety significance because it would not affect the availability of mitigating systems or functions even if it had resulted in a plant trip. No violation of NRC requirements occurred. Inspection Report#: 2005008(pdf)

Significance:

Mar 31, 2005

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO ESTABLISH ADEQUATE COMPENSATORY ACTIONS (HOURLY FIRE WATCH) ACCORDING TO FIRE PROTECTION PROGRAM PROCEDURES.

A finding of very low safety significance was identified by the inspectors on March 17, 2005, for a violation of license-required fire protection program requirements. The licensee failed to establish adequate hourly fire watches for a failed ionization detector as required by the approved fire protection program procedure. Following the inspectors' identification of this issue, the licensee established an hourly fire watch that met the requirements and recommendations of the licensee's approved fire protection program procedures.

This finding was more than minor because if left uncorrected, it could become a more significant safety concern. The licensee's ability to quickly detect a fire in the area was impaired due to an insufficient number of smoke detectors. The issue was of very low safety significance because the fire protection element impacted by the finding was still expected to provide some defense-in-depth benefit due to a second fire detector located in the room. Additionally, there were two nearby hose stations which could be used for fire suppression activities. The issue was a Non-Cited Violation of the facility operating license section 2.F which required the implementation of the fire protection program. Inspection Report#: 2005003(pdf)



Jun 30, 2005

Identified By: NRC Item Type: FIN Finding

IMPROPERLY SECURED 4160V EQUIPMENT DOORS

In December 2003 the inspectors identified a discrepant condition on the 4160 volt Bus 1C1 Reserve Feed potential transformer cubicle. The inspectors considered this to be an inspection finding with no violations of NRC requirements identified.

The inspectors determined that the issue was more than minor because the finding could be reasonably viewed as a precursor to a significant event, which if left uncorrected, could render safety related equipment inoperable. The issue was a design/seismic qualification deficiency that was determined not to cause a loss of a safety related function by the licensee's evaluation. Based on this conclusion, this finding was determined to be of very low safety significance using the Phase 1 worksheets.

Inspection Report# : 2005007(pdf)

Significance:

Sep 30, 2004

Identified By: NRC Item Type: FIN Finding

DIVISION-3 ESSENTIAL SWITCHGEAR HEAT REMOVAL (VX) SYSTEM TRIPPED DUE TO INADEQUATE IMPACT STATEMENT FOR MAINTENANCE.

A finding of very low safety significance was self-revealed during a maintenance activity when Division essential switchgear heat removal was lost as a result of an inadequate impact statement in the work order. The primary cause of this finding was related to the cross-cutting area of Human Performance. In addition to the maintenance planner missing the relationship between the safety and non-safety supply fan motors, several other opportunities to identify this inadequate impact statement were missed.

This finding was more than minor because with the division three essential switchgear heat removal system unavailable, the high pressure core spray system may be rendered inoperable. The issue was of very low safety significance because the initial temperature in the division three switchgear room was low and the loss of essential switchgear heat removal was of short duration, the high pressure core spray system was never actually inoperable. No violation of NRC requirements occurred.

Inspection Report# : 2004006(pdf)

Barrier Integrity

Significance:

Mar 31, 2005

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO FOLLOW PROCEDURE AND APPROPRIATELY ANNOTATE PORTIONS AS NOT APPLICABLE DURING THE PERFORMANCE OF REQUIRED CALIBRATION PROCEDURE IN ACCORDANCE WITH TS 5.4.1.

Through a self-revealing event (unexpected de-energized relay found during maintenance) the inspectors identified a Non-Cited Violation (NCV) of very low safety significance. This finding resulted from licensee personnel incorrectly designating procedureal steps as not applicable during the performance of a calibration procedure, Clinton Power Station (CPS) 9432.60, "Channel Functional Test for Containment Building Exhaust Radiation Monitor," required by Technical Specifications. In Issue Report (IR) 289643, the licensee documented that with the realy de-energized the affected primary containment isolation valve cannot by opened without taking the corresponding Division 2 LOCA BYPASS switch to the BYPASS position (an action administratively controlled by Operations).

The inspectors determined that the finding was greater than minor because this issue could be reasonably viewed as a precursor to a more significant event. Additionally, this finding was associated with the Barrier Integrity Cornerstone objective of providing reasonable assurance that physical design barriers protect the public from radioactive releases caused by accidents or events. The finding was of very low safety significance because this issue did not cause an actual open pathway in the physical integrity of reactor containment. The licensee documented the issue in IR 289643 and generated corrective actions as the result of a human performance investigation report being performed. These corrective actions included revising CPS 9432.60 to clearly identify the reason for placing the switch to BYPASS. Inspection Report# : 2005003(pdf)

Emergency Preparedness

Occupational Radiation Safety

Public Radiation Safety

Significance:

Mar 31, 2005

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO MAINTAIN CONTROL OF LICENSED RADIOACTIVE MATERIAL IN ACCORDANCE WITH 10 CFR 20,

SUBPART 1.

A finding of very low safety significance and an associated Non-Cited Violation were identified through a self-revealing event on October 7, 2004, when licensee personnel discovered that three nuclear instrument detectors (containing a very small amount of radioactive material) were not adequately controlled. Licensee personnel believed that the material was contained in a small container which was sealed in 1991 as part of a disposition plan for the defective instruments. The licensee' search of other material containers and documentation failed to identify the final disposition of the radioactive material.

The issue was more than minor because it was associated with the Human Performance and Programs/Process attributes of the Public Radiation Safety Cornerstone and affected the cornerstone objective of ensuring adequate protection of public health and safety from exposure to radioactive materials potentially released into the public domain. Based on various dose calculation scenarios, the very small amount of missing radioactive material would contribute a negligible radiological dose if a member of the public were to be exposed to the material. Additionally, the inspectors determined that the licensee did not have any prior radioactive material control occurrences in the previous 8 quarters. Therefore, the finding was of very low safety significance. The licensee's corrective actions for this issue included the development of procedural guidance which prohibits removing nuclear instrument detectors from the cabling as part of a disposition plan for defective units. One Non-Cited Vilation for the failure to control licensed radioactive material in accordance with 10 CFR 20, Subpart 1, was identified. Inspection Report#: 2005003(pdf)

Physical Protection

Physical Protection information not publicly available.

Miscellaneous

Last modified: November 30, 2005