

Calvert Cliffs 1

1Q/2005 Plant Inspection Findings

Initiating Events

Significance:  Jun 30, 2004

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to assess and manage risk associated with Unit 1 RPS power supply replacement activities during reduced inventory. (Section 1R13)

The inspectors identified a non-cited violation of 10 CFR 50.65(a)(4) which requires that before performing maintenance activities (including but not limited to surveillance, post-maintenance testing, and corrective and preventive maintenance), the licensee shall assess and manage the increase in risk that may result from the proposed maintenance activities. Specifically, the licensee failed to identify and therefore assess and manage the risks associated with performing maintenance on the 'B' channel of the reactor protection system (RPS) while operating in a reduced inventory condition. This maintenance activity resulted in the loss of one of the two shutdown cooling (SDC) operating trains for about 18 minutes with a corresponding heatup of the reactor coolant system (RCS) of 2 degrees Fahrenheit (F).

This finding is greater than minor because it affected an attribute and objective of the Initiating Event Cornerstone in that human performance inadequacies resulted in an event that upset plant stability during shutdown operations. This issue was evaluated in accordance with NRC Inspection Manual Chapter (IMC) 0609, Appendix G, Shutdown Operations SDP, and was determined to be of very low safety significance. The inspectors identified that a contributing cause of this finding was related to the cross-cutting area of Human Performance. (Section 1R13)

Inspection Report# : [2004005\(pdf\)](#)

Significance:  Jun 18, 2004

Identified By: Self Disclosing

Item Type: FIN Finding

Failure to comply with station work control procedures. (Section 3.4)

A self-revealing event identified a finding in that CCNPP did not follow procedural requirements in their risk assessment and control of the work on March 20, 2004, which resulted in an unanticipated reactor trip. Specifically, the provisions and controls of procedures NO-1-100, "Conduct of Operations," NO-1-117, "Integrated Risk Management," and MN-1-100, "Conduct of Maintenance," were not followed.

This finding was more than minor because the failures to follow station procedures affected the Initiating Events cornerstone in that the failure to properly risk-classify and control the work in the control room on March 20 lead to the reactor trip. This finding had very low safety significance because the finding did not represent an actual loss of a safety function, and was not potentially risk significant due to an external initiating event.

A contributing cause of the finding was related to the Human Performance cross-cutting area because CCNPP managers and staff did not properly implement station operations, risk management, and maintenance procedures. (Section 3.4)

Inspection Report# : [2004008\(pdf\)](#)

Significance:  Jun 18, 2004

Identified By: Self Disclosing

Item Type: FIN Finding

Failure to adequately implement a modification design review of the digital feedwater control system. (Section 2.4)

A self-revealing finding of very low safety significance was identified because CCNPP failed to perform an adequate design review which resulted in reduced reliability of the digital feedwater system during a plant event on March 20, 2004.

This finding was more than minor because it effected the design control attributes of the Initiating Events cornerstone. Incorrectly specifying the design voltage resulted in reduced reliability of the digital feedwater control system which increased the likelihood of an event that upset plant stability during power operation. This finding was of very low safety significance, because one of two turbine driven feedwater pumps and one of three condensate and condensate booster pumps remained operable during the Unit 1 March 20, 2004, event. (Section 2.4)

Inspection Report# : [2004008\(pdf\)](#)

Significance:  May 28, 2004
Identified By: NRC

Item Type: NCV NonCited Violation

Inadequate Combustible Material Control in Unit 1 69' West Electrical Room

The team identified a non-cited violation of License Condition 3.E, because Calvert Cliffs Nuclear Power Plant was not maintaining control of combustible materials in the Unit 1 69' West Electrical Room as described and approved in the safety evaluation report issued September 14, 1979.

Since the finding affected the initiating events cornerstone objective the finding is more than minor. The finding is of very low safety significance because the material was not located below cable trays carrying safety related cables and the material had been evaluated in the combustible loading calculations. (Section 1RO5.4)

Inspection Report# : [2004003\(pdf\)](#)

Mitigating Systems

Significance:  Mar 31, 2005

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Change SRW Operating Procedure During Sequencer Modification

The inspectors identified a non-cited violation of Technical Specification 5.4.1.a. "..., written procedures shall be established, implemented..." because plant procedural requirements were not included in all appropriate sections of the Unit 1 Operating Instruction, OI-15, "Service Water System." Specifically, certain procedural sections in OI -15 did not adhere to OI-15 precaution L, which prohibited the system to be in a configuration where two service water pumps could have loaded simultaneously onto a single emergency diesel generator (EDG). An engineering evaluation performed by the licensee, associated with the two pumps simultaneously loading onto an EDG, determined that this system alignment could have adversely affected the reliability of the safety-related Fairbanks Morse EDG following a loss of offsite power (LOOP) event concurrent with a loss of coolant accident (LOCA).

This finding is greater than minor because it was associated with the Mitigating System Cornerstone human performance attribute and affected the cornerstone's objective of ensuring the availability, reliability, and capability of systems that respond to initiating events. This finding did not involve the actual loss or degradation of equipment specifically designed to mitigate a seismic event or the loss of any safety function. As a result, this finding was determined to be of very low safety significance (Green) in accordance with a Phase 1 risk assessment performed in accordance with Inspection Manual Chapter - 0609, "Significance Determination Process." The inspectors identified that a contributing cause of this finding was related to the cross-cutting area of human performance. The relevant causal factor was personnel because the Unit 1 service water procedures were not appropriately changed by operations procedure writers although a procedure revision was noted in the modification package as being required. (Section 1R15)

Inspection Report# : [2005002\(pdf\)](#)

Significance:  Sep 29, 2004

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Properly Brace Erected Scaffolding (Section 1R15)

The inspectors identified a non-cited violation of Technical Specification 5.4.1.a. "..., written procedures shall be established, implemented..." because plant procedural requirements were not implemented during the construction of scaffolding erected in the vicinity of safety-related equipment. Specifically, on January 14, 2004, and again on September 14, 2004, the inspectors identified that scaffolding was constructed in close proximity to safety-related equipment without the required bracing. An engineering evaluation performed by the licensee, associated with the January 14, 2004 occurrence, determined that the scaffolding could adversely affect the safety-related 14A, 480 Vac electrical load center cooling function following a seismic event.

This finding is greater than minor because it was associated with the mitigating system cornerstone human performance attribute and affected the cornerstone's objective of ensuring the availability, reliability, and capability of systems that respond to initiating events. Additionally, this finding is consistent with a greater than minor finding as described in NRC Manual Chapter 0612, Power Reactor Inspection Report, Appendix E, Example 4.a. This finding did not involve the actual loss or degradation of equipment specifically designed to mitigate a seismic event or the loss of any safety function. As a result, this finding was determined to be of very low safety significance (Green) in accordance with a phase 1 risk assessment performed in the reactor safety significance determination process. The inspectors identified that a contributing cause of this finding was related to the cross-cutting areas of Human Performance since plant procedures were not followed properly. (Section 1R15)

Inspection Report# : [2004006\(pdf\)](#)

Significance:  Jun 30, 2004

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to implement planned, scheduled maintenance. (Section 40A2)

The inspector identified a non-cited violation for failure to implement procedures to control maintenance activities required by Technical Specification 5.4.1.a. and Regulatory Guide 1.33. The licensee failed to implement procedures to ensure that planned, scheduled maintenance was actually being performed. Maintenance personnel, by procedure, are permitted to decide whether or not to clean and lubricate 480v breakers. If the maintenance personnel decide not to perform the scheduled clean and lubricate, no method is specified or available to report this situation to maintenance and engineering management.

This finding is more than minor because it affects the Mitigating Systems Cornerstone objective to ensure the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences and affects the availability and reliability of the 480v electrical distribution system. The finding is of very low safety significance because the finding did not represent an actual loss of safety function and did not screen as potentially risk significant due to a seismic, fire, flooding or severe weather initiating event. (Section 40A2)
Inspection Report# : [2004005\(pdf\)](#)

G

Significance: Jun 30, 2004

Identified By: NRC

Item Type: NCV NonCited Violation

Inadequate corrective actions for 480v breaker testing deficiency. (Section 40A2)

The inspector identified a non-cited violation of 10 CFR 50, Appendix B, Criterion XVI. The licensee failed to promptly correct a testing deficiency identified during a CX relay failure in 1998. When action was taken in October of 2001, it was not sufficient to prevent further CX relay failures in December 2003 and February 2004.

This finding is more than minor because it affects the Mitigating Systems Cornerstone objective to ensure the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences and affects the availability and reliability of the 480 volt (v) electrical distribution system. The finding is of very low safety significance because the finding did not represent an actual loss of safety function and did not screen as potentially risk significant due to a seismic, fire, flooding or severe weather initiating event. The inspectors identified that a contributing cause of this finding was related to the cross-cutting area of Problem Identification and Resolution. (Section 40A2)

Inspection Report# : [2004005\(pdf\)](#)

G

Significance: Jun 30, 2004

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to implement design control measures for the 12 CC HX. (Section 1R15)

The inspectors identified a non-cited violation of 10 CFR 50, Appendix B, Criterion III, Design Control, associated with a self-revealing finding, which requires that measures be established to assure that design basis are correctly translated into procedures. Specifically, the licensee failed to incorporate a design flow calculation into an operating procedure which allowed the licensee to operate the 12 component cooling water heat exchanger (CC HX) in excess of its maximum shell side flow versus time curves. This failure resulted in tube failures in the only available, and in-service CC HX which supported SDC operations of the RCS.

This finding is greater than minor because it affected an attribute and the objective of the Mitigating System Cornerstone in that inadequate procedure quality resulted in degraded availability, reliability and capability of a system that responds to initiating events to prevent undesirable consequences. In accordance with Inspection Manual Chapter (IMC) 0609, Appendix G, Shutdown Operations SDP, this finding was determined to be of very low safety significance (Green) since the safety function of the component cooling water system was not lost. (Section 1R15)

Inspection Report# : [2004005\(pdf\)](#)

G

Significance: May 28, 2004

Identified By: NRC

Item Type: FIN Finding

Inadequate Breaker Coordination

The team identified a finding in that protective relay settings for the bustie circuit breakers for the 1A and OC emergency diesel generators were not adequately coordinated with the feeder breakers for the 4kV/480V service transformers supplying the 480VAC load centers.

Because the finding affected the design control attribute of the mitigating systems cornerstone, it was more than minor. Since the issue did not result in an actual loss of a safety function of a single train of equipment, the issue was determined to be of very low safety significance.

Inspection Report# : [2004003\(pdf\)](#)

G

Significance: May 28, 2004

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to provide protection in accordance with 10 CFR Part 50, Appendix R, Section III.G.2

The team identified a non-cited violation of 10 CFR Part 50, Appendix R, Section III.G.2, because Calvert Cliffs Nuclear Power Plant utilized manual actions to operate equipment necessary for achieving and maintaining hot shutdown in lieu of providing protection to the cables associated with that equipment, as required by the regulation.

In accordance with the guidance provided in inspection procedure 71111.05, "Fire Protection", (revision dated 3/6/03) this finding is greater than minor. The finding is of very low safety significance because the manual actions are reasonable and are expected to meet the criteria outlined in the Enclosure 2 of inspection procedure 71111.05. (Section 1RO5.5)

Inspection Report# : [2004003\(pdf\)](#)

Barrier Integrity

Emergency Preparedness

Occupational Radiation Safety

Public Radiation Safety

Physical Protection

[Physical Protection](#) information not publicly available.

Miscellaneous

Last modified : June 17, 2005