

# Hatch 1

## 2Q/2003 Plant Inspection Findings

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### Initiating Events

**Significance:**  Sep 28, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

#### **Failure to Meet Conditions of License for Requalification Examinations**

Green. A non-cited violation (NCV) of Technical Specification 5.2.2.f was identified for allowing the Operations Manager to continue in his normal duties without a current license in that he did not complete a requalification program within the two year training cycle as required by 10 CFR 55.53(h) and 10 CFR 55.59(a). The finding was more than minor because the Operations Manager was permitted to continue his daily duties, including directing day-to-day plant operation by licensed operators. The finding was of very low safety significance because the individual did not actually perform any licensed operator duties. Although not suitable for Significance Determination Process (SDP) analysis in the Operator Requalification or Reactor SDP, the issue was determined by management review to be of Green significance. (Section 1R11.2)

Inspection Report# : [2002004\(pdf\)](#)

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### Mitigating Systems

**Significance:**  Dec 31, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

#### **Calculation Error Results in Incorrect Steam Line High Flow Setpoints**

An incorrect calculation constant resulted in a non-conservative setpoint for the Unit 1 main steam line flow - high isolation setpoint. A self-revealing non-cited violation of Technical Specification (TS) table 3.3.6.1-1 was identified. This finding is greater than minor because the actual setpoint exceeded the TS allowable value and the analytical limit, as a result of the error. However, the violation is of very low significance because the increased steam released due to the higher setpoint would not significantly impact offsite radiological dose during a main steam line break accident.

Inspection Report# : [2002005\(pdf\)](#)

**Significance:**  Dec 31, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

#### **Inadequate Corrective Action for Missing Penetration Seals**

The licensee had not taken prompt corrective action for missing RHR Service Water (RHRSW) piping penetration seals at the intake structure. A non-cited violation of 10CFR50 Appendix B, Criterion XVI was identified. This finding is more than minor because the lack of penetration seals could have permitted the Plant Service Water (PSW) valve pit to flood and effected the mitigating systems cornerstone. Because flooding of the PSW valve pit had not occurred nor

were flooding conditions present, this failure to promptly correct a condition adverse to quality is of very low safety significance.

Inspection Report# : [2002005\(pdf\)](#)



**Significance:** Dec 31, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

### **Inadequate Operability Assessment of Main Steam Safety Relief Valve**

The licensee did not promptly identify the cause of a failed safety relief valve (SRV). An operability evaluation written in response to the failure was not timely and did not adequately support a determination that the remaining SRV's were operable. Consequently, this significant condition adverse to quality was not promptly corrected and adequate measures were not taken to preclude repetition. A non-cited violation of 10CFR50 Appendix B, Criterion XVI was identified. This finding is greater than minor because the licensee's operability assessment was not timely and relied primarily on unsupported engineering judgement for a determination of operable for the remaining SRV's. It also required multiple revisions when inconsistencies were identified by the inspectors. This finding was of very low significance because no loss of SRV function occurred.

Inspection Report# : [2002005\(pdf\)](#)

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## **Barrier Integrity**

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## **Emergency Preparedness**

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## **Occupational Radiation Safety**

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## **Public Radiation Safety**

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## **Physical Protection**

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## **Miscellaneous**

Last modified : September 04, 2003