How Can You Participate?

You can participate through written electronic submissions to the NNCO at http://www.nano.gov/html/society/ ehs_priorities. Submissions are welcome from all members of the public.

How Will Public Input Be Used?

All comments and recommendations that are submitted will be considered by the NEHI Working Group. Input from multiple stakeholders with various interests will be valuable to the NNI. Through activities such as this solicitation, the NSET Subcommittee and NNI member agencies are making the priority-setting process dynamic, open, and transparent.

E. Clayton Teague,

Director, National Nanotechnology Coordination Office, Nanoscale Science, Engineering, and Technology Subcommittee of the National Science and Technology Council Committee on Technology.

[FR Doc. E7-16077 Filed 8-15-07; 8:45 am] BILLING CODE 3170-WF-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. STN 50-454 and STN 50-455]

Exelon Generation Company, LLC; Notice of Withdrawal of Application for **Amendment to Facility Operating** Licenses NPF-37 and NPF-66

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Exelon Generation Company, LLC (the licensee) to withdraw its June 16, 2006, application for proposed amendment to Facility Operating License Nos. NPF-37 and NPF-66 for the Byron Station, Unit Nos. 1 and 2, located in Ogle County, Illinois.

The proposed amendment would have revised the Updated Final Safety Analysis Report pertaining to tornado generated missile protection for certain systems and components.

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the Federal Register on November 21, 2006 (17 FR 67393). However, by letter dated July 24, 2007, the licensee withdrew the proposed change.

For further details with respect to this action, see the application for amendment dated June 16, 2006, and the licensee's letter dated July 24, 2007, which withdrew the application for license amendment. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, Public

File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, http:// www.nrc.gov/reading-rm.html. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff by telephone at 1-800-397-4209, or 301-415-4737 or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 6th day of August 2007.

For the Nuclear Regulatory Commission. Robert F. Kuntz,

Project Manager, Plant Licensing Branch III-2, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. E7-16148 Filed 8-15-07; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Consolidated Decommissioning Guidance; Notice of Revision to, Withdrawal of Portions of, and Process for Updating

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of revision to, withdrawal of portions of, and process for updating NUREG-1757.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) staff is revising, withdrawing portions of, and describing the process for updating guidance in "Consolidated Decommissioning Guidance: Characterization, Survey, and Determination of Radiological Criteria' (NUREG-1757, Vol. 2, Rev. 1), Appendix N, "ALARA Analyses." This notice also describes the staff's process for developing interim guidance and future revisions to the three volumes of its "Consolidated Decommissioning Guidance" (NUREG-1757).

ADDRESSES: NUREG-1757 is available for inspection and copying for a fee at the Commission's Public Document Room, NRC's Headquarters Building, 11555 Rockville Pike (First Floor), Rockville, Maryland. The Public Document Room is open from 7:45 a.m. to 4:15 p.m., Monday through Friday, except on Federal holidays. NUREG-1757 is also available electronically on the NRC Web site at: http:// www.nrc.gov/reading-rm/doccollections/nuregs/staff/sr1757/, and from the ADAMS Electronic Reading

Room on the NRC Web site at: http:// www.nrc.gov/reading-rm/adams.html.

The NRC's decommissioning Web page is at: http://www.nrc.gov/aboutnrc/regulatory/decommissioning.html.

FOR FURTHER INFORMATION CONTACT:

Duane W. Schmidt, Division of Waste Management and Environmental Protection, Office of Federal and State Materials and Environmental Management Programs, Mail Stop T-8F5, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001. Telephone: (301) 415-6919; e-mail: dws2@nrc.gov.

SUPPLEMENTARY INFORMATION: In September 2006, the NRC staff published Revision 1 of Volume 2 of NUREG-1757, entitled, "Consolidated Decommissioning Guidance: Characterization, Survey, and Determination of Radiological Criteria," which provides technical guidance on compliance with the radiological criteria for license termination in the NRC's License Termination Rule (LTR) (Code of Federal Regulations, Title 10, Part 20, Subpart E). Volume 2 is applicable to all licensees subject to the LTR. Volume 2 is one of three volumes of the NUREG-1757 series, which, combined, provide consolidated guidance on decommissioning.

The NRC staff considers the development of its guidance as an iterative process. Formal revisions to of NUREG-1757 (i.e., publishing new revised volumes of NUREG-1757) are anticipated in the future. When these revised volumes are developed, the NRC staff intends to publish them as drafts for public comment. Between formal revisions of the NUREG-1757 volumes, errors needing correction or other revisions may be identified and the NRC staff may develop interim guidance and post it on the NRC's decommissioning web page, to make it available for use by licensees and other stakeholders.

During the review of a recently submitted decommissioning plan, proposing decommissioning in accordance with the restricted use provision of the LTR, the NRC staff determined that there are certain errors in Vol. 2 of NUREG-1757, Appendix N. The specific errors concern compliance with the "as low as is reasonably achievable" (ALARA) provisions of the

The guidance being corrected or withdrawn is described below.

Error 1. On page N-1 of Appendix N, the first paragraph provides a general introductory discussion of ALARA. In this paragraph, the word "feasible" is used twice when referring to ALARA. The correct word is "reasonable."

Error 2. On page N-4 of Appendix N, The last paragraph discusses the monetary value for collective dose averted and discount rates that may be used in ALARA calculations. In particular, the paragraph includes the following two sentences: "For doses averted within the first 100 years, a discount rate of 7% should be used. For doses averted beyond 100 years, a 3% discount rate should be used. "The discussion of discount rate in these two sentences is incorrect. Therefore, these two quoted sentences are withdrawn from the guidance of NUREG-1757, Vol. 2 and should not be used.

Error 3. On page N–10 of Appendix N, Table N.2 summarizes acceptable parameter values for use in decommissioning ALARA analyses. This table includes a row describing the monetary discount rate, r. Consistent with Error 2, above, the description for the second column (the "value" description) of the row on monetary discount rate, r, is withdrawn from the guidance of NUREG–1757, Vol. 2.

Error 4. On page N–12 of Appendix N, Example 3 is an ALARA calculation for removing surface soil contaminated with a long-lived radionuclide. Use of the single discount rate in the example may be misleading, because the guidance in NUREG/BR–0058 recommends multiple analyses be performed. Therefore, Example 3 is withdrawn from Appendix N of NUREG–1757, Vol. 2, and should not be used.

Error 5. On page N–18 of Appendix N, the last paragraph again discusses acceptable values for the discount rate, r. In particular, this paragraph includes the sentence: "Values for r are given in NUREG/BR–0058, Revision 2, and OMB policy (OMB 1996)." The referenced guidance is out-of-date, and this quoted sentence is withdrawn from the guidance of NUREG–1757, Vol. 2.

The staff intends to develop interim guidance to address the withdrawn portions of guidance discussed above and will post the interim guidance on the NRC's decommissioning Web page, to make it available for use by licensees and other stakeholders.

The guidance in NUREG—1757 and any corrections to NUREG—1757 are intended for use by NRC staff and licensees. The NUREG and any corrections are not substitutes for NRC regulations, and compliance with them is not required. The NUREG and corrections describe approaches that are generally acceptable to NRC staff. However, methods and solutions different than those in the NUREG and corrections will be acceptable, if they provide a basis for concluding that the

decommissioning actions are in compliance with NRC regulations.

Dated at Rockville, MD, this 10th day of August, 2007.

For the Nuclear Regulatory Commission.

Keith I. McConnell,

Deputy Director, Decommissioning & Uranium Recovery, Licensing Directorate, Division of Waste Management and Environmental Protection, Office of Federal and State Materials and Environmental Management Programs.

[FR Doc. E7–16131 Filed 8–15–07; 8:45 am] BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Notice of Opportunity To Comment on Model Safety Evaluation on Technical Specification Improvement To Revise Control Rod Notch Surveillance Frequency, Clarify SRM Insert Control Rod Action, and Clarify Frequency Example

AGENCY: Nuclear Regulatory Commission.

ACTION: Request for comment.

SUMMARY: Notice is hereby given that the staff of the Nuclear Regulatory Commission (NRC) has prepared a model safety evaluation (SE) relating to the revision of Standard Technical Specifications (STS), NUREG-1433 (BWR/4) and NUREG-1434 (BWR/6). Specifically the SE addresses: (1) The revision of the TS surveillance requirement (SR) 3.1.3.2 frequency in STS 3.1.3, "Control Rod OPERABILITY," (2) a clarification to the requirement to fully insert all insertable control rods for the limiting condition for operation (LCO) in STS 3.3.1.2, Required Action E.2, "Source Range Monitor Instrumentation" (NUREG-1434 only), and (3) the revision of Example 1.4–3 in STS Section 1.4 "Frequency" to clarify the applicability of the 1.25 surveillance test interval extension. The NRC staff has also prepared a model license amendment request and a model no significant hazards consideration (NSHC) determination relating to this matter. The purpose of these models are to permit the NRC to efficiently process amendments that propose to modify TS control rod SR testing frequency, clarify TS control insertion requirements, and clarify SR frequency discussions. Licensees of nuclear power reactors to which the models apply could then request amendments, confirming the applicability of the SE and NSHC determination to their plant licensing basis. The NRC staff is requesting

comment on the model SE, model amendment request, and model NSHC determination prior to announcing their availability for referencing in license amendment applications.

DATES: The comment period expires September 17, 2007. Comments received after this date will be considered if it is practical to do so, but the Commission is able to ensure consideration only for comments received on or before this date.

ADDRESSES: Comments may be submitted either electronically or via U.S. mail. Submit written comments to Chief, Rulemaking, Directives, and Editing Branch, Division of Administrative Services, Office of Administration, Mail Stop: T-6 D59, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001. Hand deliver comments to: 11545 Rockville Pike, Rockville, Maryland, between 7:45 a.m. and 4:15 p.m. on Federal workdays. Copies of comments received may be examined at the NRC's Public Document Room, 11555 Rockville Pike (Room O-1F21), Rockville, Maryland. Comments may be submitted by electronic mail to CLIIP@nrc.gov.

FOR FURTHER INFORMATION CONTACT:

Timothy Kobetz, Mail Stop: O–12H2, Technical Specifications Branch, Division of Inspection & Regional Support, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, telephone 301–415–1932.

SUPPLEMENTARY INFORMATION:

Background

Regulatory Issue Summary 2000–06, "Consolidated Line Item Improvement Process for Adopting Standard **Technical Specification Changes for** Power Reactors," was issued on March 20, 2000. The consolidated line item improvement process (CLIIP) is intended to improve the efficiency of NRC licensing processes, by processing proposed changes to the STS in a manner that supports subsequent license amendment applications. The CLIIP includes an opportunity for the public to comment on proposed changes to the STS after a preliminary assessment by the NRC staff and finding that the change will likely be offered for adoption by licensees. This notice solicits comment on a proposed change to the STS that modifies a TS control rod SR testing frequency, clarifies TS control rod insertion requirements, and clarifies SR frequency discussions. The CLIIP directs the NRC staff to evaluate any comments received for a proposed change to the STS and to either reconsider the change or announce the