Docket Office for information about materials not available through the OSHA Web page and for assistance using the Web page to locate docket submissions.

Electronic copies of this **Federal Register** notice as well as other relevant documents are available on OSHA's Web page.

II. Background

The Department of Labor, as part of its continuing effort to reduce paperwork and respondent (*i.e.,* employer) burden, conducts a preclearance consultation program to provide the public with an opportunity to comment on proposed and continuing information collection requirements in accordance with the Paperwork Reduction Act of 1995 (PRA–95) (44 U.S.C. 3506(c)(2)(A)).

This program ensures that information is in the desired format, reporting burden (time and costs) is minimal, collection instruments are clearly understood, and OSHA's estimate of the information collection burden is accurate. The Occupational Safety and Health Act of 1970 (the Act) (29 U.S.C. 651 *et seq.*) authorizes information collection by employers as necessary or appropriate for enforcement of the Act or for developing information regarding the causes and prevention of occupational injuries, illnesses, and accidents (29 U.S.C. 657).

Paragraph (g)(4)(ii)(I) of the Cranes and Derricks Standard for Construction (Sec. 1926.550) requires employers to post conspicuously with a plate or other permanent marking the weight and rated load capacity or maximum intended loads of each platform used to raise and lower employees to a worksite using a crane or derrick. This requirement helps employers to avoid exceeding the lifting capacity of such platforms and the cranes or derrick being used to lift the platforms. Therefore, this requirement can prevent the platform, crane, or derrick from collapsing and causing serious injury to death to employees on or below the platform.

III. Special Issues for Comment

OSHA has a particular interest in comments on the following issues:

- Whether the proposed information collection requirements are necessary for the proper performance of the Agency's functions, including whether the information is useful;
- The accuracy of OSHA's estimate of the burden (time and costs) of the information collection requirements, including the validity of the methodology and assumptions used;
- The quality, utility, and clarity of the information collected; and
- Ways to minimize the burden on employers who must comply; for example, by using automated or other technological information collection and transmission techniques.

IV. Proposed Actions

OSHA is proposing to extend the information collection requirement specified by paragraph (g)(4)(ii)(I) of Sec. 1926.550. The Agency will summarize the comments submitted in response to this notice and will include this summary in its request to OMB to extend the approval of this information collection requirements contained in the Standard.

Type of Review: Extension of currently approved information collection requirements.

Title: Cranes and Derricks Standard for Construction; Posting Weight and Load Capacity of Personnel Platforms (29 CFR 1926.550).

OMB Number: 1218–0151.

Affected Public: Business or other for-profit; not-for-profit institutions; Federal government; State, local, or Tribal governments.

Number of Respondents: 2,750 (platforms).

Frequency of Response: On occasion.

Total Responses: 2,750.

Average Time per Response: Five minutes to post or mark a platform.

Estimated Total Burden Hours: 229.

Estimated Cost. (Operation and Maintenance): $0.

V. Authority and Signature

John L. Henshaw, Assistant Secretary of Labor for Occupational Safety and Health, directed the preparation of this notice. The authority for this notice is the Paperwork Reduction Act of 1995 (44 U.S.C. 3506 *et seq.*), and Secretary of Labor's Order No. 5–2002 (67 FR 65008).

Signed at Washington, DC, on October 4, 2004.

John L. Henshaw,

Assistant Secretary of Labor.

[FR Doc. 04–22600 Filed 10–6–04; 8:45 am]

BILLING CODE 4510–26–M

NUCLEAR REGULATORY COMMISSION

Draft Regulatory Guide; Issuance, Availability

The U.S. Nuclear Regulatory Commission (NRC) has issued for public comment a draft of a new guide in the agency's Regulatory Guide Series. This series has been developed to describe and make available to the public such information as methods that are acceptable to the NRC staff for implementing specific parts of the NRC's regulations, techniques that the staff uses in evaluating specific problems or postulated accidents, and data that the staff needs in its review of applications for permits and licenses.

The new draft regulatory guide, entitled ''Risk-Informed, Performance-Based Fire Protection for Existing Light-Water Nuclear Power Plants,'' is temporarily identified by its task number, DG–1139, which should be mentioned in all related correspondence. The draft regulatory guide contains the staff's regulatory position on ''Guidance for Implementing a Risk-Informed, Performance-Based Fire Protection Program Under 10 CFR 50.48(c),'' which the Nuclear Energy Institute (NEI) has promulgated as document #NEI 04–02.

It is the staff's intent to endorse a version of NEI 04–02, as appropriate, in the final regulatory guide, consistent with the new risk-informed, performance-based fire protection rule, specified in Title 10, Section 50.48(c), of the *Code of Federal Regulations* [10 CFR 50.48(c)], which the NRC has issued for existing light-water nuclear power plants. This new regulation provides a voluntary alternative to the requirements of Appendix R to 10 CFR part 50, ''Fire Protection Program for Nuclear Power Facilities Operating Prior to January 1, 1979.'' As such, the new rule endorses a national consensus standard (NEI 04–02), sets performance goals and criteria, and takes advantage of experience and enhanced methodologies.

The NRC staff is soliciting comments on draft regulatory guide DG–1139, and comments may be accompanied by relevant information or supporting data. Please mention DG–1139 [50.48(c) RG] in the subject line of your comments. Comments on this draft regulatory guide submitted in writing or in electronic form will be made available to the public in their entirety on the NRC's rulemaking Web site. Personal information will not be removed from your comments. You may submit comments by any of the following methods.

Mail comments to: Rules and Directives Branch, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001.

Email comments to: NRCREP@nrc.gov. You may also submit comments via the NRC's rulemaking Web site at *http://ruleforum.llnl.gov.* Address questions about our rulemaking

Web site to Carol A. Gallagher (301) 415–5905; email *CAG@nrc.gov*.

Hand-deliver comments to: Rules and Directives Branch, Office of Administration, U.S. Nuclear Regulatory Commission, 11555 Rockville Pike, Rockville, Maryland 20852, between 7:30 a.m. and 4:15 p.m. on Federal workdays.

Fax comments to: Rules and Directives Branch, Office of Administration, U.S. Nuclear Regulatory Commission at (301) 415–5144.

Request for information about draft regulatory guide DG–1139 may be directed to Paul W. Lain at (301) 415–2346 or via email to *PWL@nrc.gov*.

Comments would be most helpful if received by December 15, 2004. Comments received after this date will be considered if it is practical to do so, but the NRC is able to ensure consideration only for comments received on or before this date. Although a time limit is given, comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time.

Electronic copies of the draft regulatory guide are available through the NRC's public Web site under Draft Regulatory Guides in the Regulatory Guides document collection of the NRC's Electronic Reading Room at *http://www.nrc.gov/reading-rm/doc-collections/*. Electronic copies are also available in the NRC's Agencywide Documents Access and Management System (ADAMS) at *http://www.nrc.gov/reading-rm/adams.html*, under Accession No. ML042740308. In addition, regulatory guides are available for inspection at the NRC's Public Document Room (PDR), which is located at 11555 Rockville Pike, Rockville, Maryland; the PDR's mailing address is USNRC PDR, Washington, DC 20555–0001. The PDR can also be reached by telephone at (301) 415–4737 or (800) 397–4205, by fax at (301) 415–3548; and by email to *PDR@nrc.gov*. Requests for single copies of draft or final guides (which may be reproduced) or for placement on an automatic distribution list for single copies of future draft guides in specific divisions should be made in writing to the U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, Attention: Reproduction and Distribution Services Section; by email to *DISTRIBUTION@nrc.gov*; or by fax to (301) 415–2289. Telephone requests cannot be accommodated. Regulatory guides are not copyrighted, and Commission approval is not required to reproduce them. (U.S.C. 552(a))

Dated at Rockville, Maryland, this 30th day of September , 2004.

For the U.S. Nuclear Regulatory Commission.

Sharon D. Stewart,

Acting Director, Program Management, Policy Development and Analysis Staff, Office of Nuclear Regulatory Research.

[FR Doc. 04–22545 Filed 10–6–04; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Proposed Generic Communication; Steam Generator Tube Integrity and Associated Technical Specifications

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of opportunity for public comment.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is proposing to issue a generic letter (GL) to request that addressees submit a description of their program for ensuring steam generator (SG) tube integrity for the interval between inspections and description of the methodology used to assess the effects of non-pressure-related loads such as bending on SG tube integrity. Addressees should also provide a safety assessment demonstrating that the SG tubes will have adequate structural and leakage integrity (with appropriate regulatory margins) at the time of their next SG tube inspection, taking into account the effects of non-pressure-related loads.

This **Federal Register** notice is available through the NRC's Agencywide Documents Access and Management System (ADAMS) under accession number ML042710075.

DATES: Comment period expires December 6, 2004. Comments submitted after this date will be considered if it is practical to do so, but assurance of consideration cannot be given except for comments received on or before this date.

ADDRESSEES: Submit written comments to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Mail Stop T6-D59, Washington, DC 20555–0001, and cite the publication date and page number of this **Federal Register** notice. Written comments may also be delivered to NRC Headquarters, 11545 Rockville Pike (Room T–6D59), Rockville, Maryland, between 7:30 am and 4:15 pm on Federal workdays.

FOR FURTHER INFORMATION CONTACT: Kenneth Karwoski, NRR at 301–415–2752 or by e-mail at *kjk1@nrc.gov* or Maitri Banerjee, NRR at 301–415–2277 or by e-mail at *mxb@nrc.gov.*

SUPPLEMENTARY INFORMATION:

Draft NRC Generic Letter 2004–XX: Steam Generator Tube Integrity and Associated Technical Specifications

Addressees

All holders of operating licenses for pressurized-water reactors (PWRs), except those who have permanently ceased operations and have certified that fuel has been permanently removed from the reactor vessel and {the following plants that have already modified their technical specifications to be consistent with those in the Attachment}.

Purpose

The U.S. Nuclear Regulatory Commission (NRC) is issuing this generic letter (GL) to:

(1) Request that addressees submit a description of their program for ensuring steam generator (SG) tube integrity for the interval between inspections; and

(2) Request that addressees submit a description of the methodology used to assess the effects of non-pressure-related loads such as bending on SG tube integrity. Addressees should also provide a safety assessment demonstrating that the SG tubes will have adequate structural and leakage integrity (with appropriate regulatory margins) at the time of their next SG tube inspection, taking into account the effects of non-pressure-related loads.

Discussion

Steam generator tubes function as an integral part of the reactor coolant pressure boundary (RCPB) and also serve to isolate radiological fission products in the primary coolant from the secondary coolant and the environment. For the purposes of this generic letter, tube integrity means that the tubes are capable of performing these functions in accordance with the plant design and licensing basis, including applicable regulatory requirements.

During operation, licensees are required to monitor and maintain the condition of the SG tubing with the objective of ensuring its continued integrity. Specifically, licensees are required by 10 CFR 50.55a(b)(2)(iii), 10 CFR 50.55a(g), or by the plant technical specifications to perform periodic inservice inspections and to repair (*e.g.*, sleeve) or remove from service (by installing plugs in the tube ends) all tubes found to contain flaws exceeding