at the NRC Public Document Room, One White Flint North, 11555 Rockville Pike, Room O–1 F21, Rockville, MD 20852. OMB clearance requests are available at the NRC worldwide Web site: http://www.nrc.gov/public-involve/ doc-comment/omb/index.html. The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions should be directed to the OMB reviewer listed below by August 7, 2006. Comments received after this date will be considered if it is practical to do so, but assurance of consideration cannot be given to comments received after this date. John A. Asalone, Office of Information and Regulatory Affairs (3150–0031), NEOB–10202, Office of Management and Budget, Washington, DC 20503.

Comments can also be e-mailed to *John_A._Asalone@omb.eop.gov* or submitted by telephone at (202) 395–4650.

The NRC Clearance Officer is Brenda Jo. Shelton, 301–415–7233.

Dated at Rockville, Maryland, this 29th day of June, 2006.

For the Nuclear Regulatory Commission. Brenda Jo. Shelton,

NRC Clearance Officer, Office of Information Services.

[FR Doc. E6–10523 Filed 7–5–06; 8:45 am] BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 72–7 and 50–255; License No. DPR–20]

Nuclear Management Company, LLC; Consideration of Request for Action Under 10 CFR 2.206

AGENCY: Nuclear Regulatory Commission.

ACTION: Receipt and consideration of request for action under 10 CFR 2.206.

FOR FURTHER INFORMATION CONTACT: L.

Raynard Wharton, Senior Project Manager, Spent Fuel Project Office, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Telephone: (301) 415–1396; Fax number: (301) 415–8555: E-mail: *Irw@nrc.gov.*

Introduction

Notice is hereby given that by petition dated April 4, 2006, Mr. Terry J. Lodge (Counsel for Petitioners) has requested that the Nuclear Regulatory Commission (NRC) take action with regard to the Nuclear Management Company, LLC (NMC) Palisades Nuclear Plant (PNP). The petitioners' request that the NRC take enforcement action against PNP by condemning and stopping the use of the two independent spent fuel storage installation (ISFSI) concrete pads, constructed in 1992 and 2003, which hold dry spent fuel storage casks at the plant site.

Request

As the basis for the request, the petitioners state that both ISFSI concrete pads at PNP do not conform to NRC requirements for earthquake stability standards and pose a distinct hazard in the event of an earthquake.

The request concerning slope stability of the 2003 concrete pad is being treated pursuant to 10 CFR 2.206 of the Commission's regulations. The request has been referred to the Director of the Spent Fuel Project Office within the Office of Nuclear Material Safety and Safeguards. As provided by 10 CFR 2.206, appropriate action will be taken on this petition within a reasonable time. Representatives of Mr. Lodge spoke with the Petition Review Board on April 26, 2006, to discuss the petition. The results of that discussion were considered in the Board's determination regarding condemning and stopping the use of the two ISFSI concrete pads and in establishing a schedule for the review of the petition. By letter dated June 27, 2006, the Spent Fuel Project Office Deputy Director accepted the petition for review in part, specifically with respect to slope stability of the concrete pad constructed in 2003.

Further Information

A copy of the petition may be inspected at NRC's Public Electronic Reading Room at http://www.nrc.gov/ reading-rm/adams.html. This document may also be viewed electronically on the public computers located at the NRC's Public Document Room (PDR), O-1F21, One White Flint North, 11555 Rockville Pike, Rockville, MD 20852. The PDR reproduction contractor will copy documents for a fee. Persons who do not have access to the NRC's Agencywide Documents Access and Management System (ADAMS) or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1-800-397-4209 or (301) 415–4737, or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland this 27th day of June, 2006.

For the Nuclear Regulatory Commission. L. Raynard Wharton,

Senior Project Manager, Spent Fuel Project Office, Office of Nuclear Material Safety and Safeguards.

[FR Doc. E6–10525 Filed 7–5–06; 8:45 am] BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-361 and 50-362]

Southern California Edison Company, San Diego Gas and Electric Company, the City of Riverside, CA, the City of Anaheim, CA; San Onofre Nuclear Generating Station, Units 2 and 3; Exemption

1.0 Background

Southern California Edison Company (the licensee) is the holder of Facility Operating License Nos. NPF–10 and NPF–15, which authorize operation of the San Onofre Nuclear Generating Station, Unit 2 and Unit 3 (SONGS 2 and 3), respectively. The licenses provide, among other things, that the facility is subject to all rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (NRC, the Commission) now or hereafter in effect.

The facility consists of two pressurized-water reactors located in San Diego County, California.

2.0 Request/action

Title 10 of the Code of Federal Regulations (10 CFR), Part 50, Appendix G, which is invoked by 10 CFR 50.60, requires that pressure-temperature (P-T) limits be established for reactor pressure vessels (RPVs) during normal operating and hydrostatic or leak rate testing conditions. Specifically, 10 CFR Part 50, Appendix G, states that "[t]he appropriate requirements on both the pressure-temperature limits and the minimum permissible temperature must be met for all conditions," and "[t]he pressure-temperature limits identified as 'ASME [American Society for Mechanical Engineers] Appendix G limits' in Table 3 require that the limits must be at least as conservative as limits obtained by following the methods of analysis and the margins of safety of Appendix G of Section XI of the ASME Code [Boiler and Pressure Vessel Code]." Part 50 of Title 10 of the Code of Federal Regulations, Appendix G, also specifies that the editions and addenda of the ASME Code, Section XI, which are incorporated by reference in 10 CFR 50.55a, apply to the requirements in 10 CFR Part 50,