Washington, DC 20555–0001, by telephone at (301) 415–7233, or by Internet electronic mail at *infocollects@nrc.gov*.

Dated at Rockville, Maryland, this 14th day of March 2005.

For the Nuclear Regulatory Commission.

Brenda Jo. Shelton,

NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 05–5680 Filed 3–22–05; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 72–20]

Department of Energy; Three Mile Island 2 Independent Spent Fuel Storage Installation; Notice of Docketing of Materials License SNM–2508 Amendment Application

AGENCY: Nuclear Regulatory Commission.

ACTION: License Amendment.

FOR FURTHER INFORMATION CONTACT: Joseph M. Sebrosky, Senior Project Manager, Spent Fuel Project Office, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Telephone: (301) 415–1132; fax number: (301) 415–1179; e-mail: *jms3@nrc.gov.*

SUPPLEMENTARY INFORMATION: By letter dated January 31, 2005, the Department of Energy (DOE or licensee) submitted an application to the U.S. Nuclear Regulatory Commission (NRC or the Commission), in accordance with Title 10 of the Code of Federal Regulations (10 CFR) 72.56, requesting the amendment of the Three Mile Island 2 (TMI–2) Independent Spent Fuel Storage Installation (ISFSI) license for the ISFSI located in Butte County, Idaho. DOE proposes to change the technical specification corrective actions if the 5 year leak test of the dry shielded canisters fails.

This application was docketed under 10 CFR part 72; the ISFSI Docket No. is 72–20 and will remain the same for this action. Upon approval of the Commission, the TMI–2 ISFSI license, SNM–2508, would be amended to allow this action.

The Commission may issue either a notice of hearing or a notice of proposed action and opportunity for hearing in accordance with 10 CFR 72.46(b)(1) regarding the proposed amendment or, if a determination is made that the proposed amendment does not present a genuine issue as to whether public health and safety will be significantly affected, take immediate action on the proposed amendment in accordance with 10 CFR 72.46(b)(2) and provide notice of the action taken and an opportunity for interested persons to request a hearing on whether the action should be rescinded or modified.

For further details with respect to this amendment, see the application dated January 31, 2005, which is publically available in the records component of NRC's Agencywide Documents Access and Management System (ADAMS). The NRC maintains ADAMS, which provides text and image files of NRC's public documents. These documents may be accessed through the NRC's Public Electronic Reading Room on the Internet at *http://www.nrc.gov/reading-rm/adams.html.* If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1–800–397–4209, (301) 415–4737 or by email to *pdr@nrc.gov.*

Dated at Rockville, Maryland, this 15th day of March 2005.

For the Nuclear Regulatory Commission.

John D. Monninger,

Chief, Licensing Section, Spent Fuel Project Office, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 05–5681 Filed 3–22–05; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–271]

Entergy Nuclear Vermont Yankee, LLC and Entergy Nuclear Operations, Inc. Vermont Yankee Nuclear Power Station; Exemption

1.0 Background

Entergy Nuclear Vermont Yankee, LLC and Entergy Nuclear Operations, Inc. (Entergy or the licensee) are the holders of Facility Operating License No. DPR–28 which authorizes operation of the Vermont Yankee Nuclear Power Station (VYNPS). The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the Nuclear Regulatory Commission (NRC, the Commission) now or hereafter in effect.

The facility consists of a boiling-water reactor located in Vernon, Vermont.

2.0 Request/Action

Title 10 of the Code of Federal Regulations (10 CFR), section 50.54(o), requires primary reactor containments for water-cooled power reactors to be subject to the requirements of Appendix J to 10 CFR part 50. Appendix J specifies the leakage test requirements, schedules, and acceptance criteria for tests of the leak-tight integrity of the primary reactor containment and systems and components which penetrate the containment. Option B of Appendix J is titled ''Performance-Based Requirements.'' Option B, section III.A., ''Type A Test,'' requires that the overall integrated leakage rate must not exceed the allowable leakage rate (La) with margin, as specified in the Technical Specifications (TSs). The overall integrated leakage rate, as specified in the 10 CFR part 50, Appendix J, Option B, definitions, means the total leakage rate through all tested leakage paths. The licensee is requesting a permanent exemption from Option B, section III.A., requirements to permit exclusion of the main steam pathway leakage contributions from the overall integrated leakage rate Type A test measurement. Main steam leakage includes leakage through all four main steam lines and the main steam drain line.

Option B, Section III.B of 10 CFR part 50, Appendix J, ''Type B and C Tests,'' requires that the sum of the leakage rates of all Type B and Type C local leak rate tests be less than the performance criterion (La) with margin, as specified in the TSs. The licensee also requests exemption from this requirement, to permit exclusion of the main steam pathway leakage contributions from the sum of the leakage rates from Type B and Type C tests.

The main steam leakage effluent has a different pathway to the environment, when compared to a typical containment penetration. It is not directed into the secondary containment and filtered through the standby gas treatment system as is other containment leakage. Instead, the main steam leakage is collected and treated via an alternative leakage treatment (ALT) path having different mitigation characteristics.

In performing accident analyses, it is appropriate to group various leakage effluents according to the treatment they receive before being released to the environment (*e.g.*, from main steam pathways). The proposed exemption would more appropriately permit ALT pathway leakage to be independently grouped with its unique leakage limits. In this manner, the VYNPS containment leakage testing program will be made more consistent with the limiting assumptions used in the associated accident consequence analyses.

The licensee has analyzed the main steam leakage pathway (with an increase in leakage from 62 standard cubic feet per hour (scfh) to 124 scfh at