February 1, 2001

Mr. M. Reddemann
Site Vice President
Kewaunee and Point Beach Nuclear Plants
Wisconsin Electric Power Company
6610 Nuclear Road
Two Rivers, WI 54241

SUBJECT: POINT BEACH - NRC INSPECTION REPORT 50-266/01-02(DRS); 50-301/01-02(DRS)

Dear Mr. Reddemann:

On January 12, 2001, the NRC completed a routine inspection at the Point Beach Nuclear Plant. The enclosed report presents the results of that inspection. The results were discussed on January 12, 2001, with Mr. Cayia and other members of your staff.

The inspection was an examination of activities conducted under your license as they relate to radiation safety and to compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection consisted of a selective examination of representative records, tours of your facility, and interviews with personnel. Specifically, this inspection focused on aspects of Public Radiation Safety.

No findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

We will gladly discuss any questions you have concerning this inspection.

Sincerely,

/RA/

Gary L. Shear, Chief
Plant Support Branch
Division of Reactor Safety

Docket Nos. 50-266; 50-301
License Nos. DPR-24; DPR-27

Enclosure: Inspection Report 50-266/01-02(DRS);
50-301/01-02(DRS)

cc w/encl: R. Grigg, President and Chief
Operating Officer, WEPCo
M. Wadley, Chief Nuclear Officer, NMC
J. Gadzala, Licensing Manager
D. Weaver, Nuclear Asset Manager
F. Cayia, Plant Manager
J. O'Neill, Jr., Shaw, Pittman,
Potts & Trowbridge
K. Duveneck, Town Chairman
Town of Two Creeks
D. Graham, Director
Bureau of Field Operations
A. Bie, Chairperson, Wisconsin
Public Service Commission
S. Jenkins, Electric Division
Wisconsin Public Service Commission
State Liaison Officer

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos: 50-266; 50-301
License Nos: DPR-24; DPR-27

Report No: 50-266/01-02(DRS); 50-301/01-02(DRS)

Licensee: Wisconsin Electric Power Company

Facility: Point Beach Nuclear Plant, Units 1 and 2

Location: 6610 Nuclear Road
Two Rivers, WI 54241

Dates: January 8-12, 2001

Inspector: D. Nelson, Radiation Specialist

Approved by: Gary L. Shear, Chief
Plant Support Branch
Division of Reactor Safety

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety	Radiation Safety	Safeguards
● Initiating Events	● Occupational	● Physical Protection
● Mitigating Systems	● Public	
● Barrier Integrity		
● Emergency Preparedness		

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: http://www.nrc.gov/NRR/OVERSIGHT/index.html.

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4 SUMMARY OF FINDINGS

IR 05000266-01-02(DRS), 05000301-01-02(DRS), on 01/08-01/12/2001, Wisconsin Electric Power Company, Point Beach Nuclear Plant, Units 1 and 2. Radiation Specialist report.

The inspection was conducted by a regional radiation specialist. No findings of significance were identified.

Report Details

Summary of Plant Status: Unit 1 and Unit 2 were at full power during the inspection period.

2. RADIATION SAFETY

Cornerstone: Occupational Radiation Safety

2OS1 Access Control

.1 Plant Walkdowns and Radiological Boundary Verifications

a. Inspection Scope

The inspector conducted walkdowns of the radiologically controlled area (RCA) to verify the adequacy of radiological boundaries and postings. Specifically, the inspector walked down several radiologically significant work area boundaries (high and locked high radiation areas) in the Unit 1 and 2 Primary Auxiliary Building.

b. Findings

No findings of significance were identified.

Cornerstone: Public Radiation Safety

2PS2 Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems

.1 Offsite Dose Calculation Manual (ODCM)

a. Inspection Scope

The inspector reviewed the 1999 Annual Monitoring Report to verify that the effluent program was implemented as described in the Final Safety Analysis Report (FSAR), the Offsite Dose Calculation Manual (ODCM) and the Radiological Effluent Control Manual (RECM). The inspector reviewed the report for significant changes to the ODCM and to the design and operation of the radioactive waste system.

a. Findings

No findings of significance were identified.

.2 Gaseous and Liquid Release Systems Walkdowns

a. Inspection Scope

The inspector performed walkdowns of the major components of the gaseous and liquid release systems (e.g., radiation and flow monitors, demineralizers and filters, tanks, and vessels) to verify that the current system configuration was as described in the FSAR,

the ODCM and the RECM, and to observe ongoing activities and equipment material condition.

b. Findings

No findings of significance were identified.

.3 Gaseous and Liquid Release Permits

a. Inspection Scope

Since there were no gaseous or liquid releases during the inspection period, the inspector reviewed several radioactive gaseous and liquid waste release permits, including the projected doses to members of the public, to verify that appropriate treatment equipment was used and that the radioactive gaseous and liquid effluents were processed and released in accordance with ODCM and Radiological Effluent Control Manual (RECM) requirements.

b. Findings

No findings of significance were identified.

.4 Changes to the ODCM

a. Inspection Scope

The inspector reviewed changes made by the licensee to the ODCM as well as to the liquid or gaseous radioactive waste system design, procedures, or operation since the last inspection. For each ODCM revision that impacted effluent monitoring or release controls, the inspector reviewed the licensee's technical justifications for the changes and determined if the changes were made in accordance with the requirements of the Technical Specifications.

b. Findings

No findings of significance were identified.

.5 Dose Calculations

a. Inspection Scope

The inspector reviewed a selection of monthly, quarterly, and annual dose calculations to ensure that the licensee has properly calculated the offsite dose from radiological effluent releases and to determine if any annual Technical Specification (TS) or ODCM (i.e., Appendix I to 10 CFR Part 50 values) limits were exceeded.

b. Findings

No findings of significance were identified.

.6 Air Cleaning Systems

a. Inspection Scope

The inspector reviewed air cleaning system surveillance test results to ensure that test results are within the licensee's acceptance criteria. The inspector reviewed surveillance test results for the stack and vent flow rates to verify that the flow rates are consistent with FSAR values.

b. Findings

No findings of significance were identified.

.7 Effluent Monitor Calibrations

a. Inspection Scope

The inspector reviewed records of instrument calibrations performed since the last inspection for each point of discharge effluent radiation monitor and flow measurement device. The inspector reviewed any completed system modifications and the current effluent radiation monitor alarm setpoint value for agreement with ODCM requirements.

b. Findings

No findings of significance were identified.

.8 Interlaboratory Comparison Program

a. Inspection Scope

The inspector reviewed the results of the year 1999 interlaboratory comparison program as reported in the 1999 Annual Monitoring Report to verify the quality of radioactive effluent sample analyses performed by the licensee. The inspector reviewed the licensee's quality control evaluation of the interlaboratory comparison test and associated corrective actions for any deficiencies identified.

b. Findings

No findings of significance were identified.

.9 Identification and Resolution of Problems

a. Inspection Scope

The inspector reviewed years 1999 and 2000 assessment reports to evaluate the effectiveness of the self-assessment process to identify, characterize, and prioritize problems and verified that previous radiological instrumentation related issues were adequately addressed. The inspector also reviewed all year 2000 Condition Reports (CR) that addressed radioactive treatment and monitoring program deficiencies. The

review was conducted to verify that the licensee had effectively implemented the corrective action program.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA1 Performance Indicator Verification

a. Inspection Scope

The inspector verified the licensee's assessment of its performance indicators (PI) for Public Radiation Safety. Since no reportable elements were identified by the licensee for the 1st, 2nd, 3rd, and 4th quarters of 2000, the inspector compared the licensee's data with 1st, 2nd, 3rd,and 4th quarter dose assessments to verify that there were no occurrences concerning the Public Radiation Safety Cornerstone.

b. Findings

No findings of significance were identified.

4OA6 Management Meetings

Exit Meeting Summary

The inspector presented the inspection results to Mr. Cayia, and other members of licensee management and staff, in an exit meeting on January 12, 2001. The licensee acknowledged the information and findings presented. No proprietary information was identified by the licensee.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

A. Cayia, Plant Manager
J. Lindsay, General Supervisor-RPt
R. Repshas, Manager-Site Services
D. Shannon, Radiation Protection Supervisor
S. Thomas, Radiation Protection Manager

NRC

M. Kunowski, Project Engineer

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None

Closed

None

Discussed

None

LIST OF ACRONYMS USED

ALARA	As-Low-As-Is-Reasonably-Achievable
CFR	Code of Federal Regulations
CR	Condition Report
FSAR	Final Safety Analysis Report
NRC	Nuclear Regulatory Commission
ODCM	Offsite Dose Calculation
PDR	Public Document Room
RCA	Radiologically Controlled Area
RECM	Radiological Effluent Control Manual
RP	Radiation Protection
RPT	Radiation Protection Technician
RWP	Radiation Work Permit
TS	Technical Specification

PARTIAL LIST OF DOCUMENTS REVIEWED

The following documents were selected and reviewed by the inspectors to accomplish the objectives and scope of the inspection and to support any findings.

Condition Reports

99-0813, 99-1066, 00-0725, 00-1025, 00-1100, 00-1594, 00-3173, 00-3221, 00-4080, 01-0027, 01-0035

Surveillance Reports

Radioactive Effluent Release Program Surveillance, March 17-19, 1999

Station Procedures

CAMP 030 (Revision 2)	Manual Preparation of Batch Liquid and Gaseous Discharge Release Permits
HPCAL 3.1 (Revision 24)	Liquid Monitor Calibration Procedure
HPIP 3.8 (Revision 16)	Stack Exhaust Monitor Calibration
HPIP 11.50 (Revision 16)	Filter Testing
ICP 4.37 (Revision 5)	Air Ejector Aftercondenser Air Flow
ICP 13.5 (Revision 4)	Radioactive Effluent Monitors
NP 5.2.16 (Revision 1)	NRC Performance Indicators
OP 9C (Revision 49)	Containment Venting and Purging

Calibration Records

Waste Disposal System Liquid Monitor	RE-218 (July 3, 1999)
SG Blowdown Liquid Monitor	RE-219 (August 18, 1999)
SG Blowdown Liquid Monitor	RE-219 (February 16, 2000)
Aux Bld Exhaust Vent NG Monitor	RE-214 (February 14, 2000)
Drummer Area Vent NG Monitor	RE-221 (December 22, 2000)
Gas Stripper Bld Exhaust NG Monitor	RE-224 (July 19, 1999)
Air Ejector Aftercondenser Air Flow Monitor	Unit 1 (October 17, 1999)
Air Ejector Aftercondenser Air Flow Monitor	Unit 2 (October 15, 2000)
Waste Condensate Pump Flow	F-1064 (June 7, 2000)
Waste Distillate Flow	LW-15 (June 8, 2000)
SG Blowdown Flow Indicator A	IFI-5940 (June 7, 2000)
SG Blowdown Flow Indicator B	IFI-5941 (June 7, 2000)
Gas Decay Tank Flow	FI-014 (June 6, 2000)

Effluent Release Permits

00-000080G	Containment Forced Vent (December 14, 2000)
01-000026G	Containment Forced Vent (January 9, 2001)
00-00067L	Liquid Discharge (September 17, 2000)

Other Documents

1999 Annual Monitoring Report
Environmental Manual, Revision 14
Offsite Dose Calculation Manual, Revision 12
Radiological Effluent Control Manual, Revision 1
Technical Specifications
Annual Filter Testing Checklist, Year 2000
Charcoal Filter Log for U2 Purge Exhaust "B"
Monthly Dose Summaries for April, June and August 2000
Complete Monthly Release Summaries for April, June and August 2000
Point Beach Nuclear Plant Manager's Supervisory Staff Meeting, April 25, 2000
SCR 2000-0318-01, 10 CFR 50.59/72.48 Screening and Safety Evaluation (Revision 13 to the ODCM)
Document Review and Approval Form for Revision 13 to the ODCM
Document Review and Approval Form for OI 38, "Circulating Water System Operation"
Action Item Status Report for CR 99-0813
Action Item Status Report for CR 00-3221