August 22, 2000

Mr. Thomas J. Palmisano
Site Vice President and General Manager
Palisades Nuclear Generating Plant
Consumers Energy Company
27780 Blue Star Memorial Highway
Covert, MI 49043-9530

SUBJECT: PALISADES - NRC INSPECTION REPORT 50-255/2000012(DRP)

Dear Mr. Palmisano:

On July 28, 2000, the NRC completed the baseline problem identification and resolution inspection at your Palisades Nuclear Generating Plant. The inspection results were discussed on July 28, 2000, with Mr. D. E. Cooper, and other members of your staff.

The inspection was an examination of activities conducted under your license as they relate to identification and resolution of problems and compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection consisted of a selected examination of procedures and representative records, observation of activities, and interviews with personnel.

Based on the results of the inspection, we concluded that your program effectively identified and resolved conditions adverse to quality in that the inspectors did not identify any issues that resulted in the operability of safety-related or risk significant plant equipment being questioned. Also, we concluded that your personnel communicated an acceptable level of responsibility in identifying and entering safety issues into the corrective action program. However, during the inspection, several examples of minor problems were identified including conditions adverse to quality that were not being entered into the corrective action program, narrowly focused condition report evaluations, and corrective actions that were ineffectively tracked or had not occurred.

The problems identified during this inspection did not result in any adverse consequences and were similar to problems identified by your staff during recent self-assessments. However, the evaluations for condition reports generated from the self-assessments had not been completed when the inspection period ended. Consequently, we could not assess the effectiveness of any resultant corrective actions.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available **electronically** for public inspection in the NRC Public Document Room **or** from the *Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from* the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

We will gladly discuss any questions you have concerning this inspection.

Sincerely,

/RA/

Michael J. Jordan, Chief
Reactor Projects Branch 3

Docket No. 50-255
License No. DPR-20

Enclosure: Inspection Report 50-255/2000012(DRP)

cc w/encl: R. Fenech, Senior Vice President, Nuclear
Fossil and Hydro Operations
N. Haskell, Director, Licensing and Performance Assessment
R. Whale, Michigan Public Service Commission
Michigan Department of Environmental Quality
Department of Attorney General (MI)
Emergency Management Division, MI Department
of State Police

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U. S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket No:	50-255
License No:	DPR-20
Report No:	50-255/2000012(DRP)
Licensee:	Consumers Energy Company 212 West Michigan Avenue Jackson, MI 49201
Facility:	Palisades Nuclear Generating Plant
Location:	27780 Blue Star Memorial Highway Covert, MI 49043-9530
Dates:	July 17 through 28, 2000
Inspectors:	J. Lennartz, Senior Resident Inspector M. Farber, Reactor Engineer N. Shah, Reactor Engineer
Approved by:	Michael J. Jordan, Chief Reactor Projects Branch 3 Division of Reactor Projects

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety	Radiation Safety	Safeguards
● Initiating Events	● Occupational	● Physical Protection
● Mitigating Systems	● Public	
● Barrier Integrity		
● Emergency Preparedness		

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: http://www.nrc.gov/NRR/OVERSIGHT/index.html.

SUMMARY OF FINDINGS

IR 05000255-00-12; on 07/17 - 07/28/00; Consumers Energy Company; Palisades Nuclear Generating Plant; Annual baseline inspection of the Identification and Resolution of Problems.

The report covers a 2-week inspection by one resident inspector and two region-based inspectors. This was an announced inspection to review the effectiveness of the corrective action program which included the methods used for identification, cause investigation, and correction of quality related problems. The inspectors used Inspection Procedure 71152, "Identification and Resolution of Problems," to conduct the inspection.

Problem Identification and Resolution

The inspectors concluded that the licensee's program effectively identified and resolved conditions adverse to quality in that the inspectors did not identify any issues that resulted in the operability of safety-related or risk significant plant equipment being questioned. Also, the inspectors concluded that the licensee personnel communicated an acceptable level of responsibility in identifying and entering safety issues into the corrective action program. However, the inspectors identified several examples of minor problems that did not result in any adverse consequences which were similar to problems identified by licensee personnel during recent self-assessments. Identified problems included conditions adverse to quality that were not being entered into the corrective action program, narrowly focused condition report evaluations, and corrective actions that were ineffectively tracked or had not occurred. Licensee personnel had not completed the evaluations for the condition reports that were initiated for the self-assessment findings when the inspection period ended. Consequently, the inspectors could not assess the effectiveness of any resultant corrective actions. Also, no self-assessments had been completed in accordance with recently revised procedures and therefore the inspectors could not assess the licensee's current program.

Report Details

Summary of Plant Status: The plant operated at full power throughout the inspection period.

4. OTHER ACTIVITIES (OA)

Cornerstones: Initiating Events, Mitigating Systems, Barrier Integrity, Emergency Preparedness, Occupational Radiation Safety, Public Radiation Safety, and Physical Protection

4OA2 Problem Identification and Resolution

1. General Comments:

The most recent revision of the licensee's corrective action program procedure, Administrative Procedure 3.03, Revision 23, was issued on July 13, 2000. The program underwent significant changes to address program weaknesses identified by licensee personnel during recent self-assessments. The program changes included:

- simplifying the role of the initiator;
- redefining authorities;
- redistributing process tasks;
- strengthening the role of the Management Review Board and Condition Review Group;
- restructuring the root cause evaluation process; and,
- tightening the guidelines for closure of condition reports.

In addition to revising this key procedure, the licensee also recently revised the self-assessment procedure, Nuclear Performance Assessment Department audit guidelines, and the stations behavioral observation program. Consequently, the inspectors were unable to assess the effectiveness of the licensee's new program.

2. Effectiveness of Problem Identification

a. Inspection Scope:

The inspectors reviewed items that pertained to the seven cornerstones of safety related to the Reactor Safety, Radiation Safety, and Safeguards strategic performance areas to determine if problems were appropriately being identified, characterized, and entered into the corrective action program. Items reviewed included select condition reports, audits and self-assessments completed in the time period from June 1998 to the present. In addition, the inspectors conducted Emergency Diesel Generator and Auxiliary Feedwater System inspections which included a review of completed corrective maintenance activities, work order backlogs, related condition reports, and system health assessments. A listing of the specific documents reviewed during the inspection is attached to the report.

b. Issues and Findings:

There were no findings identified during this inspection. The inspectors determined that the licensee was effective at identifying problems and appropriately characterizing problems.

However, the inspectors identified minor examples of problems that were not entered into the corrective action program which included the following:

- Emergency preparedness group personnel failed to document in a condition report an identified adverse trend in preparation for drills during an internal self-assessment. After discussions with the inspectors, the licensee subsequently agreed with this concern and documented this issue in Condition Report CPAL0002271, “Adverse Trend Noticed In Emergency Planning Preparation for Drills.”

- The licensee had identified the outboard pump packing on a steam driven auxiliary feedwater pump as glazed and smoking. However, no condition report was written to address a problem with replacement of a lantern ring, that was documented in the work order, nor to address the need for an evaluation of the unexamined inner four rings of packing.

- A memo, “Palisades Plant - Plant Equipment Status Quarterly Multi-Discipline Review - June 20, 2000," dated July 20, 2000, documented a multi-discipline review of plant equipment status. This working group assessed the cumulative effects of equipment deficiencies on plant operations and identified a series of action items assigned to specific individuals as documented in the memo. While the equipment status review was considered pro-active, not entering the results in the corrective action program could preclude corrective actions from being completed. Subsequent to discussions with licensee personnel ten condition reports were written (CPAL0002373 through CPAL0002382 inclusive) by the working group which were entered into the corrective action program to address this issue.

The inspectors determined, based on followup questioning, that the issues were minor in that no equipment operability concerns were revealed.

A second issue regarding the licensee’s effectiveness in problem identification that the inspectors noted pertained to the trending program. The trending program was narrowly focused in the review of the data and was very subjective. Assignment of trend codes was based on the interpretation of the person who completed the trend code sheets and identification of adverse trends was typically only done based on review of condition reports which tripped a monthly “hot button” threshold (three/month). Other sorts of the condition reports could reveal additional trends which were not coded for that aspect. For example, the inspectors requested a sort of condition reports from January 1998 to the time of the inspection on the subject of post-maintenance testing. A “keyword” search under the code “PMT,” listed 55 condition reports; however, two other searches using “PMT” and “post-maintenance testing” in the “title” and the “description” turned up 31 and 113 condition reports, respectively. Of the 31 condition reports in the “title”

search, seven were not included in the keyword search results. Identifying and understanding the limitations of the trending program raised the question as to whether or not unrecognized performance trends existed at the plant. During subsequent discussions, licensee personnel indicated that weaknesses in the trending program had previously been identified. Further, licensee personnel expressed confidence that as a result of other evaluations, including some with external assistance, no unrecognized performance trends existed. Therefore, while a vulnerability existed with the effectiveness of the trending program, the inspectors considered this issue as minor.

3. Prioritization and Evaluation of Issues

a. Inspection Scope:

The inspectors' reviewed condition reports, audits, and self-assessments to verify that identified issues were appropriately characterized, an appropriate analysis of the cause of the problem was performed for significant conditions adverse to quality, and the risk associated with combinations of issues was appropriately considered. In addition, the inspectors reviewed the licensee's evaluation of selected industry experience information to assess if issues applicable to Palisades were appropriately addressed. Documents reviewed included operating event reports, and NRC and vendor generic notices. Information that the inspectors reviewed was selected in the time period from June 1998 to the present. A listing of the specific documents reviewed during the inspection is attached to the report.

b. Issues and Findings:

There were no findings identified during this inspection. In general, issues were appropriately characterized and appropriate evaluations were conducted for significant conditions adverse to quality. The inspectors did not identify any issues regarding the risk associated with combinations of issues. In addition, the inspectors noted that the licensee's program that utilized industry information was recently revised to address licensee identified weaknesses.

However, the inspectors identified minor examples of narrowly focused evaluations. For example:

- Brass flakes were found in an oil sample from Auxiliary Feedwater Pump P-8A and was attributed to improper coupling assembly in 1989 with corrective action being that because it was an old error, there was no basis to inspect pumps with the same or similar couplings. However, later examinations of pump history by licensee personnel revealed that the coupling had been disassembled on other occasions since 1989 which could have been the cause of the brass flakes.

- Three condition reports documented occasions where the security staff had failed to perform a detailed vehicle search. The limited evaluations resulted in the corrective actions for the first two events, which occurred within a few days of each other, being ineffective in preventing the third event which occurred several months later.

- The evaluation of NRC Information Notice 99-021, "Recent Plant Events Caused By Human Performance Errors," did not address actual human performance in relation to that assumed in plant-specific probabilistic risk assessments and that performance remained consistent with management expectations. Following discussions with the inspectors, licensee personnel generated Condition Report CPAL0002343 to enter this issue into the corrective action program.

Regarding the use of industry experience information, during several audits and self-assessments in 1999, licensee personnel identified numerous deficiencies. Specifically, industry experience was not properly tracked and evaluated, and examples where corrective actions may not have been taken to prevent similar plant events. As a result, Administrative Procedure 3.16, "Industry Experience Review Program," was revised on June 16, 2000. The revised procedure appeared to address the identified weaknesses; however, technical notices issued by the nuclear steam supply system vendor were not included as industry experience. Consequently, the inspectors identified one technical notice that had not been reviewed or evaluated. Licensee personnel subsequently generated Condition Report CPAL002225, "Combustion Engineering Tech-Note 99-4 not Evaluated," to enter this issue into the corrective action program. The inspectors did not identify any adverse consequences that resulted from not reviewing the technical notice. However, the inspectors determined that the failure to include vendor technical notices as industry experience could preclude an evaluation of the issue.

4. Effectiveness of Corrective Actions

a. Inspection Scope:

The inspectors reviewed condition reports, audits, and self-assessments to verify that corrective actions commensurate with the issues were identified and implemented in a timely manner, including corrective actions to address common cause or generic concerns. Information that the inspectors reviewed was selected in the time period from June 1998 to the present. A listing of the specific documents reviewed during the inspection is attached to the report.

b. Issues and Findings:

The were no findings during this inspection. However, the inspectors noted several instances where there was insufficient documentation to track or determine completion of identified corrective actions. For example:

- A condition report was closed without documenting any action to replace or evaluate a stuck lantern ring on Auxiliary Feedwater Pump P-8B. Also, an engineering analysis request to evaluate use of a different style lantern ring was not tied to any condition report. Consequently, there was no tracking mechanism to ensure that the identified corrective actions would be completed.

  Following discussions with the inspectors, licensee personnel generated Condition Report CPAL0002328, "Actions Specified in Condition Report Evaluation Not Formally Initiated," to enter this issue into the corrective action program.

- Corrective actions to provide training to a specific audience were closed based on documented statements such as “training provided” and “routed information.” However, there was not any documentation to indicate who actually received the training.

- A corrective action to conduct an assessment in the first quarter of 2000 was closed based on scheduling the assessment. However, there was no tie from the scheduled assessment to the condition report. Consequently, the potential existed for the corrective action to not get completed if the assessment was dropped from the schedule.

In addition to the documentation deficiencies, the inspectors identified that a memo was issued to all operators which provided written guidance for breaker tagging. The inspectors considered this an inappropriate action to prevent recurrence in that a memo would not provide permanent guidance. Following discussions with the inspectors, licensee personnel generated Condition Report CPAL0002279, “Corrective Actions Provides Guidance Through Memo Only and No Permanent Guidance,” to enter this issue into the corrective action program.

The inspectors did not identify any resultant adverse consequences from the identified documentation deficiencies or the inappropriate corrective action to prevent recurrence. Therefore, the inspectors considered these deficiencies minor and administrative in nature.

5. Effectiveness of Licensee Audits and Assessments

a. Inspection Scope:

The inspectors reviewed selected licensee audits and self-assessments performed since June 1998. The review was conducted to determine whether the audit and self-assessment programs were effectively managed, adequately covered the subject areas, and to determine whether the associated findings were appropriately captured in condition reports. In addition, the inspectors interviewed licensee personnel regarding the audit and self-assessment programs. A listing of the specific documents reviewed during the inspection is attached to the report.

b. Issues and Findings:

There were no findings identified during this inspection. Recently completed self-assessments adequately covered the subject area and the associated findings were appropriately captured in condition reports.

The licensee conducted four self-assessments of the corrective action program between March and the start of this inspection which were documented in the following reports:

- 2000-01, Assessment of Palisades Implementation of INPO Document - Principles for Effective Self-Assessment and Corrective Action Programs, dated March 1, 2000
- 2000-02, Corrective Action Process, dated June 5, 2000

- 2000-03, Root Cause Quality Assessment, dated July 17, 2000
- 2000-04, Assess the Corrective Action Program to Identify and Correct Potential Weaknesses Prior To An NRC Inspection Planned for July 17, 2000, dated July 16, 2000

The assessments were considered thorough and intrusive in that a broad range of corrective action program topics were covered. Also, most of the documented findings and observations were similar to the examples identified by the inspectors.

Licensee personnel generated five condition reports as a result of these self-assessments. The inspectors determined that the important issues were correctly identified and characterized as documented in the following condition reports:

- CPAL0002048, "Improvements Needed in the Areas of Identification, Classification, and Prioritization of Condition Reports"
- CPAL0002049, "Root Cause Evaluations Not Always Effectively Performed"
- CPAL0002050, "Corrective Actions Not Always Effectively Defined"
- CPAL0002051, "Condition Report Corrective Actions are not Always Completed Timely and Justifications for Extensions and Changes are Not Well Documented"
- CPAL0002052, "Corrective Action Program Programmatic Processes"

However, all the condition reports had evaluation due dates of August 18, 2000. Consequently, the inspectors could not assess any resultant corrective actions.

Also, between June and July 2000, the licensee issued revised procedures for conducting audits and self-assessments to address licensee identified program deficiencies. Specifically, the procedures provided specific guidance on how to conduct and document self-assessments; on how to track and document audit findings and recommendations; and on conducting quarterly management reviews of the process. However, no audits or self-assessments had been completed in accordance with the revised guidance. Therefore, the inspectors could not assess the effectiveness of the licensee's current audit and self-assessment programs.

6. Assessment of Safety Conscious Work Environment

a. Inspection Scope:

The inspectors interviewed eight supervisors, eight maintenance repair technicians, five nuclear performance assessors, two auxiliary operators, two radwaste handlers, three radiological protection technicians, and two system engineers regarding the licensee corrective action program, including the employees concerns program. The type of questions included in Appendix 1 to NRC Inspection Procedure 71152, "Suggested Questions For Use In Discussions With Licensee Individuals Concerning PI&R Issues," were utilized during the interviews to assess whether conditions existed that would challenge the establishment of a safety conscious work environment. In addition, the inspectors reviewed the following documents:

- Internal Correspondence, "Employee Concerns Communication Policy Re-Issue," dated April 19, 2000;

- Palisades Employee Concerns Program Advocate Concern Log, dated July 14, 2000; and,
- Nuclear Performance Assessment Procedure - 24, "Employee Concerns Advocate," Revision 2

b. Issues and Findings:

There were no findings during this inspection. The inspectors concluded, based on information collected from interviews with 30 licensee personnel, that licensee personnel communicated an accepted responsibility to pro-actively identify and enter safety issues into the corrective action program. However, the inspectors noted the licensee's program did not contain a formal mechanism to provide feedback to plant workers regarding how condition reports were resolved. The inspectors also determined that the several interviewees lacked general information about the employees concerns program (ECP) such as who the employee concerns program advocate was; who the advocate reported to; and, how to enter a concern into the program. The inspectors considered these as potential detriments to fostering an environment to raise issues in that plant workers were not necessarily being informed how identified issues were getting resolved and that an alternate program (ECP) for raising issues was not well understood. Nuclear performance assessment personnel generated Condition Report CPAL0002387, "Less Than Adequate Site Awareness of the Employee Concerns Program," after the inspectors briefed plant management. Therefore, the issue was appropriately entered into the licensee's corrective action program.

4A05 Other

(Closed) Unresolved Item (URI) 50-255/98012-02 (DRS): "Licensee to Perform Quantitative Analysis." (Engineering Issue Regarding Containment Sump)

The inspectors reviewed the licensee's proposed and completed corrective actions. The inspectors noted that corrective actions were specified to re-analyze emergency core cooling system flow with 50 percent screen blockage, assess the strength of the containment sump screens, and re-evaluate the containment sump debris issue.

The first action remains outstanding and is scheduled for completion on June 30, 2001. Difficulty in obtaining vital information resulted in the need to develop a detailed computational flow model to predict valve characteristics. This caused a consequent delay in the analysis. Also, emergency core cooling system pump flow data must be captured during the 2001 refueling outage in order to validate the model. Given these circumstances, the length of time needed to develop the new analysis was understandable.

For the second corrective action, an engineering analysis was completed to assess the strength of the sump screens and concluded that the screen was structurally adequate up to a differential pressure of .665 pounds per square inch. The third action was closed because it was already being addressed in another condition report.

This issue was entered into the licensee's corrective action program as CPAL9801408 which will track completion of all corrective actions. This issue is closed.

4OA6 Management Meetings

Exit Meeting Summary

The inspectors presented the results to Mr. D. E. Cooper, Plant General Manager, and other members of licensee management at the conclusion of the inspection on July 28, 2000. The licensee acknowledged the findings presented.

The inspectors asked the licensee whether any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

M. P. Banks, Corrective Actions
W. A. Binnington, Maintenance and Construction
E. Chatfield, Self Assessment
D. E. Cooper, Plant General Manager
D. D. Crabtree, Performance Assessment
J. E. Giebel, System Engineering
N. L. Haskell, Director, Licensing and Performance Assessment
D. G. Malone, Regulatory Compliance
K. E. Osborne, Engineering Programs
T. J. Palmisano, Site Vice President
J. C. Petro, Nuclear Performance Assessment Department
J. R. Schepers, Nuclear Performance Assessment Department
J. D. Slinkard, System Engineering
G. B. Szczotka, Manager, Nuclear Performance Assessment Department
K. A. Toner, Industry Experience and Performance Indicators

NRC

R. G. Krsek, Resident Inspector
M. L. Dapas, Deputy Director, Division of Reactor Projects, RIII

ITEMS OPENED , CLOSED, AND DISCUSSED

Opened

None

Closed

50-255/98012-02 URI Licensee to Perform Quantitative Analysis

Discussed

None

LIST OF DOCUMENTS REVIEWED

Palisades Nuclear Plant Procedures:

- Administrative Procedure 1.09, “Self-Assessment Program,” Revision 6
- Administrative Procedure 3.03, “Corrective Action Process,” Revision 21
- Administrative Procedure 3.03, “Corrective Action Process,” Revision 22
- Administrative Procedure 3.03, “Corrective Actions Process,” Revision 23
- Administrative Procedure 3.16, “Industry Experience Review Program,” Revision 8
- Administrative Procedure 4.02, “Control of Equipment,” Revision 17
- Permanent Maintenance Procedure MSS-M-19, “Disassembly, Inspection, and Reassembly of Main Steam Isolation Valves CV-0501 and CV-0510," Revision 15
- Technical Specification Surveillance Test Procedure QO-37, “Main Steam Isolation and Bypass Valve Testing,” Revision 4
- Health Physics 2.5, “High Radiation Entry and Control,” Revision 17
- Health Physics 2.17, “Performance of Radiation and Contamination Surveys,” Revision 15
- Health Physics 2.20, “Radiation Safety Area Posting,” Revision 12
- Technical Specification Surveillance Test Procedure Refueling Radiation - 9E, “East Engineered Safeguards Room Ventilation Monitor RIA-1810 Calibration,” Revision 7
- Technical Specification Surveillance Test Procedure RR-9F, “West Engineered Safeguards Room Ventilation Monitor RIA-1811 Calibration,” Revision 6
- Technical Specification Surveillance Test Procedure RR - 9M, “Containment Atmosphere Gas Monitor RIA-1817 Calibration,” Revision 2
- Nuclear Performance Assessment Procedure - 24, “Employee Concerns Advocate,” Revision 2

Condition Reports (CPAL):

	Number	Title
•	95-0961A	“SOER 95-01 Reducing Events Resulting from Foreign Material Intrusion”
•	97-1363	“Component Cooling Water System Not Analyzed for Present LOCA Containment Analysis”
•	9800248	“Discovery of Card Reader Vulnerability and Incorrect Compensatory Actions Taken”
•	9800766	“Unassigned Assembly Area”
•	9800867	“Discovery of a Unsecured High Radiation Area Barrier”
•	9801123	“Trend in Radiation Safety Performance From Condition Reporting Group “Hot Button” Report”
•	9801190	“CV-0727 Passes Flow With Closed Signal to Valve”
•	9801298	“Auxiliary Feedwater FIC-0449 Reads 20 gpm With Pumps Off (FT-0749)”
•	9801440	“PC-0522B Setpoint Incorrectly Set to 225 psi”
•	9801499	“Aux Fd Controllers FIC-0736A/0737A Program Abnormality”
•	9901408	“Adequacy of ECCS Pump NPSH under Increased Screen Blockage”

- 9801480 “NRC Inspection Report 98010 and Notice of Violation Regarding the Plant Being In An Improper Configuration During Refueling Test - 8C on 5/16/98 and Standard Operating Procedure - 4 on 5/17/98"
- 9801533 “Report of Shipping Cask Dose Rate Exceeding Department of Transportation Contact Dose Rate Limit”
- 9801668 “INPO SOER 98-02 Circuit Breaker Reliability”
- 9801694 “Bird Feathers & Other Foreign Materials Exhausting from P-8B Steam Traps”
- 9801720 “Radiation Shipment Having External Dose Readings $\geq$ 200 mr/hr”
- 9801788 “Common Cause Trend - Maintenance Activity”
- 9801807 “Seismic Concerns”
- 9801842 “Determine if Current Process Monitor Calibration Meets Offsite Dose Calculation Manual (ODCM) Requirements”
- 9801846 “10 CFR Part 21 Report (Coltec Industries) of Wrist Pin Sleeves Supplied With Incorrect Material”
- 9801848 “Apparent Weakness in Meeting Requirements of NRC Generic Letter 90-03"
- 9801892 “Control Room and Technical Support Center Telephones Used for State/County Notifications Not Functioning”
- 9801893 “Nitrogen Station 3A Found Inoperable With Primary Coolant System Greater than 300 °F”
- 9801942 “Main Steam Isolation Valve Bypass Leakage”
- 9801957 “Abnormal Release of Radioactivity”
- 9801989 “Unidentified Noise In 1-2 Emergency Diesel Generator”
- 9900018 “Maintenance Staffing Levels Not in Accordance With Emergency Plan”
- 9900038 “EDG 1-1 Failed to Meet MO-7A-1 Criteria”
- 9900107 “Declassified Documents Determined to be Safeguards Information”
- 9900147 “Emergency Diesel Generator 1-1 Fuel Oil Priming Pump P-210A Failed To Prime”
- 9900271 “Audit-99-01, Deviation from Technical Specification Requirement - AFW Flow Indication Channel Check”
- 9900372 “Undervoltage Relay G1-2/27-K1 Pickup Set Point Out Of As Found Tolerance”
- 9900492 “Failure of Steam Trap Internals”
- 9900499 “Steam Trap ST-0521 Rebuilt Without Adequate PMT Planned”
- 9900513 “Out of Tolerance Data on Power Supply Ripple”
- 9900514 “Control Room Received a FOGG Alarm”
- 9900568 “Calculated Valve Factor from DP Test Results are Non Conservative”
- 9900638 “Errors Found In Radioactive Material Shipping Papers”
- 9900743 “Operability of ST-0521 Uncertain during Performance of MO-38"
- 9900782 “Non-conservative Assumption In Engineering Analysis”
- 9900864 “Vent Line Inadvertently Disconnected at High Efficiency Particulate Air Filter (HEPA)Intake”
- 9900884 “AFW Flow Controllers HIC & FIC-0736A Alarm Lights Lit”
- 9900966 “Emergency Response Organization Member Location Change not Reported To the Emergency Planning Group by Human Resources”
- 9901002 “Augmentation Tests”
- 9901043 “Training and Communication Concerns Regarding Operations Procedure Changes”

- 9901305 “Unposted High Radiation Area”
- 9901324 “NRC Operator License Exams - Operating Test Failures”
- 9901346 “Test MO-7A-2 Design Basis Accident Load Acceptance Criteria Less Than Required Per Recent Load Calculations”
- 9901460 “Improper Control of Safeguards Information”
- 9901469 “Temporary Modification of Failed Fuel Monitor Not Reflected In Emergency Plan or Simulator During Emergency Preparedness Drill”
- 9901480 “Personnel Exits From a Contaminated Area Without Performing a Proper Frisk Of Hands and Feet”
- 9901487 “Discrepancy In Service Water Flow Requirements For Diesel Generators”
- 9901490 “EQ Program Weaknesses Found during EPRI Self-Assessment”
- 9901699 “Shorted Diode Caused 72-302 Breaker Trip and Diesel 1-1 Failed to Start Properly”
- 9901703 “Diesel Generator 1-1 Voltage Could Not Be Controlled In Automatic Or Manual”
- 9901730 “Newly Installed Fuel Oil Booster Pump P-209A Did Not Develop Expected Discharge Pressure”
- 9901817 “Control Rod Failure to Trip”
- 9901898 “CA Tracking of Inoperable Equipment Ineffective in Tracking True Equipment Operability”
- 9901920 “NRC Identified Incomplete Vehicle Search”
- 9901935 “Line and Training Management Oversight Activities Do Not Always Ensure Training is Effectively Conducted
- 9901937 “Training Program and Student Performance Improvement Opportunities Are Not Always Effectively Identified or Implemented”
- 9901939 “Operator Training Lesson Plans Do Not Always Contain Complete and Accurate Information”
- 9901986 “Inadvertent Control Valve Manipulation”
- 9902007 “NRC Identified Incomplete Vehicle Search”
- 9902033 “PC-0522A Failed Its Post-Calibration Checks While Performing Calibrations”
- 9902168 “Apparent Adverse Trend Reflected in Condition Reports Associated with Design Issues”
- 9902209 “The Appendix R Analysis Failed To Consider the Coordination Between the Inverter Fuse and the Load Breakers”
- 9902268 “Failure of CK-SW407, CK-SW408, and CK-SW4009"
- 9902295 “CRDM Housing Crack and Indication”
- 9902331 “Adverse Trend in Foreign Material Exclusion (FME) Program”
- 9902428 “Service Water System Failed To Meet Test Minimum Flow Requirements To Emergency Diesel Generator 1-1 During Test T-216"
- 9902482 “Work Released Which Breached Primary Coolant System and Shutdown Cooling Boundary While In Reduced Inventory”
- 9902659 “Test FWS-I-19 Failed Due to Right Channel AFAS Actuation Cabinet Having No Power”
- 9902698 “AFAS/FOGG Panel +12 VDC Input Power Supply Design Deficiency”
- 9902707 “Use of Plastic Tie Wraps in Containment”
- 9902747 “Technical Specification Surveillance DWO-1 Not Performed As Scheduled”

- 9902843 “Escape Hatch in the 590' Elevation of the Component Cooling Water Area Was Not Posted as a High Radiation Area”
- 9902978 “Inadequate CR Response to INPO Finding Action Item”
- 9903038 “Management Review Board (MRB) for Self-Assessment Effectiveness not Performed as Required by AP 1.09 (NPAD Surveillance P-99-029)”
- 9903056 “Packing Failure on Outboard Pump Packing Gland Steam Driven Aux Feedwater Pump”
- 0000050 “Technical Specifications Action Statement Unknowingly Entered During Performance of Health Physics Monthly Testing”
- 0000110 “Ineffective Use of Operating Experience”
- 0000233 “Sample Valve From Letdown Line Found Open”
- 0000155 “Service Water Booster Pump P-7B Peak Calculated Electrical Power Requirement Higher Than Bounding Power Value used In Load Study”
- 0000172 “Calculated Short Time Overload On D20/D07 Inverter Cable During Station Blackout”
- 0000264 “Primary Coolant System Sampling Error”
- 0000294 “Received EK-1110, T-2 Condensate Storage Tank High Temperature Alarm”
- 0000330 “Problems Identified on Diesel Generator Output Breaker 152-07"
- 0000394 “Alternate Steam Supply Line for Aux Feedwater Pump 8B Leaking Below Floor Under Drain Cooler 7A”
- 0000408 “Control Rod Drive Mechanism No. 12 has High Seal Leak Off Temperature”
- 0000483 “P8-C Work Canceled due to Dose Concerns”
- 0000501 “Unexpected Diesel Generator 1-2 Trouble Alarm”
- 0000509 “Chemistry Sample Valve, SV-1914, Not Returned to Secured Position After Completion of Sampling”
- 0000555 “Boron Standard Used Past It’s Shelf Life”
- 0000559 “CRD-06 Has High Seal Leak Off Temperature”
- 0000567 “Unsatisfactory Control and Management of Work Order Process”
- 0000574 “Unexpected Diesel Generator Trouble Alarms”
- 0000585 “Discovery That Contents of VSC-24 Are Not Explicitly Specified by NRC Certificate of Compliance”
- 0000625 “Auxiliary Feedwater Pump P-8A Outboard Motor Bearing Oil Contained Metal Shavings”
- 0000726 “Failed Security Task During Emergency Drill”
- 0000781 “Palisades List of Key Activities Used for Condition Report Trending Needs Revision”
- 0000852 “Primary Coolant System Dose Equivalent Iodine Analysis Error”
- 0000856 “Primary Coolant System Boron Analytical Error”
- 0001019 “Radwaste System “A” Evaporator Bottoms Sample Valve Left Open”
- 0001032 “Adverse Trend In Human Performance Indicators For Chemistry”
- 0001050 “Inadvertent Manual Closure of Emergency Diesel Generator 1-1 Output Breaker Cause Motorization of Emergency Diesel Generator”
- 0001051 “A S/G Steaming Harder Than B S/G ADVS Following Plant Trip”
- 0001089 “Design Engineering Performance Weaknesses during the 99 REFOUT”
- 0001228 “Degraded System Fire Protection Performance”
- 0001240 “NRC Performance Indicator Data Entry Weaknesses”

- 0001274 “Design/Procedural Deficiency for Large Break LOCA with Loss of 1-1 D/G”
- 0001352 “Updated Motor Acceleration Times Not Included in DBA Sequencer Timing Study”
- 0001391 “Inadequate Emergency Operating Procedure (EOP) Reviews”
- 0001397 “Evaluation of Combined Loading on CV-3018 Actuator 1" Tie Bolts Not Documented in EA and Spring Can Angle from Vertical Incorrectly Identified”
- 0001398 “Review of Last HPSI Pump Outboard Bearing Oil Sample Dated 3/20/98, Identified High Ferrous Particulate and High Iron Content”
- 0001427 “Weakness in Basis for the Frequency of Safety Related Pump Oil Changes”
- 0001497 “Missing Valve Parts”
- 0001505 “Simulator Autodialer Failure to Connect to Van Buren County”
- 0001516 “Offsite Communications Concerns During the May 9 Practice Exercise”
- 0001547 “Ineffective Corrective Action for CPAL 9800522 (Weakness in the Observation Program)”
- 0001554 “C-6B’s Oil Sample Contains Metal”
- 0001572 “Variable High Power Pre-Trips and Nuclear Delta-T Deviations Received on “B” and “D” Channels”
- 0001598 “INPO March 2000 Evaluation - Area for Improvement OR 2-3"
- 0001600 “INPO Significant Operating Experience Reports 97-1, 98-2, and 98.2 Not Satisfactorily Evaluated”
- 0001631 “Moisture Separator Reheater Temperature Control Valves Spuriously Closed and Reopened “
- 0001649 “WO lacking Dedication Plan for Parts Being Installed in AFW Steam Trap”
- 0001743 “Incorrect Definition Used for NEI 99-02, rev 0, “Reactor Coolant System Leakage” Performance Indicator”
- 0001759 “Incorrect Entry of Protected Area Security Equipment Performance Indicator Data”
- 0001774 “Phone Problems Experienced by Technical Support Center State Communicator During Palisades Exercise 2000"
- 0001781 “Management Review Board Canceled Because Attendance Expectations Not Met”
- 0001803 “On-Shift Operations Human Performance Errors”
- 0001879 “Incorrect Data In Performance Indicator for Drill/Exercise Performance”
- 0001885 “Corrective Actions Closed Prior to Completion of Stated Actions Associated With CPAL 99-0638"
- 0001913 “Palisades Public Warning System Siren Failures During Scheduled June 2000 Test”
- 0001963 “Practice of Aborting Surveillance Procedures Requires Assessment”
- 0001977 “Auxiliary Feedwater Pump (P-8B) Cooling Water Manual Valve (MV-FW-144) Will Not Stroke”
- 0001982 “P-8B Turb Driver (K-8) Outboard Bearing Cooling Water Outlet Valve (MV-FW-147) Does Not Fully Stroke”
- 0001995 “Q List Error in Equipment Data Base for MV-FW-147"
- 0002048 “Improvements Needed in the Areas of Identification, Classification, and Prioritization of Condition Reports”

- 0002049 “Root Cause Evaluations Not Always Effectively Performed”
- 0002050 “Corrective Actions Not Always Effectively Defined”
- 0002051 “Condition Report Corrective Actions are not Always Completed Timely and Justifications for Extensions and Changes are Not Well Documented”
- 0002052 “Corrective Action Program Programmatic Processes”
- 0002105 “AFAS Reset Verification per GOP-2/SOP-12 Results in AFAS System Fault”
- 0002131 “Inadequate Completion and Follow Up on Self-Assessment 99-07"
- 0002218 “Change Management May Have Been Common Contributor in Operations Events (Self-Assessment CA 2000-03)”
- 0002219 “Generator Flex Connector Inspection Activities Not Identified As Corrective Actions (Self-Assessment CA 2000-03)”
- 0002220 “Potential Fuel Failure Vulnerabilities(Self-Assessment CA 2000-03)”
- 0002221 “Vulnerabilities in Interpretation of Tech Specs (Self-Assessment CA 2000-03)”
- 0002222 “Root Cause Not Identified for High Pressure Safety Injection Pump P-66A Trips in 1996 and 1997 (Self-Assessment CA 2000-03)”
- 0002223 “Potential Overflow of Cooling Towers (Self-Assessment CA 2000-03)”
- 0002224 “Root Cause Evaluation Not Performed for Level 2 Condition Report (Self-Assessment CA 2000-03)”
- 0002270 “Incorrect Determination of Setpoint for PC-0522B Related to Condition Report C-PAL-98-1440"
- 0002286 “Administrative Procedure 5.18, Step 4.2 Does Not Comply with Corrective Action of Closed Out Condition Report CPAL0000400"
- 0002298 “No Signature in Evaluated by Space on a Condition Report”
- 0002305 “Increased Vibration Levels on P-8A, Auxiliary Feedwater Motor and Pump”

Condition Reports Initiated As a Result of NRC Inspection

Number	Title
0002225	“Combustion Engineering Tech-Note 99-4 not Evaluated”
0002250	“Condition Report Actions Overdue Without Appearing on Coming Due or Overdue Lists”
0002271	“Adverse Trend Noticed In Emergency Planning Preparation for Drills”
0002277	“Inconsistent Requirements to Perform Effectiveness Reviews In Administrative Procedure 3.03, “Corrective Actions Process”
0002279	“Corrective Actions Provides Guidance Through Memo Only and No Permanent Guidance”
0002280	“Licensed Operators Not Provided Hands On Breaker Training”
0002289	“Operations and Training Communications”
0002297	“Management Review Board Review Not Documented on Condition Report “
0002328	“Actions Specified In Condition Report Evaluation Not Formally Initiated”
0002343	“Response To Information Notice 99-21 Was Narrowly Focused”

- 0002347 "Concern Regarding Effectiveness Of Corrective Actions To Resolve Inadequate Vehicle Search Events"
- 0002373 "Cumulative Impact of Open Work Orders on Chemical Volume Control System Heat Trace Circuitry"
- 0002374 "Cumulative Circulating Water System Valve Degradation May affect System Ability To Respond To A Cooling Tower Pump Trip"
- 0002375 "Failure of Corrective Action System To Correct Eyewash Station By Main Feedwater Pump P-1A, Water Temperature Too Hot For Eyes"
- 0002376 "No Administrative Controls For Operator Work Around To Prime Emergency Diesel Generator Fuel Oil Pumps"
- 0002377 "Confirm the Necessity For Operators To Reduce Steam Generator Pressure In Order To Utilize Pump P-8C To Feed the Steam Generators"
- 0002378 "Instrument Air Leaking Through N2 Station Check Valves May Potentially Overpressurize N2 System Piping"
- 0002379 "Determine If Appropriate Actions Are Being Taken To Address the Number of Open and BRT Work Orders On RWS"
- 0002380 "Determine If Appropriate Actions Are Being Taken To Address the Number of Open and BRT Work Orders On VAS"
- 0002381 "Housekeeping and Contamination Problems Caused By Frequent Oil Additions To SIRWT Recirculation Pump"
- 0002382 "Failure To Utilize the Corrective Action System In Documenting Equipment Deficiencies"
- 0002387 "Less Than Adequate Site Awareness of the Employee Concerns Program"

Licensee Self-Assessment Reports:

Number	Purpose / Objective
1999-05	Assess Plant Use of Operating Experience on a Just In Time Basis, dated December 15, 1999
1999-07	Assess the Radioactive Material Shipping Program, dated July 10, 2000
1999-09	Effectiveness Review for CPAL 98-1533, dated May 6, 1999
1999-10	Assess Corrective Actions Effectiveness for CPAL 98-1123, dated September 30, 1999
1999-18	Assess Engineering Department Use of Operating Experience, dated October 1, 1999
1999-27	Assess Use of Operating Experience In Training, dated June 22, 1999
2000-01	Assessment of Palisades Implementation of INPO Document - Principles for Effective Self-Assessment and Corrective Action Programs, dated March 1, 2000
2000-02	Corrective Action Process, dated June 5, 2000
2000-03	Roll-up of Data on Self-Assessment and Corrective Action Programs, dated February 29, 2000
2000-03	Root Cause Quality Assessment, dated July 17, 2000
2000-03	Second Assessment of Corrective Actions for CPAL-98-1123, dated February 4, 2000

- 2000-04 Assess the Corrective Action Program to Identify and Correct Potential Weaknesses Prior To An NRC Inspection Planned for July 17,2000, dated July 16, 2000
- PSPPD-0008 Determine Effectiveness of Actions in Vehicle Access Control Action Plan, dated August 4, 2000
- 2000-10 Radiological Protection Functional Area Assessment, dated February 17, 2000
- 2000-12 Self-Assessment of Corrective Actions for CPAL 99-0638, dated February 28, 2000

Licensee Audits:

	Number	Title
•	PA 99-06	"Palisades Emergency Preparedness Audit," dated August 20, 1999
•	PA 99-10	"Palisades Radiation Protection Audit, dated October 1, 1999
•	S-00-01	"NRC Performance Indicators," dated January 31, 2000
•	S-00-017	"Special Operations Assessment," dated July 19, 2000
•	A-00-01	"Palisades Operations," dated February 12, 2000

Generic Communications:

Includes: Industry Experience Reports (CIED), Combustion Engineering Tech-Notes (CETN) and NRC Information Notices (IN)

	Number	Title
•	CIED0000859	"Operating Experience 10796 - Grease On Bendix Drive Shafts Can Impact Emergency Diesel Generator Operation"
•	CIED0001340	"Emergency Preparedness Weaknesses During an Alert at Indian Point 2 (INPO Operating Experience 11062)"
•	CIED0000661	"Minimum Test Voltage for General Electric Type Circuit Breaker (follow up to 10 CFR Part 21 Notification from General Electric)"
•	CIED0000835	"Unescorted Access Granted Based on False Information Provided By Individual (follow up to Operating Experience Report OE 10783)"
•	CETN 99-01	"Safety Injection Tank Venting"
•	CETN 99-02	"Ultrasonic Flowmeter Frame Distortion"
•	CETN 99-03	"Reactor Vessel Internals Hold-Down Ring"
•	CETN 99-04	"Shutdown Cooling To Component Cooling Water Interface"
•	IN 98-002	"Nuclear Power Plant Cold Weather Problems and Protective Measures"
•	IN 98-022	"Deficiencies Identified During NRC Design Inspections"
•	IN 98-024	"Stem Binding in Turbine Governor Valves in Reactor Core Isolation Cooling (RCIC) and Auxiliary Feedwater (AFW) Systems"

- IN 98-025 "Loss of Inventory from Safety Related Closed Loop Cooling Water Systems"
- IN 99-010 "Degradation of Prestressing Tendon Systems in Prestressed Concrete Containments," Revision 1
- IN 99-019 "Rupture of the Shell Side of a Feedwater Heater at Point Beach"
- IN 99-021 "Recent Plant Events Caused By Human Performance Errors"
- IN 99-029 "Authorized Contents of Spent Fuel Casks"
- IN 00-003 "High Efficiency Particulate Air Filter Exceeds Mass Limit Before Reaching Expected Differential Pressure"
- IN 00-007 "NIOSH User's Notice Regarding Self-Contained Breathing Apparatus"

Work Orders:

Number	Activity
24011072	AFAS Actuation Channel (ACB) Remote Switch
24012200	ST-0521, Steam Trap, Steam Supply to K-8 Turbine
24012959	Aux FWP P-8B I-B Stuffing Box Cooling Water
24012968	Aux FWP Turbine Driver K-8, Outboard Bearing Cooling Water Inlet
24812244	Pressure Controller for CV-0522B
24813113	ST-0521 Inspect and Rebuild PM
24813981	AFAS Panel Power Supplies Test
24814181	P-8B Steam Supply Valve Calibration
24911270	P/S-0713; Power Supply for the 5 Volts, the Ripple was Out of Tolerance
24911543	ST-0521, Steam Supply to Auxiliary Feed Pumps
24911731	FIC-0736A/HIC-0736A: Intermittent Alarm (Yellow) Light
24911754	FIC-0737A: Inspect/Solder 250 Ohm Resistor Used on Y1 Output
24912055	HIC-0749; Check 250 Ohm Resistor (Input) for Proper Connection
24912055	HIC-0727; Check 250 Ohm Resistor (Input) for Proper Connection
24912823	Diesel Generator 1-1 Voltage Regulator Not Functioning
24912885	Diesel Generator 1-1 K-6A Fuel Oil Booster Pump
24914012	Replace Failed P-8B Outboard Packing and Shaft O-ring

Miscellaneous Documents:

- Palisades Leadership Observation and Coaching Policy, Revision 4, dated February 15, 1999
- Palisades Leadership Observation and Coaching Policy, Revision 5, dated July 19, 2000
- Licensee Event Report 99-001, "Failure to Perform Technical Specification Surveillance Channel Check of Auxiliary Feedwater Flow Indication," dated April 9, 1999
- Palisades Nuclear Plant Performance Enhancement Action Plan, Initiative 003, "Foreign Material Control Enhancement Initiative," Revision C, dated June 6, 2000
- Final Safety Analysis Report, Revision 22, Section 9.7, Auxiliary Feedwater System

- Temporary Modification TM-99-022, “Replace Breakers 52-9105 & 52-9106 with Spare Breakers,” dated September 20, 1999
- Engineering Analysis EA-E-PAL-94-041-01, “Material Susceptible to Damage During a LOCA in the Reactor Building,” dated June 10, 1994
- Engineering Analysis, EA-E-PAL-94-041-02, “Assessment of Potential Sump Blockage Due to Failed Plastic Equipment Tags, Signs, and Duct Tape,” dated May 25, 1994
- Engineering Assistance Request, EAR-2000-0302, “Install CHP Bypass for RB Spray Valves CV-3001 and CV-3002,” dated May 2, 2000
- Consumers Energy Memorandum, DEC2000.17, D.E. Cooper/D.J. Malone, Leadership Observation and Coaching Policy, dated July 19, 2000
- NRC Inspection Report 50-255/98011, dated July 2, 1998
- NRC Inspection Report 50-255/99005, dated May 19, 1999
- NRC Inspection Report 50-255/99007, dated June 7, 1999
- Internal Correspondence, “Employee Concerns Communication Policy Re-Issue,” dated April 19, 2000
- Palisades Employee Concerns Program Advocate Concern Log, dated July 14, 2000
- Degraded Equipment List, dated July 9, 2000
- Palisades Nuclear Plant Performance Enhancement Action Plan, Strategy 1.1, “Manage a Human Performance Improvement Initiative,” Revision 6, dated December 8, 1999
- Procedure Change Request 13043 and 13044, “Add a Second Sentence Which Requires a Check of Manual Trip Rod Hardware Tightness and Completeness”
- Emergency Diesel Generator 1-1 Monthly Performance Indicator Data for October 1999
- Emergency Diesel Generator 1-2 Monthly Performance Indicator Data for September 1998
- Emergency Diesel Generator 1-1 Monthly Performance Indicator Data for January 1999
- Emergency Diesel Generator 1-2 Monthly Performance Indicator Data for January 1999
- Preventative Maintenance Activity EPS079, “Emergency Diesel Generator Drain and Re-Fill Lube Oil System”
- Work Order Backlog For Auxiliary Feedwater System and Emergency Diesel Generators

- Summary of open Condition Reports and Corrective Actions for Emergency Diesel Generators for the period from June 1, 1998 through July 14, 2000
- Summary of all Condition Reports generated for Emergency Diesel Generators for the period from June 1, 1998 through July 14, 2000
- Summary of all Corrective Work Orders completed after July 1, 1998 for Emergency Diesel Generators
- Internal Correspondence, “Palisades Plant - Plant Equipment Status Quarterly Multi-Discipline Review - June 20, 2000"
- Internal Correspondence, “Nuclear Performance Assessment Department / Line Organization Interface Changes,” Dated June 7, 2000
- Security Departmental Master Action Plan (DMAP) 47, “Vehicle Access Control,” Revision 2
- Industry Experience Plant Review Committee Report dated 5/00
- Industry Experience Plant Review Committee Report dated 6/00
- Operating Experience Program Performance Indicators from 5/00 to 7/00
- Operating Experience Report 10783, “Unescorted Access Granted Based on False Information Provided by Individual”
- Training Guide HPI-07, “Radiation Survey Techniques,” Revision 7
- System Health Assessments - 2nd Quarter 1999 (5/1/99 through 7/31/99) for Control Rod Drive, Auxiliary Feedwater and Emergency Diesel Generator Systems
- System Health Assessments - 3rd/4th Quarter 1999 (8/1/99 through 1/31/00) for Control Rod Drive, Auxiliary Feedwater and Emergency Diesel Generator Systems