October 11, 2000

Mr. M. Wadley
Senior Vice President and Chief Nuclear Officer
Nuclear Management Company
700 First Street
Hudson, WI 54016

SUBJECT: MONTICELLO - NRC INSPECTION REPORT 50-263/2000016(DRS)

Dear Mr. Wadley:

On September 18 through September 22, 2000, the NRC completed a routine inspection at your Monticello Nuclear Generating Plant. The enclosed report presents the results of that inspection, which were discussed on September 22, 2000, with Mr. B. Day and other members of your staff.

The inspection was an examination of activities conducted under your license as they relate to radiation safety and to compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection consisted of a selective examination of procedures and representative records, observations of activities, and interviews with personnel. Specifically, this inspection focused on the implementation of your radiation monitoring instrumentation program.

Based on the results of this inspection, there were no findings identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the *Publicly Available Records (PARS) component of NRC's document system (ADAMS)*. *ADAMS is accessible from* the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

We will gladly discuss any question you have concerning this inspection.

Sincerely,

/RA/

Gary L. Shear, Chief
Plant Support Branch
Division of Reactor Safety

Docket No. 50-263
License No. DPR-22

Enclosure: Inspection Report 50-263/2000016(DRS)

cc w/encl: Site General Manager, Monticello
Plant Manager, Monticello
J. Malcolm, Commissioner, Minnesota
Department of Health

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Department of Health

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket No:	50-263
License No:	DPR-22
Report No:	50-263/2000016(DRS)
Licensee:	Northern States Generating Company
Facility:	Monticello Nuclear Power Plant
Location:	2807 West Highway 75 Monticello, MN 55362
Dates:	September 18-22, 2000
Inspector:	M. Mitchell, Radiation Specialist
Approved by:	Gary L. Shear, Chief Plant Support Branch Division of Reactor Safety

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas) reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety	Radiation Safety	Safeguards
● Initiating Events	● Occupational	● Physical Protection
● Mitigating Systems	● Public	
● Barrier Integrity		
● Emergency Preparedness		

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: http://www.nrc.gov/NRR/OVERSIGHT/index.html

SUMMARY OF FINDINGS

IR 50-263/2000016(DRS), on 09/18-09/22/2000; Nuclear Management Company; Monticello Nuclear Generating Plant. Radiation safety specialist report.

The inspection was conducted by a regional radiation specialist. There were no findings identified during this inspection.

Report Details

Summary of Plant Status: The reactor operated at or near 100 percent power throughout the inspection period.

2. RADIATION SAFETY

Cornerstone: Occupational Radiation Safety

2OS1 Access Control

.1 Plant Walkdowns and Radiological Boundary Verifications

b. Inspection Scope

The inspector performed walkdowns of the radiologically controlled area (RCA) to verify the adequacy of radiological boundaries and postings. Specifically, the inspector performed confirmatory radiation measurements in the Reactor and Turbine Buildings to verify that radiologically significant work areas were properly posted and controlled.

c. Findings

There were no findings identified.

2OS3 Radiation Monitoring Instrumentation

.1 Source Tests and Calibration of Radiological Instrumentation

a. Inspection Scope

The inspector verified that area radiation monitors' (ARMs) locations were as described in the Updated Safety Analysis Report and that selected ARMs were appropriately calibrated in 1999 and 2000. The inspector reviewed calibration records for the whole body counter, selected Friskall personnel contamination monitors (PCMs), selected portable radiation survey instruments and selected continuous air monitors for 1999 and 2000 to assess timely compliance with required calibrations. The inspector observed source checks of selected PCMs, ARMs, tool monitors, an alpha monitor, beta-gamma wipe test monitors, and portable radiation survey instruments to verify compliance with and adequacy of the procedures. The inspector reviewed the calibration and source check procedures with staff that would conduct the calibrations of the whole body counters to determine compliance with written procedures.

b. Findings

There were no findings identified.

.2 Radiation Protection Technician Instrument Use

a. Inspection Scope

The inspector observed several Radiation Protection (RP) technicians' selection and operational checks of portable radiation survey instruments used for RP technician job coverage to assess proper instrument selection and use. The inspector observed instrument use associated with clearing tools and equipment from RCA access control to assess RP technician instrument use during release of equipment for unrestricted use.

b. Findings

There were no findings identified.

.3 Self-Contained Breathing Apparatus Program

a. Inspection Scope

The inspector verified the adequacy of the program to provide Self-Contained Breathing Apparatus (SCBA) for unknown or emerging conditions. The inspector walked down the available equipment, reviewed the status and surveillance records of SCBA staged for use in the plant, verified the licensee's capability for refilling and transporting SCBA bottles to the control room and support locations in the plant to determine the licensee's capability to support the control room during emergency conditions. The inspector verified the training and qualification records of selected individuals in 2000, reviewed the licensees response to Information Notices 98-20 and 99-05 (no specific response was required by the notices) and interviewed control room operations staff regarding the use of SCBA in the control room to assess the understanding and readiness of SCBA users.

b. Findings

There were no findings identified.

.4 Identification and Resolution of Problems

a. Inspection Scope

The inspector reviewed the licensee's self-assessments and audits, which had been performed by the licensee during 1999 to verify that the staff conducts radiation monitoring instrument program assessments. The scope and findings of the licensee assessments were also reviewed to assure the adequacy and independence of the assessments. In addition, the inspector reviewed several Condition Report (CR) documents concerning radiological instrumentation initiated since July 1999 to verify proper implementation of the corrective action program. The inspector interviewed licensee staff regarding selected CRs to determine if the problem identification, characterization, disposition and evaluations resulted in appropriate licensee action.

b. Findings

There were no findings identified.

4. OTHER ACTIVITIES

4OA5 Management Meetings

Exit Meeting Summary

The inspector presented the inspection results to Mr. Byron Day and other members of licensee management and staff at the conclusion of the inspection on September 22, 2000. The licensee acknowledged the findings presented and did not identify any information discussed as proprietary.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

K. Bothun, Radiation Protection Coordinator
T. Corigan, Radiation Protection Technician
B. Day, Plant Manager
K. Jepson, Radiation Protection Supervisor
R. Lathum, Health Physics Supervisor
G. Mathiasen, Principle Health Physicist
M. Olson, Radiation Protection Technician
N. Olson, Radiation Protection Technician
D. Schmidt, Radiation Protection Technician
W. Shinnick, ALARA Coordinator
B. Weller, Radiation Protection Technician
J. Windschill, Radiation Protection Manager
P. Yurczyk, Special Projects Coordinator

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None

Closed

None

Discussed

None

LIST OF ACRONYMS USED

ADAMS	Agency Wide Documents Access and Management System
ARM	Area Radiation Monitor
CFR	Code of Federal Regulations
CR	Condition Report
DRS	Division of Reactor Safety
NRC	Nuclear Regulatory Commission
PARS	Publicly Available Records
PERR	Public Electronic Reading Room
PCM	Personnel Contamination Monitor
RCA	Radiologically Controlled Area
RP	Radiation Protection
SCBA	Self-Contained Breathing Apparatus

LIST OF DOCUMENTS REVIEWED

Audits, Assessments and Documents

NSP PMETS Operator Qualification Report
General Quality Services Report 1999040 (Revision 1), Radiation Protection Measuring and Test Equipment
MSA SCBA Functional Check (740L)
Updated Safety Analysis Report (Revision 18)

Condition Reports (CR) Nos.

CR-19991725, CR-19991791, CR-19991833, CR-19991863, CR-19991879, CR-19992400, CR-19992881, CR-19992956, CR-19993109, CR-19993297, CR-19993505, CR-19993506, CR-19993611, CR-20000180, CR-20001173, CR-20001173, CR-20002074, CR-20002555, CR-20002651, CR-20002884, CR-20003026, CR-20003112, CR-20003121, CR-20003651, CR-20003657, CR-20003661, CR-20003667, CR-20003668

Procedures

MNGP 0068 (Revision 15) Spent Fuel Pool and Reactor Building Exhaust Plenum Monitor Calibration
MNGP 0068 (Revision 16) Spent Fuel Pool and Reactor Building Exhaust Plenum Monitor Calibration
MNGP 0461 (Revision 4) Control Room Air Intake Monitor Calibration
MNGP 0461 (Revision 7) Control Room Air Intake Monitor Calibration
MNGP 1024 (Revision 22) Area Radiation Monitor Calibration
MNGP 1024 (Revision 23) Area Radiation Monitor Calibration
MNGP 1024 (Revision 24) Area Radiation Monitor Calibration
MNGP 1024 (Revision 25) Area Radiation Monitor Calibration
MNGP 1414 (Revision 1) Main Steam Line Radiation Monitor Test and Calibration
MNGP 3087 (Revision 29) Document Change and Hold Form
MNGP 5504 (Revision 4) Whole Body Counter Calibration
MNGP 5783 (Revision 7) DARM Functional Check
MNGP RPGP-05.06 (Revision 1) SCBA Bottle Filling Procedure
MNGP R.05.07 (Revision 10) SCBA Inspection and Functional Test
MNGP R.06.01 (Revision 12) Radioactive Source Control
MNGP R.09.01 (Revision 11) Fastscan Quality Assurance Calibration Check
MNGP R.09.20 (Revision 12) RO-2/RO-2A Tests
MNGP R.09.20 (Revision 13) Controlled Area Portal Alarm Functional Test and Posting Verification
MNGP R09.37 (Revision 11) NC Friskall Tests
MNGP R09.43 (Revision 3) PIOPS CAM Startup and Functional Check
MNGP R09.44 (Revision 2) NNC Waste Curie Monitor Operation and Tests
MNGP R.09.45 (Revision 5) Fastscan Calibration
MNGP R.09.47 (Revision 2) PIOPS CAM Calibration
MNGP R.09.49 (Revision 4) NNC Portal Monitor Tests
MNGP R.09.50 (Revision 8) DCA Area Radiation Monitor Tests
MNGP R.09.57 (Revision 3) Abacus Smear Counter Functional Checks
MNGP R.09.58 (Revision 8) Rad 51 and Rad 51T Calibration Verification Procedure
MNGP R.14.06 (Revision 7) Routine TLD Date Entry