March 3, 2005

Mr. Fred R. Dacimo
Site Vice President
Entergy Nuclear Operations, Inc.
Indian Point Energy Center
295 Broadway, Suite 1
P.O. Box 249
Buchanan, NY 10511-0249

SUBJECT: INDIAN POINT 3 NUCLEAR POWER PLANT - NRC TRIENNIAL FIRE PROTECTION INSPECTION REPORT 05000286/2005006

Dear Mr. Dacimo:

On January 28, 2005, the NRC completed a triennial fire protection team inspection at the Indian Point 3 nuclear power plant. The enclosed report documents the inspection findings which were discussed at an exit meeting on January 28, 2005, with Mr. C. Schwarz and other members of your staff.

This inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's regulations and with the conditions of your license. The purpose of the inspection was to evaluate your post-fire safe shutdown capability and fire protection program. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

Based on the results of this inspection no findings of significance were identified.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/reading-rm/ADAMS.html (the Public Electronic Reading Room).

Sincerely,

/RA/

John F. Rogge, Chief
Electrical and Fire Protection Branch
Division of Reactor Safety

Docket No. 50-286
License No. DPR-64

Enclosure: Inspection Report No. 05000286/2005006

cc w/encl:
G. J. Taylor, Chief Executive Officer, Entergy Operations, Inc.
M. R. Kansler, President - Entergy Nuclear Operations, Inc.
J. T. Herron, Senior Vice President and Chief Operations Officer
C. Schwarz, General Manager - Plant Operations
D. L. Pace, Vice President, Engineering
B. O'Grady, Vice President, Operations Support
J. McCann, Director, Licensing
C. D. Faison, Manager, Licensing, Entergy Nuclear Operations, Inc.
P. Conroy, Manager, Licensing, Entergy Nuclear Operations, Inc.
M. J. Colomb, Director of Oversight, Entergy Nuclear Operations, Inc.
J. Comiotes, Director, Nuclear Safety Assurance
J. M. Fulton, Assistant General Counsel, Entergy Nuclear Operations, Inc.
P. R. Smith, President, New York State Energy, Research and Development Authority
J. Spath, Program Director, New York State Energy Research and Development Authority
P. Eddy, Electric Division, New York State Department of Public Service
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D. O'Neill, Mayor, Village of Buchanan
J. G. Testa, Mayor, City of Peekskill
R. Albanese, Executive Chair, Four County Nuclear Safety Committee
S. Lousteau, Treasury Department, Entergy Services, Inc.
Chairman, Standing Committee on Energy, NYS Assembly
Chairman, Standing Committee on Environmental Conservation, NYS Assembly
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M. Slobodien, Director, Emergency Planning
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P. Rubin, Manager of Planning, Scheduling & Outage Services
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County Clerk, Westchester County Legislature
A. Spano, Westchester County Executive
R. Bondi, Putnam County Executive
C. Vanderhoef, Rockland County Executive
E. A. Diana, Orange County Executive
T. Judson, Central NY Citizens Awareness Network
M. Elie, Citizens Awareness Network
D. Lochbaum, Nuclear Safety Engineer, Union of Concerned Scientists
Public Citizen's Critical Mass Energy Project
M. Mariotte, Nuclear Information & Resources Service
F. Zalcman, Pace Law School, Energy Project
L. Puglisi, Supervisor, Town of Cortlandt
Congresswoman Sue W. Kelly
Congresswoman Nita Lowey
Senator Hillary Rodham Clinton
Senator Charles Schumer
J. Riccio, Greenpeace
A. Matthiessen, Executive Director, Riverkeeper, Inc.
M. Kaplowitz, Chairman of County Environment & Health Committee
A. Reynolds, Environmental Advocates
M. Jacobs, Director, Longview School

D. Katz, Executive Director, Citizens Awareness Network
P. Gunter, Nuclear Information & Resource Service
P. Leventhal, The Nuclear Control Institute
K. Coplan, Pace Environmental Litigation Clinic
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OFFICE	RI:DRS		RI:DRS		RI:DRP			
NAME	LScholl/***LLS***		JRogge/***JFR***		BMcDermott/***BJM***			
DATE	03/03/05		03/03/05		03/03/05			

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U. S. NUCLEAR REGULATORY COMMISSION

REGION I

Docket No.	50-286
License No.	DPR-64
Report No.	05000286/2005006
Licensee:	Entergy Nuclear Northeast
Facility:	Indian Point 3 Nuclear Power Plant
Location:	295 Broadway, Suite 1 Buchanan, NY 10511-0249
Dates:	January 10, 2005 - January 28, 2005
Inspectors:	L. Scholl, Sr. Reactor Inspector, Division of Reactor Safety (DRS) T. O'Hara, Reactor Inspector, DRS K. Young, Reactor Inspector, DRS T. Sicola, Reactor Inspector, DRS
Approved by:	John F. Rogge, Chief Electrical and Fire Protection Branch Division of Reactor Safety

Enclosure

SUMMARY OF FINDINGS

IR 05000286/2005006 on 01/10/2005 - 01/28/2005, Entergy Nuclear Northeast, Indian Point 3 Nuclear Power Plant. Fire Protection Team Inspection.

The report covered a two-week triennial fire protection team inspection by specialist inspectors. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

A. NRC-Identified Findings

No findings of significance were identified.

B. Licensee-Identified Violations

None

REPORT DETAILS

Background

This report presents the results of a triennial fire protection inspection conducted in accordance with NRC Inspection Procedure (IP) 71111.05T, “Fire Protection.” The objective of the inspection was to assess whether Entergy Nuclear Northeast has implemented an adequate fire protection program and that post-fire safe shutdown capabilities have been established and are being properly maintained at the Indian Point Nuclear Generating Unit 3 (IP3) facility. The following fire areas (FAs) and fire zones (FZs) were selected for detailed review based on risk insights from the IP3 Individual Plant Examination (IPE)/ Individual Plant Examination of External Events (IPEEE):

- FA CTL-3, Fire Zone 14
- FA CTL-3, Fire Zone 10
- FA TBL-5, Fire Zone 37A

1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems

1R05 Fire Protection

1. Fire Area Boundaries and Barriers

a. Inspection Scope

The team walked down accessible portions of the selected fire areas to observe material condition and the adequacy of design of fire area boundaries, fire doors, and fire dampers. The team reviewed engineering evaluations, as well as surveillance and functional test procedures for selected items. The team also reviewed the licensee submittals and NRC safety evaluation reports (SERs) associated with fire protection features at IP3. Additionally, the team reviewed the design and qualification testing of selected barriers and reviewed surveillance procedures for structural fire barriers and penetration seals. These reviews were performed to ensure that the passive fire barriers were properly maintained and met the licensing and design bases as described in the licensee submittals, NRC SERs, the fire hazards analysis (FHA) and the IP3 Final Safety Analysis Report (FSAR).

b. Findings

No findings of significance were identified.

2. Post-Fire Safe Shutdown Lighting and Communications

a. Inspection Scope

The team observed the placement and coverage area of eight-hour emergency lights throughout the selected fire areas to evaluate their adequacy for illuminating access and egress pathways and any equipment requiring local operation for post-fire safe shutdown. The team also reviewed preventive maintenance procedures and various documents, including the vendor manuals and completed surveillance tests, to determine if adequate surveillance testing and periodic battery replacements were in place to ensure reliable operation of the eight-hour emergency lights.

The team reviewed radio repeater location, power sources and preventive maintenance procedures to ensure fire department and operator communications could be maintained for fire fighting and post-fire safe shutdown conditions.

b. Findings

No findings of significance were identified.

3. Programmatic Controls

a. Inspection Scope

During tours of the facility, the team observed the material condition of fire protection systems and equipment, the storage of permanent and transient combustible materials, and control of ignition sources. The team also reviewed the procedures that controlled hot-work activities and combustibles at the site. Additionally, the team reviewed a sample of hot work permits and transient combustible evaluations. These reviews were accomplished to ensure that Entergy Nuclear Northeast was maintaining the fire protection systems, controlling hot-work activities, and controlling combustible materials in accordance with the FSAR, administrative procedures and other fire protection program procedures.

b. Findings

No findings of significance were identified.

4. Fire Detection Systems and Equipment

a. Inspection Scope

The team reviewed the adequacy of the fire detection systems in the selected plant fire areas. This included a walkdown of the systems and review of the type of installed detectors as shown on location drawings. The team also reviewed licensee submittals and the NRC SERs associated with the selected fire areas. These reviews were performed to ensure that the fire detection systems for the selected fire areas were

installed in accordance with the design and licensing bases of the plant. Additionally, the team reviewed fire detection surveillance procedures to determine the adequacy of the fire detection component testing and to ensure that the detection system would function as required.

b. Findings

No findings of significance were identified.

5. Fixed Fire Suppression Systems

a. Inspection Scope

Carbon Dioxide and Sprinkler Systems

The team reviewed the adequacy of the FA CTL-3, Fire Zone 14 and the FA CTL-3, FZ 10 automatic low pressure total flooding carbon dioxide (CO_2) systems by performing walkdowns of the systems and the fire area envelopes. The team also reviewed the design and installation to ensure it was in accordance with NFPA 12, "Standard on Carbon Dioxide Extinguishing Systems." Initial discharge tests that verified the adequacy of CO_2 concentration and hold times were reviewed. Completed surveillance procedures were also reviewed to ensure periodic testing of the systems was being accomplished. These reviews were performed to ensure that the low pressure total flooding CO_2 systems met the design and licensing bases as described in the licensee submittals, NRC SERs, the FHA and the FSAR and that the system could perform its intended function in the event of a fire in these areas.

The team reviewed the adequacy of the 31 EDG room automatic wet pipe sprinkler system by performing walkdowns of the system. The team also reviewed the design and installation to ensure it was in accordance with NFPA 13, "Standard for the Installation of Sprinkler Systems." Completed surveillance procedures were also reviewed to ensure appropriate periodic testing of the system was being accomplished. These reviews were performed to ensure that the sprinkler system met the design and licensing bases as described in the licensee submittals, NRC SERs and the FSAR and that the systems could perform their intended function in the event of a fire in these areas.

b. Findings

No findings of significance were identified.

6. Manual Fire Suppression Capability

a. Inspection Scope

The team walked down selected standpipe systems and observed portable extinguishers to determine the material condition of the manual fire fighting equipment and verify locations as specified in the pre-fire plans and fire protection program documents. The team reviewed electric and diesel fire pump flow and pressure tests to ensure that the pumps were meeting their design requirements. The team also reviewed the fire main loop flow tests to ensure that the flow distribution circuits were able to meet the design requirements. The team inspected the fire brigade's protective ensembles, self-contained breathing apparatus (SCBA), and various fire brigade equipment to determine operational readiness for fire fighting.

The team reviewed pre-fire plans and smoke removal plans for the selected fire areas to determine if appropriate information was provided to fire brigade members and plant operators to identify safe shutdown equipment and instrumentation, and to facilitate suppression of a fire that could impact post-fire safe shutdown.

The team performed in-plant walk downs to evaluate the physical configuration of electrical raceway and safe shutdown components in the selected fire areas to determine whether water from an inadvertent fire suppression system pipe rupture or from manual fire suppression activities in the selected areas could cause damage that could inhibit the ability to safely shutdown the plant.

The team reviewed fire brigade initial training and continuing training course materials to verify appropriate training was being conducted for the station fire fighting personnel. Additionally, the team reviewed selected fire drills and critiques to ensure that drills were being conducted in risk significant areas.

The team reviewed the qualifications of several fire brigade leaders and members to ensure that they had met and maintained the requirements to be fire brigade leaders and members.

b. Findings

No findings of significance were identified.

7. Safe Shutdown Capability

a. Inspection Scope

The team reviewed IP3's Fire Hazards Analysis (FHA), Safe Shutdown Analysis, and FSAR to determine the methods and equipment that Entergy Nuclear Northeast used to achieve safe shutdown following postulated fires. The team assessed the adequacy of the selected systems for reactivity control, reactor coolant makeup, reactor heat removal, process monitoring, and associated support system functions. In addition, the

team evaluated IP3's fire response procedures, alarm response procedures and operating procedures for the selected fire areas to assess the adequacy of methods and equipment used to achieve safe shutdown following a fire. The team's review included piping and instrumentation drawings (P&IDs) for post-fire safe shutdown systems to identify required components for establishing flow paths, to identify equipment required to isolate flow diversion paths, and to verify whether appropriate components were included in the alternate safe shutdown equipment list. The team performed field walkdowns to evaluate whether alternate safe shutdown equipment was adequately protected from the effects of fires.

The team evaluated selected safe shutdown components and their power and control circuits to verify that proper isolation and alternate power sources were provided for those components necessary for alternate shutdown in the event of a fire affecting the 480 Volt switchgear room. In addition, the team reviewed selected alternate shutdown equipment surveillance tests to verify periodic testing was adequate to demonstrate component operability and that the tests satisfied the applicable surveillance requirements.

Post-fire safe shutdown procedures were evaluated to determine if appropriate information was provided to plant operators to identify protected equipment and instrumentation and whether recovery actions specified in post-fire shutdown procedures considered manpower needs for performing required actions. The team also reviewed training lesson plans and job performance measures for shutdown actions, discussed training with licensed operators, reviewed the adequacy of shift manning, and evaluated the accessibility of the safe shutdown operating stations and required manual action locations.

Specific procedures reviewed for safe shutdown included:

- 3-AOP-SSD-1, Rev. 3, "Control Room Inaccessibility Safe Shutdown Control"
- 3-SOP-ESP-001, Rev. 12, “Local Operation of Safe Shutdown Equipment”
- SOP-EL-012, Rev. 17, “Operation of the Alternate Safe Shutdown Equipment”
- SOP-EL-013, Rev. 20, "Appendix R Diesel Generator Operation"

The team performed a walkdown of procedures for an evacuation and plant shutdown from outside the main control room. The walkdown was performed by an operations crew and focused primarily on the portion of the procedure associated with achieving stable hot shutdown conditions. Plant operators were accompanied by NRC team members during the walkdown and the approximate time for completing critical steps, such as establishing makeup flow to the reactor vessel, aligning the appendix R diesel generator to supply electrical power to safe shutdown loads and establishing seal injection for reactor coolant pump (RCP) seal cooling were noted and evaluated to assess the ability of the operators to maintain plant parameters within procedural limits.

b. Findings

No findings of significance were identified.

8. Safe Shutdown Circuits

a. Inspection Scope

The team reviewed cable routing for post-fire safe shutdown components to confirm that cables subject to fire damage in the selected fire areas were identified and adequately addressed. The team also reviewed cable raceway drawings for a sample of components required for post-fire safe shutdown to verify that cables were routed as described in the cable routing matrices.

Cable failure modes were reviewed for the following components: (1) valves PCV-455C and PCV-456 (PORV's), (2) head vent valves RCS-SOV-653, 653, 654 and 655, and (3) RHR isolation valves MOV-730 and 731.

The team reviewed circuit breaker coordination studies to ensure equipment needed to conduct post-fire safe shutdown activities would not be impacted by improper coordination. The team confirmed that coordination studies had addressed multiple faults due to fire. Additionally, the team reviewed a sample of circuit breaker maintenance procedures and testing records to verify that circuit breakers for components required for post-fire safe shutdown were properly maintained in accordance with procedural requirements.

The team reviewed the electrical isolation capability of selected equipment needed for post-fire safe shutdown to ensure that such equipment could be operated locally or from the alternate shutdown panels, if needed. The team also reviewed surveillance test procedures and test records for the alternate shutdown control transfer switches and alternate power supplies, to ensure that functionality of the transfer switches and alternate power sources were adequately demonstrated.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA2 Identification and Resolution of Problems

1. Corrective Actions for Fire Protection Deficiencies

a. Inspection Scope

The team reviewed the open corrective maintenance work orders for fire protection and safe shutdown equipment, selected condition reports (CRs) for fire protection and safe shutdown issues and recent fire protection system health reports to evaluate the prioritization of deficiencies and the effectiveness of corrective actions. The team also reviewed recent Quality Assurance (QA) Audits of the fire protection program to determine if Entergy Nuclear Northeast was identifying program deficiencies and implementing appropriate corrective actions.

b. Findings

No findings of significance were identified.

4OA6 Meetings, Including Exit

Exit Meeting Summary

The team presented their preliminary inspection results to Mr. C. Schwarz, Plant Manager, and other members of the IP3 staff at an exit meeting on January 28, 2005. No proprietary information was included in this inspection report.

ATTACHMENT: SUPPLEMENTAL INFORMATION

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee Personnel

J. Bencivegna, Mechanical Design Manager
F. Dacimo, Site Vice President
P. Coleman, Consultant (EPM)
P. Conroy, Licensing Manager
J. Comiotes, Director Nuclear Safety Assurance
G. Dahl, Licensing
K. Elliot, Fire Protection Engineer
C. Embry, Operations
T. Jones, Licensing
D. Leach, Director Engineering
T. Orlando, PCE Manager
S. Petrosi, Design System Manager
J. Pineda, Supervisor, Control Supervisor
R. Schimp, Engineer, Communications
C. Schwarz, General Manager - Plant Operations
S. VanBuren, Fire Chief
G. Vranjesevic, Electrical Engineer
S. Wilkie, Supervisor, Fire Protection

NRC

J. Rogge, Chief, Electrical and Fire Protection Branch, Division of Reactor Safety
T. Hipschman, Senior Resident Inspector, Indian Point Generating Unit 3
R. Berryman, Resident Inspector, Indian Point Generating Unit 3

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

NONE

Open and Closed

NONE

Closed

NONE

Discussed

NONE

LIST OF DOCUMENTS REVIEWED

Fire Protection Licensing Documents

Indian Point 3 Nuclear Power Plant, Final Safety Analysis Report (FSAR)
Indian Point 3 Technical Requirements Manual, Rev. 1
SER dated March 6, 1979, Fire Protection
SER dated April 21, 1983, Appendix R Fire Protection Review for The Indian Point Nuclear Generating Plant, Unit No. 3
SER "Alternate Safe Shutdown for Control Room Fire, 10 CFR 50, Appendix R, Section III.G and III.L for Indian Point 3," 9/9/88
SER Input For Appendix R To 10 CFR Part 50, Items III.G and III.L - Indian Point 3, 12/30/82
NRC Letter dated September 29,1978, Request for Additional Information & Staff Positions IP Unit 3
NRC Letter dated May 2, 1980, Supplement to the Fire Protection Safety Evaluation Report Indian Point Unit 3
NRC Letter dated January 7, 1987, Grants Exemptions to the Technical Requirements of Sections III.G. and III.J. of Appendix R to 10 CFR 50.
NYPA Letter dated August 4, 1977, Nuclear Fire Protection Functional Responsibilities, Administrative Controls & Quality Assurance, IP Units No.1, 2, and 3
NYPA Letter dated October 23, 1978, IP3 Nuclear Power Plant Fire Protection Review
NYPA Letter dated February 13, 1979, Response to NRC Request of 12/12/78 for Additional Information (Encl. 1) on Fire Protection Program Administrative Controls
NYPA Letter dated April 20, 1979, Revised Responses to NRC Request of 12/12/78 for Additional Information (Encl. 1) on Fire Protection Program Administrative Controls
NYPA Letter dated September 19, 1985, Information to Support the Evaluation of IP-3 to 10 CFR 50.48 and Appendix R to 10 CFR 50
NYPA Letter dated December 17, 1986, Change to Emergency Lighting Additions

Calculations/Engineering Evaluation Reports

IP-CALC-04-01171, Hydraulic Analysis of IP2 & IP3 Fire Protection Water Supply Systems and Several Unit 2 Suppression Systems and Unit 2 Standpipe Systems, Rev. 0
IP-CALC-FP-01981, Hydraulic Calculations For Standpipes, Rev. 0
DC-94-03-381FBAR, Manhole 33 Seal Installation, Rev. 0
ENG-10, Operational & Start-Up Test Procedure for Low Pressure CO_2 Fire Protection System, Rev. 1
ENG-527, Fire Barrier Inspections - Flamemastic Fire Stops Evaluation No. 10, September 16, 1993
IP3-RPT-CO_2-00777, Total Flooding Carbon Dioxide System Design Basis, Rev. 0
IP3-RPT-FP-00962, NFPA Codes of Record For NFPA Code Compliance Review, Rev. 0

IP-CALC-04-00766, IP3 SG Boil-Dry Analysis with RETRAN-3D, Rev. 0
IP3-ANAL-FP-01503, Rev. 2, 12/21/2000; Indian Point U3 10 CFR 50 Appendix R, Sections III.G and III.L, Safe Shutdown Analysis Report
IP3-CALC-MULT-382, Rev. 1, N2 Backup to Auxiliary Feedwater Bldg. Valves and Atmospheric Dump Valves
IP3-CALC-FP-00068, Rev. 2, Appendix R Diesel Generator Static Load Study
IP3-RPT-ED-00922, Rev. 2, Appendix R Diesel Generator System Evaluation
IP3-CALC-FP-01561, Rev. 0, Cable Requirements For The Implementation Of Appendix R Repair Procedure ELC-004-FIR
IP3-ANAL-FP-01264, Fire Barrier Analysis, Fire Doors Numbers 201, 203 and 205 Separating the Turbine Building From The Control Building, Rev. 0
IP3-ANAL-FP-01325, Fire Damper Assembly Analysis, Fire Dampers 4 and 9 Separating The Control Building From The Electrical Tunnel and The Turbine Building, Rev. 0
IP3-ANAL-FP-02143, Fire Hazards Analysis (FHA) Report, Rev. 4
IP3-DBD-321, Design Basis Document For Fire Protection, Rev. 2

Procedures

ONOP-FP-1, Rev. 17, Plant Fires
ELC-004-FIR, Rev. 9, Appendix R Repair - Cable Repairs
SOP-EL-14, Rev. 7, Energization of the 480 Vac Buses From the Appendix R Diesel Generator
3-SOP-EL-013, Rev. 20, Appendix R Diesel Generator Operation
3-SOP-EL-012, Rev. 17, Operation Of The Alternate Safe Shutdown Equipment
3-COL-RCS-1, Rev. 27, Reactor Coolant System
3-COL-RHR-1, Rev. 21, Residual Heat Removal System
ENG-533, Rev. 3 – Appendix 'R' Emergency Battery Lighting Area Blackout Test Procedure
3-AOP-SSD-1, Rev. 3, Control Room Inaccessibility Safe Shutdown Control
3-SOP-ESP-1, Rev. 12, Local Operation of Safe Shutdown Equipment
STR-002-SWS, Rev. 1, Main and Back-up Service Water Pump Strainer Manual Backwashing
AP-64.1, Rev. 2, Fire Protection/Appendix R Systems and Components Governed by TRM and TS
AP-16, Rev. 11, Quality Assurance Programs
ELC-018-GEN, Rev. 18, Inspection, Repair Replace and Semi-Annual Operability Testing of Appendix R Lighting Units
0-ELC-406-FIR, Rev. 0, Appendix R Light Units Test, Inspect, Repair, Replace
ENN-DC-127, Rev. 1, Control of Hot Work and Ignition Sources
FP-6, Rev. 10, Fire Brigade Membership Qualifications
FP-16, Rev. 8, Handling and Storage of Flammable and Combustible Liquids and Compressed Gas Cylinders
FP-19, Rev. 10, Fire Door Inspection
FP-22, Rev. 9, Fire Watch
IP-SMM-TQ-122, Rev. 0 & Rev. 1, Fire Protection Training Program
SNN-DC-901, Rev. 1, IPEC Fire Protection Program Plan
SOP-EL-013, Rev. 3, Appendix R Diesel Generator Operation
3-PT-2Y004, Rev. 1, CO_2 System Test For Cable Spreading and Switchgear Rooms
3-PT-2Y005, Rev. 0, CO_2 System Test For 31, 32 and 33 EDG Rooms
3-PT-A14, Rev. 13, Diesel Generator Sprinkler System

3-PT-A41, Rev. 0, Diesel Generator Fire Detection System Test
3-PT-M042A, Rev. 3, Electric Fire Pump Test
3-PT-M042B, Rev. 3, Diesel Fire Pump Test
3-PT-M80, Rev. 15, Monthly Emergency Battery Light Unit Functional Test
3-PT-M088, Rev. 7, CO_2 Fire Protection Flow Path Verification
3-PT-R102, Rev. 4, Fire Barrier Wrap/Radiant Shield Inspection
3PT-R082B, Rev. 5, Functional Test of Heat Detectors For Hazard Areas 1A, 1B, 1C, 7, and 8
3PT-R084, Rev. 16, Fire Pump Functional Test
3PT-R148, Rev. 8, Eight Hour Discharge Test For Appendix R Emergency Battery
3PT-SA13, Rev. 15, Fire Protection Smoke Detector Test
3PT-SA17, Rev. 11, Fire Protection Ultra-Violet Flame Detectors
3PT-SA27C, Rev. 0, Wet Pipe Sprinkler System #5, 6, and 7 Functional
3PT-R152, Rev. 7, Operability Test Of Safe Shutdown Instrumentation

Completed Tests/Surveillances

3PT-A41, Diesel Generator Building Fire Detection System Test, Rev. 0, (Completed February 20, 2004)
3-PT-M042A, Electric Fire Pump Test, Rev. 3, (Completed December 1, 2004)
3-PT-Q104, Appendix R Instrument Channel and Miscellaneous Equipment Checks, Rev. 9, (Completed December 29, 2004)
3-PT-R084, Fire Pump Functional Test, Rev. 16, (Completed May 13, 2004)
3-PT-R100, Fire Barrier Penetration Seal Inspection, Rev. 6, (Completed July 25, 2001)
3PT-R113, High Pressure Water Fire Protection System Flush and Loop Flow Determinations, Rev. 9, (Completed November 6, 2002 and November 8, 2002)
3PT-R152, Operability Test of Safe Shutdown Instrumentation, Rev. 7, (Completed March 2, 2003 and May 30, 2003)
3-PT-2Y014, Rev. 0, Appendix R DG Rated Load And Overspeed Test (Completed 3/12/04)
3PT-R090E, Rev. 10, 32 ABFP Local Operation Verification Test, (Completed 12/20/03)
3PT-R138, Rev. 3, Main Steam Atmospheric Dump Valves Backup N2 Supply, (Completed 4/14/03)
3PT-R198, Rev. 2, 32 ABFP Turbine Overspeed Trip, (Completed 4/14/03)
3PT-M090, Rev. 11, Appendix R DG Functional Test, (Completed 12/22/04)
3PT-Q104, Rev. 9, Appendix R Instrument Channel And Miscellaneous Equipment Checks, (Completed 12/29/04)
3PT-Q104, Rev. 8, Appendix R Instrument Channel And Miscellaneous Equipment Checks, (Completed 10/6/04)
3PT-R150, Rev. 3, Test of Appendix R Alternate Feed To Component Cooling Pump 32, (Completed 9/14/03)
3PT-M090, Rev. 9, Appendix R DG Functional Test, (Completed 4/20/02)
3PT-M090, Rev. 10, Appendix R DG Functional Test, (Completed 12/29/02)
3PT-M090, Rev. 11, Appendix R DG Functional Test, (Completed 12/17/04)
3PT-Q062A, Rev. 7, 31 Charging Pump Operability Test (Completed 11/19/04)
3PT-Q062B, Rev. 8, 32 Charging Pump Operability Test (Completed 12/17/04)
3PT-M099, Rev. 6, Safe Shutdown Instrument Channel Checks And Miscellaneous Equipment Surveillances, (Completed 12/12/02)
3-PT-R084, Rev. 16, Fire Pump (Electrical) Functional Test (Completed 10/8/04)

3-PT-R148, Rev. 8, 8 Hour Discharge Test for Appendix 'R' Emergency Battery Lighting (Completed 8/13/02)
3-PT-R100A, Rev. 0, Controlled Fire Inspection (Completed 3/15/95)
3-PT-R102, Rev. 4, Fire Barrier Wrap/Radiant Energy Shield Inspection (Completed 4/02/03)
3-PT-SA17, Rev. 11, Fire Protection Ultra Violet Flame Detectors (Completed 9/10/04)
3-PT-SA13, Rev. 15, Fire Protection System Smoke Detector Test (Completed 3/25/04)
3-PT-M80, Rev. 15, Monthly Emergency Battery Light Unit Function Test (Completed 12/03/04)
3-PT-MO23A, Rev. 3, Electrical Fire Pump Test (Completed 12/01/04)
3-PT-A23, Rev. 7, Balance of plant Conventional Fire Detection and Alarm System (Completed 9/21/04)
3-PT-A14, Rev. 13,Diesel Generator Sprinkler System (Completed 4/22/04)
3-PT-R082, Rev. 14, Functional Test of CO_2 System for Hazard Areas 1A, 1B, 1C, 7, 8, 9A, and 9B (Completed 9/04/03)
3-PT-A41, Rev. 0, Diesel Generator Building Fire Detector System Test (Completed 2/20/04)
3-PT-W012, Rev. 18, Appendix 'R' Diesel Support System Inspection (Completed 1/01/05)
3-PT-M090, Rev. 11, Appendix 'R' Diesel Functional test (Completed 12/12/04)
3-PT-M66, Rev. 21, Appendix 'R' Battery Inspection (Completed 12/13/04)
3-PT-R099, Rev. 8, Appendix 'R' Diesel Generator Rated Load and Overspeed Test (Completed 5/09/02)
3-PT-R103, Rev. 12, Appendix 'R' Diesel Battery Load Test (Completed 3/24/03)

Quality Assurance (QA) Audits and System Health Reports

A02-09I, IP3 Fire Protection Program
A03-12-I, IPEC Fire Protection Program
IPEC U3, Fire Protection System Health Report, 1st Quarter 2004
IPEC U3, Fire Protection System Health Report, 2nd Quarter 2004
IPEC U3, Fire Protection System Health Report, 3rd Quarter 2004

Drawings

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9321-F-96543, Rev. 0, Excore Neutron Flux Detector System Wiring Diagram
9321-LL-38334, Rev. 1, 118VAC Instrument Bus No. 32A Breaker No. 3 Circuit Schedule Electrical
9321-LL-30412, Rev. 1, 240/120VAC Distribution Panel Upper Electrical Tunnel
9321-F-33533, Rev. 7, Wiring Diagram And Miscellaneous Details Instrument Power Cabinet KH-4 Steam Gen. & Press. Instrument Isolation
9321-F-33501, Rev. 5, Miscellaneous Sections & Details Of Diesel Generator Isolation Cabinet, Switchgear Isolation Cabinets And Instrument Power Cabinets KH4 & POE
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Pre-Fire Plans

PFP-351, Unit 3, Control Bldg. - Switchgear Room EL. 15'-0", Rev. 0
PFP-354. Unit 3, Diesel Generator Bldg. - Diesel Generators 31, 32, and 33, EL. 15'-0", Rev. 0
PFP-362A, Unit 3, Turbine Bldg., 6.9 KV Switchgear Area, EL. 15'-0", Rev. 0

Fire Brigade Documents

IP3 2004 Fire Brigade Record, January 4, 2005

Unit 3 Fire Drills (Unannounced) and Critique

Team 3C, # 31 ABFP - 18'6" EL., conducted March 13, 2004
Team 3D, #31 ABFP - 18'6" EL., conducted March 13, 2004
Team 3B, #33 ABFP, Motor Leads Fire, July 17, 2004
Team 3E , 480 V Switchgear Room, August 20, 2004

Fire Brigade Training

Lesson I, The Fire Brigade
Lesson II, Science of Fire
Lesson III, Use and Selections of Portable Fire Extinguishers
Lesson IV, Fire Terms and Definitions
Lesson V, Protective Clothing and Turnout Gear
Lesson VI, Self-Contained Breathing Apparatus (SCBA)
Lesson VII, Fire Hose, Nozzles, and Water Appliances
Lesson VIII, Water and Foam Application
Lesson IX, Electricity and Fire
Lesson X, Tools and Forcible Entry
Lesson XI, Ventilation
Lesson XIV, Site Specific Training
Lesson XV, Fire fighting and Radiation

Operator Safe Shutdown Training

Job Performance Measure Exam (JPM) No. 002, Align City Water to the Charging Pumps, dated May 3, 2004
JPM No. 003, Transfer 32 Charging Pump to MCC-312A, dated May 3, 2004
JPM No. 005, Local Operation of the Atmospheric Steam Dumps, dated May 3, 2004
JPM No. 006, Isolate Steam to 32 Aux. Boiler Feed Pump, dated May 19, 2003
JPM No. 007, Locally Shut the MSIVs, dated May 3, 2004

JPM No. 008, Reset 32 Aux Boiler Feed Pump Trip, dated May 3, 2004
JPM No. 010, Manually Operate Auxiliary Boiler Feed Pump Feed Reg. Valve with Air, dated July 8, 2004
JPM No. 011, Locally Start 32 Aux Boiler Feed Pump, dated July 8, 2004
JPM No. 019, Energizing 38 Service Water Pump From MCC 312A, dated May 14, 2003
JPM No. 020, Start the Appendix-R Diesel Generator, dated July 8, 2004
JPM No. 021, Transfer 31 Charging Pump to MCC 312 A, dated July 8, 2004
JPM No. 027, Safe Shutdown From Outside the Control Room, dated May 11, 2004
JPM No. 050, Perform CRS Initial Responsibilities of ONOP-FP-1A, dated August 3, 2001
JPM No. 119, Transfer 32 CCW Pump to Alternate Feed (MCC-312A), dated May 3, 2004
JPM No. 121, Locally Charge to Maintain PZR Level/Press, dated September 3, 1999
Lesson Plan NIC-ONP-01, Safe Shutdown/Cooldown Outside CCR, Rev. 3
Lesson Plan LRQ-PRC-18, Safe Shutdown Outside CR, Rev. O4

Hot Work and Ignition Source Permits

Unit 3 TH 15' 33 Water Box, January 4, 2005
Unit 3 OSB Transfer Pump and Roof, December 2, 2004
Unit 2 Material Support Building, January 6, 2005
Unit 2 5' SW Pit, December 29, 2004
Unit 2 New Simulator, December 14, 2004
Unit 2 12 SAC, December 9, 2004
Unit 2 53' West Side 23A MSR, November 20, 2004

Transient Combustible Evaluations

#04-002
#04-005

Miscellaneous Documents

Modification P3-03-20457, Rev. 1, 3/1/04
Engineering Tracking List, January 25, 2005
Fire Protection Impairment Log, January 10, 2005
IP3 Internal Flood Analysis, Rev. 1
NFPA Code Compliance Review, Indian Point 3 Nuclear Power Plant, Stand Pipe and Hose Systems, April 1994
NFPA 12, Carbon Dioxide System - 1977 Edition
NFPA 13, Installation of Sprinkler Systems - 1976 Edition
NFPA 72E, Automatic Fire Detectors - 1974 Edition
Preventive Maintenance Overview Report, January 10, 2005
Technical Manual - Emergency Systems, Inc. - Big Beam
Technical Manual - Exide Emergency Lightguard
Transient Combustible Evaluation Log, May 2004 - December 2, 2004
Unit 3 Appendix R Emergency Light Survey
Teledyne Big Beam, Model 2S6L100-80
Dual-Lite, Model AS-210I-INDP

Exide/Lightguard, Model B-200
Exide/Lightguard, Model N101
Work Order Tracking List, January 25, 2005
Memorandum IP-OPS-95-139, 3/28/95; Feedback From Appendix R Training - March 20-25 and 28, 1995 (IP3 ASSD Timeline)
Westinghouse Owner's Group Emergency Response Guidelines, September 1, 1983
IP3-ANAL-EML-01627, Rev. 1, Appendix 'R' Emergency Lighting Unit Review - Evaluation of Emergency Lighting Blackout Testing , dated 10/24/095
Entergy Quality Assurance Program Manual, Rev. 11
SMM-DC-901, Rev. 1, IPEC Site Management Manual
SWR – 78106-02-03 – Typical Time Curves For Type CO-8/CO-6 relays - Entergy Nuclear Northwest Relay Test Report
41-100.1B – Type CO Overcurrent relay – Class 1E Applications 11/99

Condition Reports

CR-IP3-2002-04716
CR-IP3-2003-01628
CR-IP3-2003-01724
CR-IP3-2003-03468
CR-IP3-2003-04717
CR-IP3-2003-05448
CR-IP3-2004-00052
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CR-IP3-2004-03592
CR-IP3-2004-03911
CR-IP3-2004-03984
CR-IP3-2004-04005
CR-IP3-2004-09363
CR-IP3-2005-00114
CR-IP3-2005-00136
CR-IP3-2005-00190
CR-IP3-2005-00264
CR-IP3-2005-00272

Work Orders

IP-3-02-24313	IP3-04-01727	IP3-04-06260	IP3-04-20227
IP3-03-01514	IP3-04-04568	IP3-04-09260	IP3-04-20265
IP3-03-02684	IP3-04-04868	IP3-04-15533	
IP3-03-14370	IP3-04-04677	IP3-04-18565	
IP3-03-23969	IP3-04-06138	IP3-04-18567	

LIST OF ACRONYMS USED

CFR	Code of Federal Regulations
CO_2	Carbon Dioxide
CR	Condition Report
DRS	Division of Reactor Safety
EDG	Emergency Diesel Generator
FA	Fire Area
FHA	Fire Hazards Analysis
FSAR	Final Safety Analysis Report
FZ	Fire Zone
IP	Inspection Procedure
IPE	Individual Plant Examination
IPEEE	Individual Plant Examination of External Events
IP3	Indian Point Nuclear Generating Unit 3
IR	Inspection Report
NFPA	National Fire Protection Association
NRC	Nuclear Regulatory Commission
PAR	Publicly Available Records
P&ID	Piping and Instrumentation Drawing
PORV	Power Operated Relief Valves
QA	Quality Assurance
RCP	Reactor Coolant Pump
RHR	Residual Heat Removal
SCBA	Self-Contained Breathing Apparatus
SER	Safety Evaluation Report
TRM	Technical Requirements Manual