# 94

**Twenty-fourth Annual Report** 

Radiation Exposures for DOE and DOE Contractor Employees - 1991

**November 1994** 

Special Topic: New Dose Reporting Quantities II



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### TWENTY-FOURTH ANNUAL REPORT

# RADIATION EXPOSURES FOR DOE AND DOE CONTRACTOR EMPLOYEES - 1991

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### **FOREWORD**

This is the 24th in a series of annual radiation exposure reports published by the Department of Energy (DOE) and its predecessor agencies. This report summarizes the radiation exposures received by both employees and visitors at DOE and DOE contractor facilities during 1991. Trends in radiation exposures are evaluated by comparing the doses received in 1991 to those received in previous years. The significance of the doses is addressed by comparing them to the DOE limits and by correlating the doses to health risks based on risk estimates from expert groups.

This report is the fourth that is based on detailed exposure data for each individual monitored at a DOE facility. Prior to 1988, only summarized data from each facility were available. This report contains information on different types of radiation doses, including total effective, internal, penetrating, shallow, neutron, and extremity doses. It also contains analysis of exposures by age, sex, and occupation of the exposed individuals. This report also continues the precedent established in the Twenty-First (1988) Annual Report by conducting a detailed, one-time review and analysis of a particular topic of interest. The special topic for this report is a comparison of occupational radiation exposure health risks for various groups of the DOE workforce to health risks for the general U.S. population and workers in other occupations.

We believe this report will provide useful data to organizations or individuals involved in radiation protection activities. National and international organizations such as the National Council on Radiation Protection and Measurements, the International Commission on Radiological Protection, and the United Nations Scientific Committee on the Effects of Atomic Radiation have used DOE radiation exposure data in the past in formulating their recommendations and analyses. The information in these reports is also used by the DOE to identify areas of needed improvement to ensure continued commitment to the as low as reasonably achievable (ALARA) philosophy of radiation protection.

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### **PREFACE**

This report is one of a series of annual reports provided by the U.S. Department of Energy (DOE) summarizing occupational radiation exposures received by DOE and DOE contractor employees. These reports provide an overview of radiation exposures received each year and identify trends in exposures being experienced over the years.

Beginning with this report, Appendix D, "Exposure Data by Dose Range, Exposure Type, Facility Type, Age, Sex, and Occupation for DOE and DOE Contractor Employees and Visitors," is no longer included. Due to additional radiation dose reporting categories required by DOE order 5484.1, and the data comparisons provided in Appendix D, the resultant size of the annual report and associated publication costs necessitated this change. A copy of Appendix D is, however, available upon request.

In January 1975, with the separation of the AEC into the Energy Research and Development Administration (ERDA) and the U.S. Nuclear Regulatory Commission (NRC), each agency assumed responsibility for collecting and maintaining occupational radiation exposure information reported by the facilities under its jurisdiction. Former AEC licensees reported to the NRC while contractors reported to ERDA. At the same time, a contract was established with Union Carbide Corporation at Oak Ridge, Tennessee, to computerize the reporting and processing of both the ERDA and NRC radiation exposure reporting systems. On October 1, 1977, DOE was formed and assumed the responsibilities of ERDA. Processing and programming of exposure information continued at Oak Ridge until October 1978, when management and further development of the DOE radiation exposure reporting system was assigned to the System Safety Development Center, EG&G Idaho, Inc.; the NRC system remained at Oak Ridge.

Radiation exposure data for ERDA and ERDA contractor employees and visitors for 1974 through 1976 were reported in ERDA 76/119, ERDA 77-29, and DOE/EV-0011/9. The DOE and DOE contractor radiation exposure data for 1977-1979 were presented in DOE/EV-0066/10, 11, and 12, respectively. A revised version of the 1979 report was issued as DOE/EP-0039. The data for 1980-1982 were presented in DOE/EP-0040, DOE/EP-0040/1, and DOE/EP-0040/2. The data for 1983-1990 were presented in DOE/PE-0072, DOE/EH-0011, DOE/EH-0036, DOE/EH-0069,

DOE/EH-0128, DOE/EH-0171P, DOE/EH-0286P, and DOE/EH-0287P, respectively. This report contains 1991 radiation exposure data for DOE and DOE contractor employees and visitors.

Previous reports for AEC/ERDA/DOE government and contractor employees and visitors may be obtained from the DOE Office of Scientific and Technical Information, P.O. Box 62, Oak Ridge, TN 37830.

### **SUMMARY**

All U.S. Department of Energy and DOE contractors are required by DOE Order 5484.1, Chapter IV, to submit occupational radiation exposure records to a central depository. For 1991, data were required to be submitted for all employees who were required to be monitored in accordance with DOE Order 5480.11 and for all visitors who received a measurable dose. The data required included the total effective dose equivalent, external penetrating whole-body dose equivalent, internal dose equivalent, the shallow dose equivalent, neutron dose equivalent, and extremity dose equivalent. Data regarding the exposed individuals included the individual's age, sex, and occupation category. This report is a summary of data reported by DOE and DOE contractors for the calendar year 1991.

A total of 112,875 DOE and DOE contractor employees were reported to have been monitored for whole-body ionizing radiation exposure in 1991. This represents 61.5% of all DOE and DOE contractor employees and is an increase (13.5%) from the number of monitored employees for 1990. In addition to employees, 11,827 visitors were monitored. (For more information, see Table 4.1.)

Of all monitored employees reported, 72.9% received a total effective dose equivalent that was less than measurable, 26.9% received a dose equivalent between measurable and 1 rem (10 mSv), and 0.2% received a dose equivalent greater than 1 rem (10 mSv). Although no employee received a penetrating dose equivalent greater than 2 rem (20 mSv), 45 did receive a total effective dose equivalent greater than 2 rem (20 mSv). The total effective dose equivalent received by 62.4% of the visitors to DOE facilities was less than measurable, 36.8% received a dose equivalent between measurable and 1 rem (10 mSv), and 0.8% received a dose equivalent greater than 1 rem (10 mSv). There were eight visitors who received a total effective dose equivalent greater than 2 rem (20 mSv). (These data are detailed in Table 4.1.)

The collective dose equivalent for DOE and DOE contractor employees in 1991 was 2,491 person-rem (24.91 person-Sv), which represents a decrease of 12.7% from 1990. The collective dose equivalent for visitors was 453 person-rem (4.53 person-Sv), which represents a decrease of 45%. The average total effective dose equivalent for all monitored employees reported was 22 mrem (0.22 mSv), and the average dose equivalent for all employees reported who received a measurable exposure was 82 mrem (0.82 mSv). The average dose equivalent for all monitored individuals

(employees and visitors) reported was 24 mrem (0.24 mSv), and the average dose equivalent for all individuals reported who received a measurable exposure was 84 mrem (0.84 mSv). Activities at weapons fabrication and testing facilities resulted in the highest average dose equivalent of 50 mrem (0.50 mSv) for all monitored DOE and DOE contractor employees. The lowest average dose equivalent (1 mrem (0.01 mSv)) was received at DOE offices. These averages are significantly less than the DOE 5 rem/yr (50 mSv/yr) radiation protection standard for whole-body exposures.

Of the ten occupation categories reported (not including those classified as "unknown"), production workers received both the highest collective dose equivalent (537 person-rem (5.37 person-Sv)) and the highest average dose equivalent per individual who received a measurable exposure (115 mrem (1.15 mSv)). Agricultural workers received both the lowest collective dose (<1 person-rem (0.01 person-Sv)) and the lowest average dose equivalent (<1 mrem (<0.01 mSv)) per individual who received a measurable exposure.

The 5-year age group receiving the highest collective dose equivalent (450 person-rem (4.50 person-Sv)) was the 35-to-39 age group. The  $\geq 65$  age group had the highest average dose equivalent of 288 mrem (2.88 mSv) per individual who received a measurable exposure. The group receiving the lowest collective dose equivalent and average dose equivalent per individual who received a measurable exposure was the  $\leq 19$  age group.

The average dose for all males who received a measurable exposure was 89 mrem (0.89 mSv); for females, the average was 57 mrem (0.57 mSv). Males received a total of 2,634 person-rem (26.34 person-Sv), while females received 269 person-rem (2.69 person-Sv). A total of 41 person-rem (0.41 person-Sv) was received by individuals for whom sex was not specified on the report forms.

Of the 2,944 person-rem (29.44 person-Sv) received by DOE and DOE contractor employees and visitors at DOE facilities, 1,737 person-rem (17.37 person-Sv (59%)) was attributable to beta-gamma exposures, 343 person-rem (3.43 person-Sv (12%)) was attributable to neutron exposures and 839 person-rem (8.39 person-Sv (~29%)) was attributable to internal exposures. In addition to the penetrating dose equivalent (beta-gamma and neutron), DOE and DOE contractor employees and visitors received a collective shallow dose of 2,643 person-rem (26.43 person-Sv).

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### 1.0 INTRODUCTION

The purpose of this report is to disseminate information regarding radiation exposures received at U.S. Department of Energy (DOE) and DOE contractor facilities. At these facilities, dose equivalents received by both workers and visitors are carefully monitored and recorded. The primary purpose of this practice is to ensure that the DOE occupational dose limits are not exceeded and that as low as reasonably achievable (ALARA) goals are met. A secondary purpose, however, is to provide information that can be used by other organizations and individuals who wish to collect and analyze such information. This information may be useful for estimating the effect of changing dose limits on operations at DOE facilities, determining the progress of DOE with respect to the ALARA principle, or, in combination with other epidemiological data, assisting researchers in assessing the health-effect risks of low doses of ionizing radiation.

This report contains seven main sections and four appendices. Section 2.0 presents relevant DOE operating requirements including dose limits, ALARA, and reporting requirements. Section 3.0 presents brief descriptions of the various categories of DOE facilities and the sources of radiation exposure at each facility category.

Section 4.0 presents a summary of the radiation doses received at DOE and DOE contractor facilities in 1990. The data are presented according to dose-equivalent interval, facility type, field organization, occupation category, age, sex, and type of exposure (external penetrating, shallow, internal, etc.). The section concludes with an evaluation of recent exposure trends at DOE and DOE contractor facilities.

Section 5.0 presents a comparison of the doses received at DOE and DOE contractor facilities and the consequent risks relative to other risks that occur both in the workplace and as a part of everyday life. Section 6.0 presents reporting requirements for radiation exposure incidents at DOE and DOE contractor facilities. The magnitude of the postulated health effects from radiation doses received at DOE facilities is discussed in Section 7.0 of this report. Section 8.0 lists the references cited in this report.

Three appendices are included in the report, all of which contain detailed exposure data for DOE and DOE contractor employees and visitors. Appendix A presents the 1991 distribution of total effective dose equivalents by facility type for each DOE field organization. Appendix B presents the 1991 distribution of total effective dose equivalents by contractor for each DOE field organization. Appendix C presents the 1991 distribution of total effective dose equivalents by DOE field organization for DOE government employees and visitors.

Comments or suggestions that would improve the report or make it more useful should be sent to the U.S. Department of Energy, Assistant Secretary for Environment, Safety, and Health, Washington, D.C. 20585.

### 2.0 OPERATING REQUIREMENTS

One of the primary objectives of the DOE is to ensure that all its operations and those of DOE contractors are conducted safely. To help achieve this objective, the DOE has established radiation protection standards and program requirements to protect workers from ionizing radiation. The basic DOE standards are radiation dose limits, which establish maximum permissible doses to workers. In addition to the requirement that radiation doses to workers be maintained below the limits, it is the Department's policy that doses be maintained as far below the limits as is reasonably achievable.

### 2.1 DOSE LIMITS

In order to ensure that workers at DOE facilities are adequately protected from ionizing radiation, the DOE promulgates radiation protection standards for occupational workers. These standards include radiation dose limits to protect workers from both external radiation and internally deposited radionuclides. Radiation dose limits in effect for 1991 were promulgated January 1, 1989, in DOE Order 5480.11. This order included limits on annual dose equivalents to the whole-body and to individual organs (Table 2.1). Personnel monitoring in 1991 was required by DOE Order 5480.11 when the potential existed for an individual to receive an annual effective dose equivalent above 100 mrem (1 mSv), or an annual dose equivalent to an individual organ greater than 10% of the occupational radiation exposure limits shown in Table 2.1. Depending on the administrative policy of the field organization or contractor, monitoring may also have been provided to some or all individuals, such as clerical workers, for whom the exposure potential is extremely low.

The DOE radiation protection standards are based on the Environmental Protection Agency's (EPA's) revised guidance to federal agencies for protection against occupational radiation exposure (EPA 1987). This guidance was a result of a review by EPA of the 1976 recommendations of the International Commission on Radiological Protection (ICRP) and the National Council on Radiation Protection and Measurements (NCRP). The primary new feature of the guidance is that weighted internal doses are added to external doses to determine total effective dose equivalent. In the past, these were limited separately. The DOE became the first federal agency to implement the revised guidance when it promulgated its revised radiation protection standards (DOE Order 5480.11) for occupational workers on January 1, 1989.

**TABLE 2.1.** DOE Limiting Values for Assessed Dose from Exposure of Occupational Workers to Radiation (effective January 1, 1989)

Exposure Category	Limit
Total effective dose equivalent	5 rem/yr (effective dose equivalent)
Lens of eye	15 rem/yr (dose equivalent)
Extremity	50 rem/yr (dose equivalent)
Skin of the whole body	50 rem/yr (dose equivalent)
Other organ or tissue	50 rem/yr (dose equivalent)
Unborn child	0.5 rem/gestation period (dose equivalent)

### 2.2 ALARA PRINCIPLE

It has long been DOE's policy that radiation exposures should be maintained as far below the dose limits as is reasonably achievable. This policy, known as the ALARA principle of radiation protection, maintains that radiation exposures should be maintained as low as reasonably achievable, economic and social factors being taken into account (ICRP 1977).

The ALARA principle is based on the hypothesis that even very low radiation doses carry some risk. As a result, it is not enough to maintain doses at or slightly below the limits; the lower the doses, the lower the risks. Because it is not possible to reduce all doses at DOE facilities to zero, economic and social factors must be considered to determine the optimal level of radiation doses. According to the ALARA principle, if doses are too high, resources should be well spent to reduce them. At some point, the resources being spent to maintain low doses are exactly balanced by the risks avoided. Reducing doses below this point results in a misallocation of resources; the resources could be spent elsewhere and have a greater impact on health and safety.

To ensure that doses are maintained ALARA at DOE facilities, the DOE has mandated that ALARA plans and procedures be implemented and documented. To help ensure that facilities meet this requirement, the DOE has developed a manual of good practices for reducing exposures to ALARA levels (Munson et al. 1988). These include guidelines for administration of ALARA programs, techniques for performing ALARA calculations based on cost-benefit principles, guidelines for setting

and evaluating ALARA goals, and methods for incorporating ALARA criteria into both radiological design and operations. The establishment of ALARA as a required practice at DOE facilities demonstrates DOE's commitment to ensure minimum risk to workers from the operation of its facilities.

### 2.3 REPORTING REQUIREMENTS

In 1987, the DOE promulgated revised reporting requirements in DOE Order 5484.1 (DOE 1987). Formerly, contractors were required to report only the number of individuals who received an occupational whole-body exposure in one of 16 dose-equivalent ranges. However, contractors are required by the revised Order to report exposure data for individual employees and visitors. Data required include total effective dose equivalent, external penetrating dose equivalent (including neutron), internal effective dose equivalent, shallow dose equivalent, and extremity dose equivalent. Other data required include the individual's age, sex, employment status, and occupation, as well as the relevant organization and facility type.

### 3.0 FACILITY DESCRIPTIONS

DOE Order 5484.1 requires contractors to indicate for each reported individual the facility contributing the predominant portion of individual's effective dose equivalent. In cases when this cannot be distinguished, the facility indicated should represent the facility wherein the greatest portion of work service was performed.

The facility indicated must be one of eleven general facility categories: accelerator, fuel/uranium enrichment, fuel fabrication, fuel processing, maintenance and support (site-wide), reactor, general research, fusion research, waste processing/management, weapons fabrication and testing, and other. Because it is not always a straightforward procedure to determine the appropriate facility type for each individual, the assignment of an individual to a particular facility type is a policy decision of each contractor.

The facility descriptions that follow indicate the types of facilities included in each category. Also included are the types of work performed at the facilities and the sources of the majority of the radiation exposures.

### 3.1 ACCELERATOR

The DOE administers approximately a dozen laboratories that perform significant accelerator-based research. The accelerators range in size from small single-room electrostatic devices to a four-mile circumference synchrotron, and their energies range from keV to TeV.

The differences in accelerator types, sizes, and energies result in differences in the radiation types and dose rates associated with the accelerator facilities. In general, radiation doses to employees at the facilities are attributable to neutrons and x-rays, as well as muons at some larger facilities. Dose rates inside the primary shielding can range up to 200 mrem/h as a result of x-ray production near some machine components. Outside the shielding, however, x-ray exposure rates are very low, and neutron dose rates are generally less than 5 mrem/h (0.05 mSv/h). Average annual doses at these facilities are slightly higher than the overall average for DOE; however, the collective dose is lower than the collective dose for most other DOE facility categories because of the relatively small number

of employees at accelerator facilities. Regarding internal exposures, tritium and short-lived airborne activation products exist at some accelerator facilities, although annual internal doses are generally quite low.

### 3.2 FUEL/URANIUM ENRICHMENT

The DOE involvement in the nuclear fuel cycle generally begins with uranium enrichment operations and facilities (Rich et al. 1988). The current method of enrichment is isotopic separation using the gaseous diffusion process, which involves diffusing uranium through a porous membrane and using the different molecular weights of the uranium isotopes to achieve separation.

Although current facility designs and physical controls result in low doses from internally deposited uranium, the primary radiological hazard is the potential for inhalation of airborne uranium (Rich et al. 1988). Because of the low specific activity of uranium, external dose rates are usually a few millirem per hour or less. Most of the external doses that are received are attributable to gamma exposures, although neutron exposures can occur, especially when work is performed near highly enriched uranium. Both the average and collective external doses at these facilities are among the lowest of any DOE facility category.

### 3.3 FUEL FABRICATION

Activities at fuel fabrication facilities involve the physical conversion of uranium compounds to usable forms, usually rod-shaped metal. Radiation exposures to personnel at these facilities are attributable almost entirely to gamma and beta radiation. However, beta radiation is considered the primary external radiation hazard because of high beta dose rates (up to several hundred mrad per hour) at the surface of uranium rods (Rich et al. 1988). For example, physical modification of uranium metal by various metalworking operations, such as machining and lathing operations, requires protection against beta radiation exposures to the skin, eyes, and extremities. Average external doses at fuel fabrication facilities are generally higher than at other types of DOE facilities; however, collective doses are relatively low because the number of employees is low. Internal doses from inhalation of uranium are kept very low.

### 3.4 FUEL PROCESSING

The DOE administers several facilities that reprocess spent reactor fuel. These facilities separate the plutonium produced in reactors for use in defense programs. They also separate the fission products and uranium; the fission products are normally designated as radioactive waste products, while the uranium can be refabricated for further use as fuel.

The very high radioactivity of fission products in spent nuclear fuel results in employees at fuel processing facilities consistently having among the highest average doses of any DOE facility type. However, the collective dose at these facilities is less significant because of the small total number of employees. Penetrating doses are attributable primarily to gamma photons, although some neutron exposures do occur. Skin and extremity doses from handling of samples are also significant, although only a few employees typically receive skin doses greater than 5 rem (50 mSv) per year. Strict controls are in place at fuel reprocessing facilities to prevent internal depositions; however, several measurable intakes typically occur per year. Plutonium isotopes represent the majority of the internal depositions, and annual effective dose equivalents from the depositions are typically less than 500 mrem (5 mSv).

### 3.5 MAINTENANCE AND SUPPORT

Most DOE sites have facilities dedicated to maintaining and supporting the site. In addition, some employees may be classified under this facility type if their main function is to provide site maintenance and support, even though they may not be located at a single facility dedicated to that purpose.

Because many maintenance and support activities at DOE sites do not involve work near sources of ionizing radiation, the average dose equivalent per monitored employee is typically among the lowest of any facility type. However, those employees who do perform work near radiation sources receive relatively high average annual doses, as is indicated by the relatively high average annual dose per employee who receives a measurable exposure. Also, collective doses are relatively high because there is a large number of these employees relative to the number classified under other facility types. The sources of ionizing radiation exposure are primarily gamma photons. However, variations in the

types of work performed and work locations result in exposures of all types, including exposures to beta particles, x-rays, neutrons, and airborne radioactivity.

### 3.6 REACTOR

The DOE and its predecessors have built and operated dozens of nuclear reactors since the mid-1940s. These facilities have included plutonium and tritium production reactors, prototype reactors for energy production, research reactors, reactors designed for special purposes such as production of medical radioisotopes, and reactors designed for the propulsion of naval vessels.

In 1989, many of the DOE reactors were not operating. As a result, personnel exposures at DOE reactor facilities were attributable primarily to gamma photons and beta particles from contaminated equipment and plant areas, spent reactor fuel, activated reactor components, and other areas containing fission or activation products encountered during plant maintenance and decommissioning operations. Neutron exposures do occur at operating reactors, although the resultant doses are a very small fraction of the collective penetrating doses. Gamma dose rates in some plant areas can be very high (up to several rems per hour), requiring extensive protective measures. The average and collective external doses relative to other facility types are highly dependent on the status of reactor operations. Inhalation of airborne radioactive material is a concern in some plant areas. However, protective measures, such as area ventilation or use of respiratory-protection equipment, result in low internal doses.

### 3.7 RESEARCH, GENERAL

The DOE contractors perform research at many DOE facilities, including all of the national laboratories. Research is performed in general areas including biology, biochemistry, health physics, materials science, environmental science, epidemiology, and many others. Research is also performed in more specific areas such as global warming, hazardous waste disposal, energy conservation, and energy production, just to name a few.

The wide variety of research being performed at DOE facilities results in a wide variety of radiological conditions at those facilities where ionizing radiation or radioactive materials are an

important part of the research. Depending on the research performed, personnel may be exposed to virtually any type of external radiation, including beta particles, gamma photons, x-rays, and neutrons, as well as the potential for inhalation of radioactive material. Area dose rates and individual annual doses are also highly variable. Relative to other facility types, average annual individual doses are slightly above average at general research facilities. The collective dose equivalent is higher than at most other facility types because of the many individuals employed at general research facilities.

### 3.8 RESEARCH, FUSION

The DOE currently operates on major and several smaller facilities that participate in research on fusion energy. In general, both penetrating and shallow radiation doses are minimal at these facilities because the dose rates near the equipment are both low and intermittent. The external doses that do occur are attributable primarily to x-rays from energized equipment. Relative to other DOE facility types, average individual doses and collective doses are typically the lowest at fusion research facilities. Regarding internal exposures, airborne tritium is a concern at some fusion research facilities, although the current level of operation results in minimal doses.

### 3.9 WASTE PROCESSING/MANAGEMENT

Most DOE sites have facilities dedicated to the processing and disposal of radioactive waste. In general, the dose rates to employees when handling waste are very low because of the low specific activities or the effectiveness of shielding materials. As a result, very few employees at these facilities receive annual doses greater than 100 mrem (1 mSv). At two DOE sites, however, large-scale waste processing facilities exist in order to properly dispose of radioactive waste products generated during the nuclear fuel cycle. At these facilities, radiation doses to some employees can be relatively high, sometimes exceeding 1 rem/yr (10 mSv/yr). Penetrating doses at waste processing facilities are mostly attributable to gamma photons; however, neutron exposures are significant at the large-scale facilities. Skin doses are generally not a significant problem. Overall average annual doses at waste processing/management facilities are among the highest of any DOE facility type, which is attributable primarily to the two large-scale facilities and the shift in DOE mission from national defense production to waste management and environmental restoration. The annual

collective doses are closer to the average of all facility types, however, because of the relatively small number of employees at this type of facility.

### 3.10 WEAPONS FABRICATION AND TESTING

The primary function of a facility in this category is to fabricate weapons-grade material for the production or testing of nuclear weapons. At the testing facilities, radiation doses received by personnel are generally minimal because of the strict controls over personnel access to testing areas, although extremity doses can be relatively high from handling neutron-activated materials. Radiation doses are a greater concern at facilities where weapons and weapons-grade nuclear material are handled. At these facilities, neutron radiation dose rates can be significant when processing relatively small quantities of <sup>238</sup>Pu or larger quantities of mixed plutonium isotopes (Faust et al. 1988). Penetrating doses from gamma photons and plutonium x-rays can also be significant in some situations, as can skin and extremity doses from plutonium x-rays. Overall, average individual annual doses at these facilities are slightly higher than the DOE average. The collective doses received by employees at these facilities are generally higher than the collective doses at other facility types because of the large number of individuals employed.

Also of significant concern at these facilities is inhalation of plutonium, where inhalation of very small amounts could result in doses exceeding limits. To prevent plutonium intakes, strict controls are in place including process containment, contamination control procedures, and air monitoring and bioassay programs (Faust et al. 1988). As a result, significant internal exposures are very rare at these facilities.

### **3.11 OTHER**

Individuals placed in this facility type can be generally classified under three categories: 1) those who worked in a facility that did not match one of the ten facility types described above; 2) those who did not work for any appreciable time at any specific facility, such as transient workers; or 3) those for whom facility type was not indicated on the report forms. Examples of a facility type not included in the ten described above include construction and irradiation facilities. In general, employees classified under this facility type receive annual doses significantly less than the annual doses averaged

over all DOE facilities. However, the wide variation in the type of work performed by these individuals results in a wide variation in the types and levels of exposures. Although exposures to gamma photons are predominant, some individuals may be exposed to beta particles, x-rays, neutrons, or airborne radioactive material.

### 4.0 SUMMARY OF IONIZING RADIATION DOSES

Monitoring in 1991 was required by DOE Order 5480.11 when the potential existed for an individual to receive an annual effective dose equivalent above 100 mrem (1 mSv), or an annual dose equivalent to individual organs above 10% of the exposure limits. Depending on the administrative policy of the contractor, monitoring may also have been provided to individuals, such as clerical workers, for whom the exposure potential is extremely low.

On November 6, 1987, DOE promulgated revised reporting requirements in DOE Order 5484.1, which affected the reporting of occupational doses received during 1987 and beyond. Before 1987, DOE contractors were required to report only the number of individuals who received an occupational whole-body exposure in one of 16 dose-equivalent intervals ranging from "less than measurable" to "greater than 10 rem." Contractors are now required, however, to submit detailed exposure data for individual employees who were monitored and for visitors who received a measurable exposure. (Contractors are also required to provide a count of the total number of visitors monitored.) Data now required to be submitted for each individual include total effective dose equivalent, external penetrating dose equivalent (including neutron), shallow dose equivalent, and extremity dose equivalent. This report is a summary of the dose equivalents received by DOE and DOE contractor employees and visitors in 1991 as reported pursuant to DOE Order 5484.1.

This report is the second to contain data on total effective dose equivalent, internal dose, and extremity dose for all DOE sites. In reports previous to 1990, the primary radiation quantity analyzed was whole-body penetrating dose. In this report, the primary quantity to be analyzed will be total effective dose equivalent. Caution should be used when comparing these data to those of past annual reports since the total effective dose quantity represent the total of the penetrating and internal dose components for employees and visitors. Data shown in tables and graphs for years previous to 1990 represent only the values for whole-body penetrating dose.

### 4.1 DISTRIBUTION BY DOSE INTERVAL

The number of employees and visitors who received a total effective dose equivalent in each of 16 dose-equivalent ranges is presented in Table 4.1. A total of 112,875 DOE and DOE contractor employees were reported to have been monitored for whole-body ionizing radiation exposure in 1991. This represents 61.5% of all DOE and DOE contractor employees. In addition to the employees, 11,827 visitors were monitored at DOE facilities. Visitors may include radiation workers from another DOE facility present on a temporary basis.

**TABLE 4.1.** Distribution of Total Effective Dose Equivalent for DOE/DOE Contractor Employees and Visitors by Dose-Equivalent Interval, 1991<sup>(a)</sup>

•	Number of Persons			Collective Person-rem		
Dose-Equivalent Interval (rem)	Employees	<u>Visitors</u>	<u>Total</u>	<u>Employees</u>	<u>Visitors</u>	<u>Total</u>
< Measurable	82,320	7,380	89,700	0	0	0
Measurable to 0.10	24,558	3,754	28,312	650	79	729
0.10 to 0.25	3,798	286	4,084	585	44	629
0.25 to 0.50	1,463	163	1,626	501	56	557
0.50 to 0.75	351	101	452	211	64	276
0.75 to 1.00	173	52	225	150	45	195
1 to 2	167	83	250	218	107	325
2 to 3	23	2	25	. 56	5	61
3 to 4	9	0	9	30	0	30
4 to 5	8	0	8	37	0	37
5 to 6	0	1	1	0	6	6
6 to 7	2	0	2	13	0	13
7 to 8	0	2	2	0	15	15
8 to 9	1	1	2	8	8	16
9 to 10	0	1	1	0	10	10
> 10	2	1	3	32	14	<u>47</u>
Total	112,875	11,827	124,702	2,491	453	2,944

<sup>(</sup>a) Minor variations in collective dose-equivalent values may be due to rounding.

No DOE or DOE contractor employee received a total effective dose equivalent greater than 5 rem (50 mSv) due to exposures received during 1991. There are five employees and six visitors, however, who did receive a total effective dose equivalent greater than 5 rem (50 mSv) because of past internal uptakes of radionuclides. Annual dose due to these past internal uptakes is calculated each year and is expressed in the values for total effective dose equivalent. No DOE or DOE contractor employee or visitor received a whole-body penetrating dose equivalent greater than 2 rem (20 mSv), which is significantly less than the DOE radiation protection standard of 5 rem (50 mSv) (See Table 4.2).

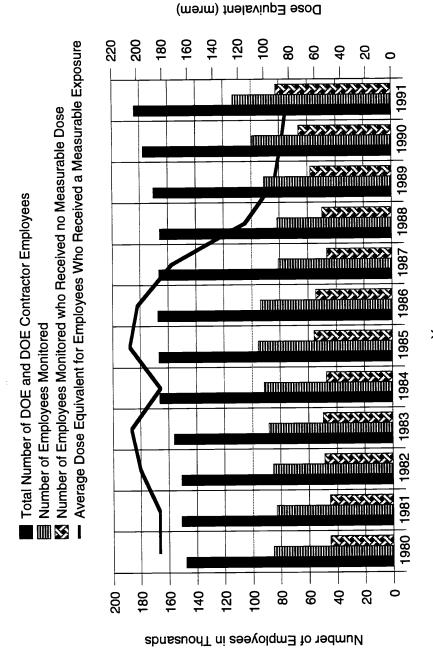
A comparison of the number of DOE and DOE contractor employees, the number of monitored employees reported, and the number of monitored employees reported who did not receive a measurable dose equivalent is presented for the years 1980-1991 in Figure 4.1. The figure also illustrates the average dose equivalent per employee who received a measurable exposure. The number of monitored employees reported for 1991 has increased from the number reported for previous years because of the greater number of DOE and DOE contractor employees involved in environmental remediation activities and because of the requirements of DOE Order 5480.11.

Of the monitored employees reported for 1991, 72.9% received a total effective dose equivalent that was less than measurable; 26.9% received a dose equivalent between measurable and 1 rem (10 mSv); and 0.2% received a dose equivalent greater than 1 rem (10 mSv) (Figure 4.2). The dose equivalent received by 62.4% of the visitors to DOE facilities was less than measurable; 36.8% received a dose equivalent between measurable and 1 rem (10 mSv); and 0.8% received a dose equivalent greater than 1 rem (10 mSv) (Figure 4.2).

**TABLE 4.2.** Distribution of Whole-Body Penetrating and Total Effective Dose Equivalents for DOE/DOE Contractor Employees, 1965-1991<sup>(a)</sup>

		Monitored	135,214	137,932	108,386	107,986	102,918	96,661	94,315	89,460	91,977	78,232	88,425	90,200	95,220	102,020	104,986	85,465	83,049	85,324	88,283	91,603	92,806	94,040	81,028	81,629	90,882	99,443	99,443	112,875	112,875
		<u>×12</u>		-					2						2		5											-	-	,	-
em)		11-12	2																					-				-	-		-
umber of Employees Receiving Radiation Doses in Each Dose-Equivalent Range (rem)		10-11	9		-		-		-																						
valent		9-10	22	2	4			-																							
ose-Equi		89	25	9	17								-		2								-					-	-		<b>-</b>
Each D		7-8	56	24	23			2							-																
ses in		7-9	32	47	59	-		4	က	2	-													-				2	2		2
tion Do		2-6	70	88	35	က	4	2	œ	∞	2	4		-			-											-	-		
ng Radia		4-5	294	313	168	144	98	158	118	92	09	40	142	9	23	11	10		'n	28	31	Ξ	œ	÷				∞	∞		∞
Receivir		3-4	515	593	555	425	335	279	275	219	172	149	232	70	103	23	33	16	53	26	49	31	21	35	36			∞	∞		∞
loyees		2-3	1.704	1,630	1,572	1,408	1,313	1,329	888	929	727	688	753	475	545	439	416	387	263	313	294	312	356	349	283	34	21	37	37		23
of Emp		1-2	4.158	3.706	3,472	2,799	2,554	2,698	2,380	2,130	1,944	1,667	1,846	1,679	1,579	1,323	1,286	1,113	296	1,010	1,270	1,226	1,366	1,298	1,258	505	437	191	191	74	167
Number		Meas1										32,500	42,141	47,886	49,948	55,296	52,235	38,895	36,561	34,949	36,768	42,696	38,085	37,774	32,939	31,260	32,891	32,896	33,896	26,739	30,343
	(p)	<meas.< td=""><td></td><td></td><td>102,510</td><td></td><td></td><td></td><td></td><td></td><td></td><td>43.1</td><td>43,310</td><td>40,083</td><td>43,017</td><td>44,898</td><td>50,003</td><td>45,054</td><td>45.224</td><td>48,968</td><td>49,871</td><td>47,327</td><td>55,939</td><td>54,581</td><td>46,512</td><td>49,833</td><td>57,533</td><td>66,297</td><td>66,297</td><td>86,062</td><td>82,320</td></meas.<>			102,510							43.1	43,310	40,083	43,017	44,898	50,003	45,054	45.224	48,968	49,871	47,327	55,939	54,581	46,512	49,833	57,533	66,297	66,297	86,062	82,320
		Year	1965	1966	1967	1968	1969	1970	1971	1972	1973	1974	1975	1976	1977	1978	1979	1980	1981	1982	1983	1984	1985	1986	1987	1988	1989	1990,	1990 <sup>(c)</sup>	1991,	1991 <sup>(C)</sup>

Throughout this report there may be minor variations in collective dose-equivalent values because of rounding. Separation of data before 1974 is unavailable. Data for total effective dose equivalent. (a) (c)



Comparison of Number of Employees, Number of Employees Monitored, and Number of Employees Monitored Who Received No Measurable Dose Equivalent, 1980-1991. (Data previous to 1990 is based on whole-body penetrating dose only; data since 1990 is based on total effective dose equivalent.) FIGURE 4.1.

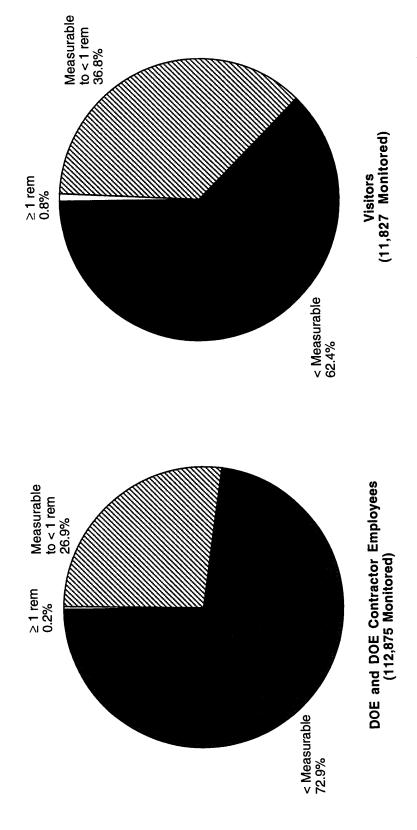


FIGURE 4.2. Percentage of Monitored Employees and Percentage of Monitored Visitors Who Received Total Effective Dose Equivalents Less than Measurable, Measurable to 1 rem, or Greater Than 1 rem, 1991

The total effective collective whole-body dose equivalent was 2,491 person-rem (24.91 person-Sv) for all DOE and DOE contractor employees, and 453 person-rem (4.53 person-Sv) for visitors to DOE facilities, for a total DOE collective dose equivalent of 2,944 person-rem (29.44 person-Sv). The contribution of the individuals (employees and visitors) in each dose-equivalent interval to the collective dose equivalent is shown in Figure 4.3. Individuals whose exposure was between measurable and 1 rem (10 mSv) contributed the greatest portion (81.0%) of the collective dose.

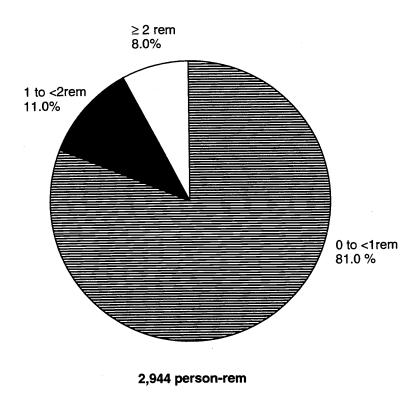


FIGURE 4.3. Contribution of Each Dose-Equivalent Interval to the Total Collective Dose Equivalent, 1991

The distribution of whole-body penetrating and total effective doses for DOE and DOE contractor employees for the years 1965-1991 is presented in Table 4.2. As indicated, the fraction of all monitored employees who received a penetrating dose equivalent greater than 1 rem (10 mSv) has declined dramatically since 1965, starting at about 5%, leveling off at about 2% from 1977 to 1987, and dropping to less than 1% for the period 1988-1991. This general downward trend in occupational radiation exposures can be observed in Figure 4.4, which shows the collective dose equivalent for employees who received a dose equivalent greater than 1 rem (10 mSv) from 1965 to 1991. The collective dose equivalent for employees who received an exposure less than 1 rem (10 mSv) was not included because, before 1974, less-than-measurable exposures were not distinguished from measurable exposures in the reporting system. The trend reflects both changes in the nature of the work performed at DOE facilities and the required application of ALARA practices throughout all DOE operations. The most recent decrease may be attributable in part to reduced operations and mission changes at some DOE facilities.

Analysis of occupational doses is commonly performed by fitting the data to a lognormal distribution (Brodsky et al. 1976; Brooks 1988). Figure 4.5 presents the 1991 data for DOE and DOE contractor employees on a lognormal probability plot. This figure is useful for indicating the fraction of employees whose dose equivalents exceed various values as well as the fraction of the collective dose equivalent that is attributable to various ranges of individual dose equivalent. For example, the figure indicates that although less than 1% of monitored DOE and DOE contractor employees received a dose equivalent greater than 1 rem (10 mSv), approximately 20% of the employee collective dose equivalent was attributable to individual dose equivalents greater than 1 rem (10 mSv).

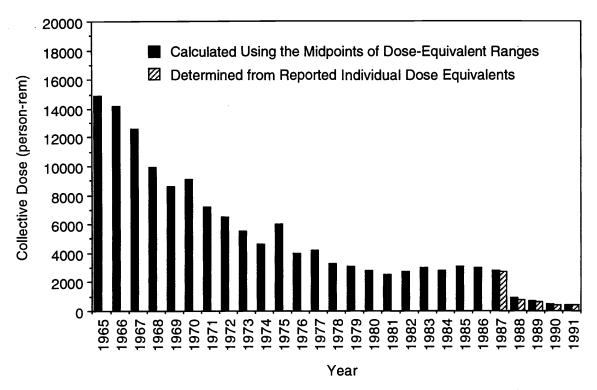


FIGURE 4.4. Total Collective Dose Equivalent for All DOE/DOE Contractor Employees Who Received a Dose Equivalent Greater Than 1 rem, 1965-1991

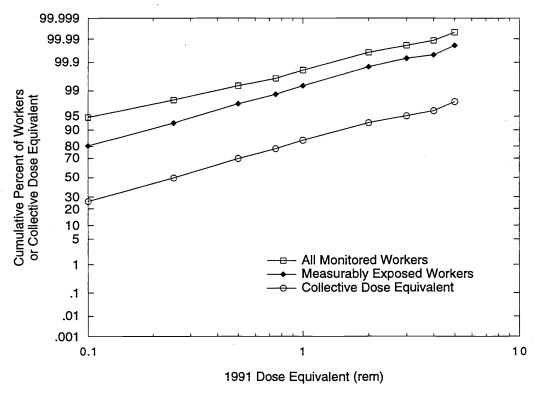


FIGURE 4.5. Lognormal Probability Plots of Annual Exposure for Potentially Exposed and Measurably Exposed DOE and DOE Contractor Employees, 1991

#### 4.2 DISTRIBUTION BY FACILITY TYPE

The number of individuals (employees and visitors) and the distribution of the annual whole-body dose equivalents in each of 11 facility categories were reported to the central repository. The assignment of exposures to one of the 11 facility types (listed in DOE Order 5484.1) is a policy decision of each field organization. For this section of the report, the categories of "visitors" and "DOE offices" were each considered a "facility type." The contribution of each facility type to the collective dose equivalent is shown in Figure 4.6. The largest percentage of the total collective dose equivalent (28.5%) was in the category "Weapons Fabrication and Testing." The smallest contribution (0.06%) was from DOE offices. A summary of the data is presented in Table 4.3.

Collective dose increased 13%, when compared with 1990 data, for the "Weapons Fabrication and Testing" category. This increase may be due to a larger penetrating dose component caused by an increased workload at weapons fabrication facilities (actually dismantling weapons). In addition,

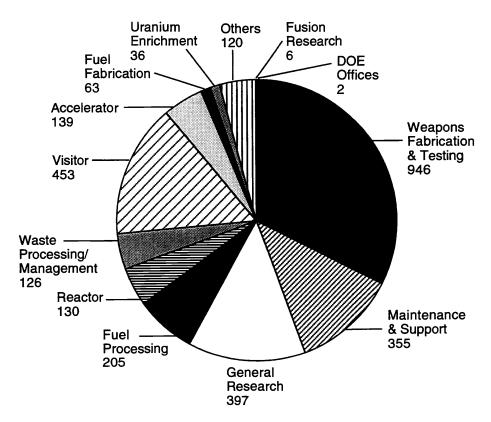


FIGURE 4.6. Contribution of Each Facility Type to the Total Collective Effective Dose Equivalent, 1991 (numbers indicate person-rem)

TABLE 4.3. Distribution of Annual Whole-Body Radiation Doses for Monitored DOE/DOE Contractor Employees and Visitors by Facility Type, 1991<sup>(a)</sup>

	Number of Persons Receiving Radiation Doses in Each Dose-Equivalent Range (rem) Meas 0.10- 0.25- 0.50- 0.75-	of Person	ons Rec 0.10-	ns Receivind 0.10- 0.25-	Radiation 0.50- 0.75	ation [ 0.75-	oses	n Ea	G G	ose-	Egui	vale	nt Ra	nge (r	em)	Total	Total Person-
Facility Type	< Meas.	<0.10		0.50	0.75	1.00	1-2	2-3	3-4	4-5 5	9-9	7 7-9	7-8 8-9	9 9-10	기	۱.	rem
Accelerator	5,294	1,063	186	84	53	6	ro								-	6,671	139
Fuel/Uran. Enrichment	7,623	1,647	32	7	1		1									9,311	36
Fuel Fabrication	992	277	74	15	-	3	1									1,363	59
Fuel Processing	3,334	1,063	341	179	48	16	10									4,991	205
Maint. and Support	19,069	5,026	612	201	34	15	23	4								24,984	355
Reactor	2,473	2,201	225	71	12	7	2			,						4,994	130
Research, General	16,091	3,193	490	206	84	47	48	-	-	-		-				20,163	397
Research, Fusion	996	158	တ													1,133	9
Waste Proc./Management	3,936	1,137	226	104	15	∞	4	2								5,432	126
Weapons Fab. & Test.	10,150	6,605 1,410	1,410	511	117	61	69	16	œ	9		-		_	-	18,956	946
Other	10,784	2,016	192	82	10	7	1			-						13,096	120
Visitors	7,380	3,754	286	163	101	52	83	2			-		2	1 1	-	11,827	453
DOE Offices	1,608	172	- !	1	!	1				1			; }		į	1,781	2
Total Persons	89,700	89,700 28,312 4,084 1,626	4,084	1,626	452	225	250	25	6	80	-	2	2	-	က	124,702	

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

a large portion of the "Weapons Fabrication and Testing" collective dose (approximately 64%) is due to internal dose caused by the uptake of internal emitters that occurred in previous reporting years.

Collective dose decreases of 29% and 41% were seen for the "Reactor" and "Fuel Processing" categories, respectively. These decreases were probably due to reduced activities in both of these production-related categories during 1991. Decreases in collective dose of 10% and 38% were also seen for the "General Research" and "Maintenance and Support" categories. These decreases, along with an overall decrease in total collective dose when compared with 1990, is likely due to ongoing efforts within the DOE community to follow the ALARA concept of radiation protection.

The average dose equivalent by facility type per individual monitored and per individual who received a measurable dose equivalent is shown in Table 4.4. The average dose equivalent per individual monitored for all facilities was 24 mrem (0.24 mSv). The highest average dose equivalent per individual monitored (50 mrem) (0.50 mSv) was observed at weapons fabrication and testing facilities, and the lowest was observed at DOE offices (1 mrem) (0.01 mSv). The average dose equivalent per individual who received a measurable dose equivalent was 84 mrem (0.84 mSv). The highest average dose equivalent per individual who received a measurable dose equivalent (124 mrem) (1.24 mSv) was observed at fuel processing facilities, and the lowest (13 mrem) (0.13 mSv) was observed at DOE offices.

## 4.3 DISTRIBUTION BY FIELD ORGANIZATION

For each field organization, the number of monitored individuals reported, the number of individuals who received a measurable dose equivalent, and the collective dose equivalent are shown in Table 4.5.

Differences in the collective dose equivalent at each field organization reflect differences in the number of employees at the facilities, the nature of the work performed, and the administrative policy concerning whether the dose distribution is reported for all monitored employees or only for those for whom monitoring is required. Table 4.6 provides an indication of the work performed at each field organization by showing the fraction of the collective dose equivalent attributed to each facility type

**TABLE 4.4.** Collective Dose-Equivalent for Monitored DOE/DOE Contractor Employees and Visitors by Facility Type, 1991<sup>(a)</sup>

Facility Type	Number of Individuals	Number of Individuals with Measurable Doses	Collective Dose-Equivalent (Person-rem)	Average Dose-Equivalent (mrem) per Individual	Average Dose-Equivalent (mrem) per Individual with Measurable Doses
Accelerator	6,671	1,377	139	21	101
Fuel/Uran. Enrichment	9,311	1,688	36	4	21
Fuel Fabrication	1,363	371	59	21	78
Fuel Processing	4,991	1,657	205	41	124
Maint. and Support	24,984	5,915	355	14	09
Reactor	4,994	2,521	130	26	52
Research, General	20,163	4,072	397	20	86
Research, Fusion	1,133	167	9	z,	35
Waste Proc./Management	5,432	1,496	126	, 23	84
Weapons Fab. & Test.	18,956	8,806	946	50	107
Other	13,096	2,312	120	თ	52
Visitors	11,827	4,447	453	38	102
DOE Offices	1,781	173	2	1	13
Total	124,702	35,002	2,944 24 84	24	84

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

**TABLE 4.5.** Collective Dose-Equivalent for Monitored DOE/DOE Contractor Employees and Visitors by Field Organization, 1991<sup>(a)</sup>

Field Organization	Number of Monitored Individuals	Number of Individuals with Measurable Doses	Collective Dose-Equivalent (Person-rem)	Average Dose-Equivalent (mrem)	Average Dose-Equivalent (mrem) per Individual with Measurable Doses
Albuquerque Operations	21,379	2,884	389	18	135
Chicago Operations	11,493	2,648	173	15	65
DOE Headquarters	872	65	0	0	5
Idaho Operations	7,402	1,273	177	24	139
Nevada Operations	1,196	40	က	က	85
Oak Ridge Operations	26,467	4,717	172	9	36
Pittsburgh N.R. Office	2,195	1,703	84	38	50
Richland Operations	9,404	3,058	275	53	06
Rocky Flats Operations	8,358	7,643	905	108	118
San Francisco Operations	10,622	613	77	7	126
Savannah River Operations	22,583	8,391	459	20	55
Schenectady N.R. Office	2,731	1,967	233	85	118
Total DOE	124,702	35,002	2,944	24	84

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

**TABLE 4.6.** Percent of Collective Dose-Equivalent for Monitored DOE/DOE Contractor Employees and Visitors Attributed to a Facility Type Within Each Field Organization, 1991<sup>(a)</sup>

							Facility Type	/ Type					
		2	Fuel		Maint&		Rese	Research	Waste Weapon	Weapon			DOE
Field Organization	Accel	Enrich	Fab.	Proc	Support	Reactor	Genr	Fusion	Proc.	F&T	<u>Other</u>	Visit	Office 0
Albuquerque Operations	10.5				8.8	2.3	46.1	0.1	1.4	6.7	2.9	21.2	
Chicago Operations	45.8		9.0		6.7	5.9	14.8	3.1	9.0		0.3	22.1	
DOE Headquarters													100.0
Idaho Operations				34.5	1.1	. 18.7	2.5		0.4		12.7	30.2	
Nevada Operations	91.9									1.4		9.9	
Oak Ridge Operations		18.5	15.9	4.6			22.5		0.3	13.6		24.5	
Pittsburgh N.R. Office						22.3	74.3				1.2	2.2	
Richland Operations			0.2	3.0	39.2	6.8	17.3		29.3		3.5	0.4	0.2
Rocky Flats Operations										98.2		1.8	
San Francisco Operations	21.0	5.4			23.1		29.4		0.1	6.2	9.3	5.4	0.1
Savannah River Operations				27.8	39.5	4.5	2.2		8.2	1.2	14.8	1.4	0.3
Schenectady N.R. Office	1	 			1	8.5	2.7		!	i ! ! !	0.1	88.7	1 1 1
Total DOE	4.7	1.2	1.0	7.0	12.0	4.4	13.5	0.2	4.3	32.1	4.1	15.4	0.1

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

at each field organization. Table 4.7 presents collective dose equivalents for each field organization from 1982 to 1991.

## 4.4 DISTRIBUTION BY OCCUPATION CATEGORY

DOE Order 5484.1 requires that for each monitored individual (employee and visitor), a three-digit occupation code be included indicating the generic occupation that best fits the individual's occupation title. The 44 three-digit codes pertain to DOE occupation codes summarizing all Standard Occupational Classification (SOC) codes from the Department of Commerce's SOC Manual of 1980. The DOE is considering a revised requirement to report occupations by the full four-digit SOC code. This would eliminate the need for an intermediate code, standardize occupational classifications, and provide research data at a greater level of detail.

For this report, the 44 DOE occupational classifications were summarized into 11 general occupations to facilitate analysis:

• Management - managers and administrators, sales, support and clerical

Scientists - scientists, engineers, health physicists, miscellaneous professionals,
 physicians, and nurses

• Technicians - health technicians, engineering technicians, science technicians, radiation monitors/technicians, miscellaneous technicians

• Service - firefighters, security guards, food service employees, janitors, miscellaneous service

• Agriculture - groundskeepers, forest workers, miscellaneous agriculture

Construction - mechanics/repairers, masons, carpenters, electricians, painters, pipe fitters,
 miners/drillers, miscellaneous repair/construction

Production - machinists, sheet metal workers, operators - plant/system/utility, machine setup/operators, welders and solderers, miscellaneous precision/production

• Transport - truck drivers, bus drivers, pilots, equipment operators, miscellaneous transport

• Laborers - handlers/laborers/helpers

• Miscellaneous - military, miscellaneous

• Unknown - indicates that an occupation code was not specified on the form.

**TABLE 4.7.** Collective Dose-Equivalent (person-rem)<sup>(a)</sup> for Monitored DOE/DOE Contractor Employees and Visitors by Field Organization, 1982-1991<sup>(b)</sup>

Field Organization	1982	1983	1984	1985	1986	1987	1988	1989	1990	1991
Albuquerque Operations <sup>(c)</sup>	1,112	1,190	1,423	1,344	979	483	556	432	363	389
Chicago Operations	587	623	615	505	408	348	310	240	214	173
Idaho Operations	363	353	441	420	620	318	253	336	366	177
Nevada Operations	59	25	24	34	65	80	13	9	7	က
Oak Ridge Operations	401	371	419	353	587	517	360	218	173	172
Pittsburgh N.R. Office	194	220	180	180	109	78	98	85	23	84
Richland Operations	2,272	2,458	2,399	2,548	2,321	2,477	654	619	353	275
Rocky Flats Operations <sup>(c)</sup>	1,173	1,142	1,315	1,556	1,407	880	654	412	769	905
San Francisco Operations	289	267	195	187	66	78	74	82	64	77
Savannah River Operations	1,310	1,293	1,283	1,394	1,498	945	887	804	753	459
Schenectady N.R. Office	147	217	130	165	167	220	81	140	240	233
Total	7,879	8,158	8,422	8,684	8,261	6,353	3,928	3,375	3,327	2,944

(E)

Throughout this report there may be minor variations in collective dose-equivalent values because of rounding. The data may differ slightly from previous reports due to revisions received after publication. Effective 1/1/90, Rocky Flats Operations was designated as a separate DOE field organization. Accordingly, all current and historical radiation data associated with the Rocky Flats facilities have been extracted from Albuquerque Operations data and identified separately. Table 4.8 lists the number of individuals monitored, the number of individuals monitored who received a measurable dose equivalent, and the average dose equivalents for each occupation category. The "Scientists" category accounted for both the most individuals monitored and the most individuals monitored who received a measurable exposure. Individuals in the "Production" category received the highest average dose equivalent per individual monitored (60 mrem (0.60 mSv)) and received the highest average dose equivalent per individual monitored who received a measurable exposure (115 mrem (1.15 mSv)). Figure 4.7 illustrates the data in Table 4.8 including an indication of the sex distribution of the individuals. Figure 4.8 illustrates the collective dose equivalent values in Table 4.8 as a pie chart. Table 4.9 lists the number of individuals monitored according to occupation and facility type.

### 4.5 DISTRIBUTION BY AGE AND SEX

The 1991 exposure data submitted per DOE Order 5484.1 included information on the age and sex of the exposed individuals (employees and visitors). Unfortunately, some records were submitted without the required information. For the analysis in this report, 12 age categories were defined: 19 and less, 65 and greater, nine 5-year age groups beginning with the 20-24 age group and ending with the 60-64 age group, and unknown age. Regarding sex of the exposed individuals, a separate category for unspecified sex was defined. It was clear from the data that if sex was not specified on the form, other information such as age, occupation, or facility type was likely to be unspecified or unknown as well. For example, of the 1,286 individuals for whom sex was not specified on the report form, 1,114 (87%) also were not identified by age. Similarly, the occupation was listed as unknown or was unspecified for 1,232 (96%) of the individuals for whom sex was unspecified.

**TABLE 4.8.** Distribution of Total Effective Dose Equivalent for DOE/DOE Contractor Employees and Visitors by Occupation, 1991<sup>(a)</sup>

0.csina+ion	Number of Individuals Monitored	Number of Individuals Monitored Who Received a Measurable Exposure	Collective Dose Equivalent (person-rem)	Average Dose Equivalent per Individual Monitored (mrem)	Average Dose Equivalent per Individual Monitored Who Received a Measurable Exposure (mrem)
Ilnknown	22,909	6,212		26	95
Management	16,215	3,556	306	19	84
Scientists	35,946	4,847	440	12	. 99
Technicians	13,903	4,847	549	39	113
Service	7,350	1,684	73	10	43
Agriculture	136	27	0	0	0
Construction	11,166	4,086	311	28	9/
Production	8,979	4,666	537	09	115
Transportation	2,145	545	33	15	61
Laborers	2,662	965	89	33	92
Miscellaneous	3,291	009	17	2	28

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

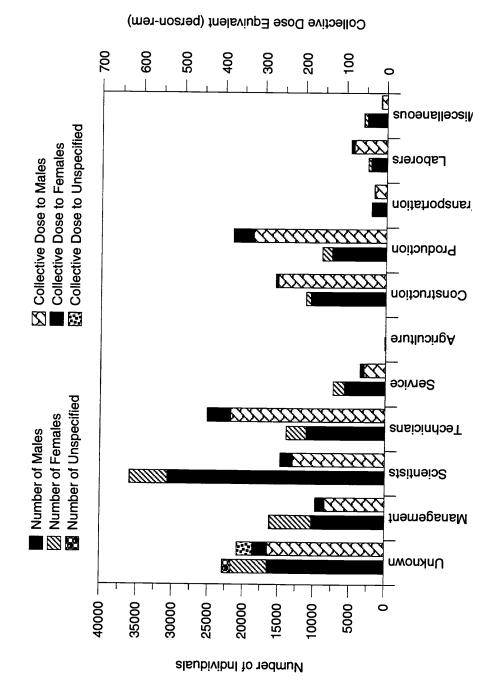


FIGURE 4.7. Penetrating Doses Received by DOE and DOE Contractor Employees and Visitors by Occupation, 1991

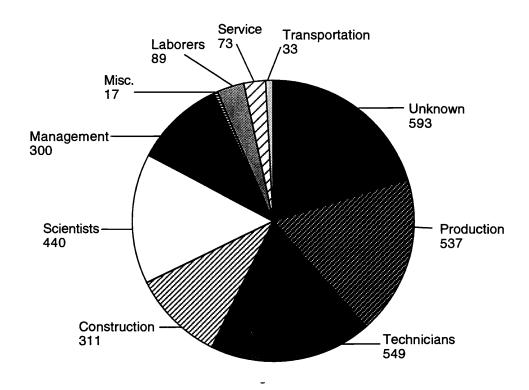


FIGURE 4.8. Contribution of Each Occupation Category to the Total Collective Dose Equivalent, 1991 (numbers indicate person-rem)

Figure 4.9 illustrates the number of individuals by sex who received total effective dose equivalents in various dose-equivalent ranges. Figure 4.10 illustrates the number of individuals by sex and age range who were monitored for ionizing radiation in 1991.

Table 4.10 lists the number of individuals monitored, the numbers of individuals monitored who received a measurable exposure, and the collective and average dose equivalents received by age range. The age groups receiving the highest average dose equivalent per individual monitored was the 65-and-greater age group (64 mrem) (0.64 mSv); the age group receiving the lowest was the 19-or-less group (2 mrem) (0.02 mSv). The age group receiving the highest average dose equivalent per individual who received a measurable exposure was the 65-and-greater age group (288 mrem) (2.88 mSv); the lowest was the 19-or-less group (18 mrem) (0.18 mSv). Internal dose contributions

TABLE 4.9. Number of Monitored DOE/DOE Contractor Employees and Visitors by Occupation and Facility Type, 1991(a)

Total Person-	156	39	55	204	423	339	411	7	128	996	214		2,944
Miscellaneous	488	157	20	100	44	346	1,755	46	59	976	14938	3,291	17
Laborers	15	249	994	52	1,046	64	254	-	952	360	454	2,662	88
Transportation	66	61	29	42	724	82	98	0	101	396	495	2,149	33
Production	177	948	102	1,233	1,124	198	436	33	973	2,820	272	8,979	537
Construction	410	1,128	418	624	3,017	390	737	112	687	1,378	2,265	11,166	311
Agriculture	17	0	0	0	59	2	10	0	0	0	78	136	0
Service	238	546	129	76	1,792	164	463	45	389	738	2,773	7,350	73
<u> Technicians</u>	1,867	935	294	244	2,977	643	2,800	264	865	1,877	1,137	13,903	549
Scientists	3,337	2,272	953	1,896	5,020	2,325	7,835	999	1,979	4,779	4,984	35,946	440
Management	322	2,119	440	755	3,720	1,065	2,347	91	824	3,113	1,419	16,215	300
Unknown	1,313	1,599	r.	0	5,754	17	4,651	127	159	5,066	4,218	22,909	593
Total Persons Monitored	8,283	10,014	2,549	4,995	25,247	5,959	21,374	1,282	6,131	20,264	18,244	124,702	
Facility Type	Accelerator	Fuel/Uranium Enrichment	Fuel Fabrication	Fuel Processing	Maintenance and Support	Reactor	Research, General	Research, Fusion	Waste Proc./ Management	Weapons Fabrication and Testing	Other	Total Persons Monitored	Total Person- rem

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

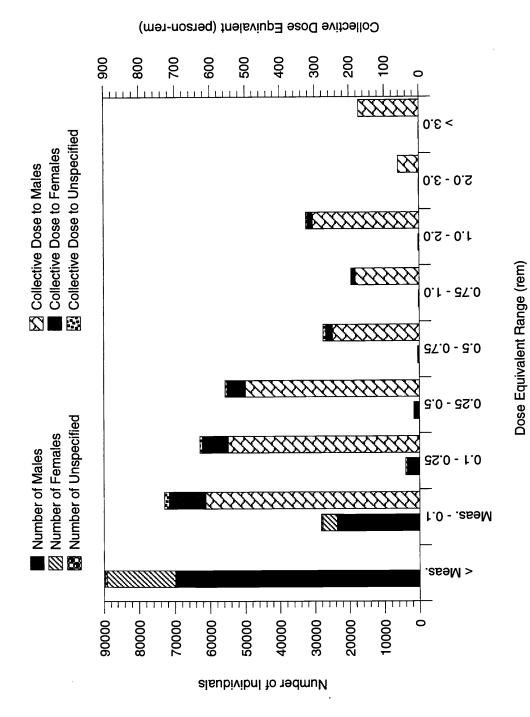


FIGURE 4.9. Distribution of Total Effective Dose Equivalents by Sex and Dose-Equivalent Range for DOE and DOE Contractor Employees and Visitors, 1991

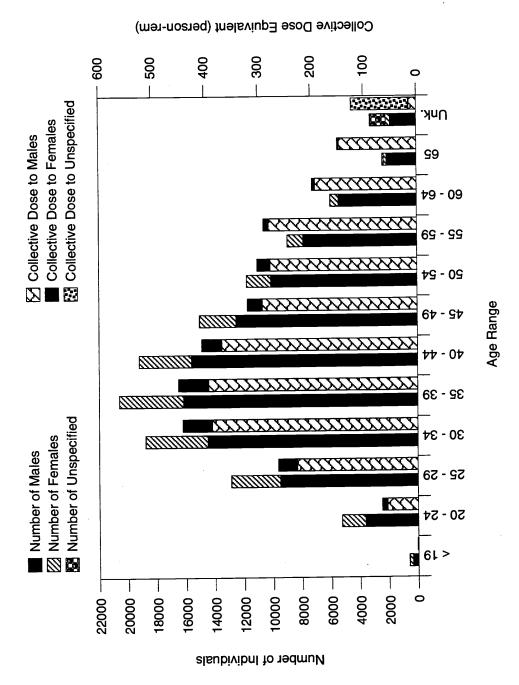


FIGURE 4.10. Number of Individuals (Employees and Visitors) Monitored and Collective Dose Equivalent by Age Range and Sex, 1991

TABLE 4.10. Number of Individuals Monitored and Average Total Effective Dose Equivalent by Age, 1991<sup>(a)</sup>

Average Dose Equivalent per Individual Monitored Who Received a Measurable Exposure (mrem)	18	51	92	77	92	77	81	102	122	122	288	45	84
Average Dose Equivalent per Individual Monitored (mrem)	2	13	20	24	22	21	21	26	32	33	64	17	24
Collective Dose Equivalent (person-rem)	П	29	263	442	450	406	320	302	290	198	150	537	2,944
Number of Individuals Who Received a Measurable Exposure	57	1,318	4,017	5,758	5,938	5,304	3,955	2,974	2,370	1,617	520	1,174	35,002
Number of Individuals Monitored	610	5,278	12,867	18,778	20,600	19,215	15,052	11,808	8,980	6,003	2,326	3,185	124,702
Age Range	6 <b>⋝</b>	20-24	25-29	30-34	35-39	40-44	45-49	50-54	55-59	60-64	<b>%</b>	Unknown	All Individuals

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

(due to past uptakes) to the total effective dose equivalent quantity are the reason the 65-and-greater age group had the highest average dose equivalent per individual who received a measurable exposure.

Table 4.11 presents similar data by sex rather than age. Males received approximately 86% of the collective dose equivalent received by individuals for whom sex was specified. Males also received higher average dose equivalents per individual monitored than did females (26 mrem versus 11 mrem) (0.26 mSv versus 0.11 mSv) as well as higher average dose equivalents per individual monitored who received a measurable exposure (89 mrem (0.89 mSv) versus 57 mrem (0.57 mSv)).

Because of the sensitivity of the fetus to ionizing radiation, which is greater than that of children or adults, it is important to evaluate the doses received by women of child-bearing age. Table 4.12 presents the number of women of child-bearing age (arbitrarily assumed to include women up to the age of 44) who received a measurable dose equivalent in 1991, by facility type. A total of 3,604 women of child-bearing age received a collective dose equivalent of 197 person-rem (1.97 person-Sv). The average individual dose equivalent for these women over all facilities was 55 mrem (0.55 mSv).

Figure 4.11 presents the age distributions of both the number of workers and collective dose equivalents for males and females. As indicated by the ages pertaining to the 50% mark on the figure, the median ages for monitored workers at DOE facilities were approximately 38 and 42 for females and males, respectively. The median ages for collective dose equivalent were approximately 38 and 43, respectively, indicating that, in general, younger workers receive slightly higher doses than do older workers.

TABLE 4.11. Number of Individuals Monitored and Average Total Effective Dose Equivalent by Sex, 1991<sup>(a)</sup>

Average Dose Equivalent per Individual Monitored Who Received a Measurable Exposure (mrem)	89	22	64	84
Average Dose Equivalent per Individual Monitored (mrem)	26	11	32	24
Collective Dose Equivalent (person-rem)	2,634	269	411	2,994
Number of Individuals Who Received a Measurable Exposure	29,619	4,738	64512	35,002
Number of Individuals Monitored	99,491	23,925	1,286	124,702
	Male	Female	Unspecified	All Individuals

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE 4.12. Total Doses Received by Female Employees and Visitors of Childbearing Age, 1991<sup>(a)</sup>

	Total <u>Person-rem</u>	3	5	က	27	43	12	56		11	54	13		198
S	40-44	12	51	7	47	142	30	54	1	30	261	53	688	38
rable Dose	35-39	19	75	13	27	194	36	110		45	286	83	918	22
es Receiving Measu in Each Age Group	30-34	21	7.1	6	79	217	52	105	1	99	254	81	946	99
emales Rece in Each	25-29	21	45	18	47	201	40	71	1	25	140	113	749	36
Number of Females Receiving Measurable Doses in Each Age Group	20-24	16	16	9	18	72	12	30		22	41	09	293	10
	<b>V</b> 0	2	1	1	-	က		1			1		10	0
	Persons	91	259	54	249	829	170	371	က	205	983	390	3,604	
	Facility Type	Accelerator	Fuel/Uran. Enrichment	Fuel Fabrication	Fuel Processing	Maint. and Support	Reactor	Research, General	Research, Fusion	Waste Proc./Management	Weapons Fab. & Test.	0ther	Total Persons	Total Person-rem

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

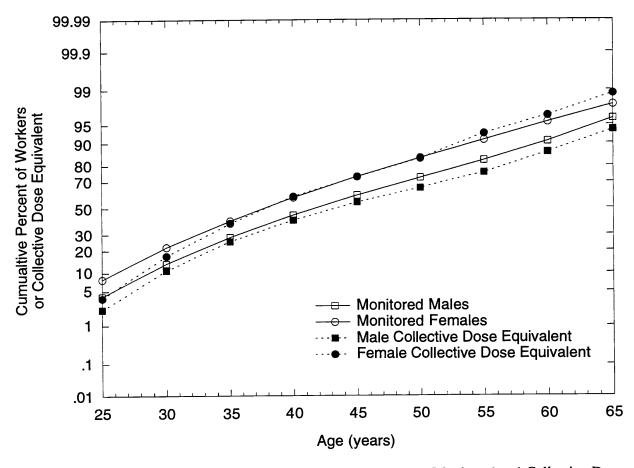


FIGURE 4.11. Number of Individuals (Employees and Visitors) Monitored and Collective Dose Equivalent by Age Range and Sex, 1991

## 4.6 DISTRIBUTION BY TYPE OF EXPOSURE

For calendar year 1991, DOE Order 5484.1 required that specific information on the types of radiation doses received by each worker be reported. Specifically, these included the total effective dose equivalent, the external penetrating dose equivalent (at a depth in tissue of 1.0 cm) including neutron exposure, the dose equivalent from neutron exposure only, the internal effective dose equivalent, the shallow dose equivalent, and the extremity dose equivalent. From these data, the external penetrating beta-gamma dose equivalent can be derived by subtracting the neutron dose equivalent from the external penetrating dose equivalent including neutron exposure. That is, the two contributors to external penetrating dose equivalent are beta-gamma radiation and neutron radiation. The Order does not require reports of dose to the eye.

Table 4.13 lists the various types of dose equivalents received by facility type. Of the total effective dose equivalent of 2,944 person-rem (29.44 person-Sv) received, 2,080 person-rem (20.80 person-Sv (71%)) were attributable to total penetrating radiation and 839 person-rem (8.39 person-Sv (28%)) were attributable to internally deposited radionuclides. When added, the penetrating and internal collective dose equivalent values are less than the collective dose value of total effective dose equivalent. This is due to reporting errors from some of the DOE sites. Of the total external penetrating dose equivalent of 2,080 person-rem (20.80 person-Sv), 1,737 person-rem (17.37 person-Sv (84%)) were attributable to beta-gamma radiation and 343 person-rem (3.43 person-Sv (16%)) were attributable to neutron radiation. Neutron radiation contributed the highest percentage (30%) of the total penetrating dose equivalent at general research facilities. The total shallow dose reported to have been received was 2,643 person-rem (26.43 person-Sv). Relative to the total penetrating dose equivalent, the total shallow dose equivalent was greatest at fuel/uranium enrichment and weapons fabrication and testing facilities, where the shallow dose equivalent exceeded the penetrating dose equivalent by a factors of 2.6 and 1.7, respectively. However, because the critical organ regarding shallow dose equivalents is the skin and because the radiation risk coefficient for induction of fatal skin cancers is low (NCRP 1987a), the penetrating dose equivalents are of the most concern regarding health effects. Collective extremity dose equivalents were 2,252 person-rem (22.52 person-Sv) to the hand and arm and 639 person-rem (6.39 person-Sv) to the foot and leg. Exposure of the hand and arm accounted for 78% of the total extremity collective dose while foot and leg exposure accounted for 22% of the overall extremity exposure. The total extremity collective dose equivalent exceeded the total penetrating collective dose equivalent by 8% (172 person-rem (1.72 person-Sv)).

A detailed comparison of the dose equivalent quantities by sex, age range, occupation, and facility type can be found in Section 5.0 of this report. The magnitude of the postulated health effects from radiation doses received at DOE facilities is discussed in Section 7.0 of this report.

#### 4.7 EVALUATION OF TRENDS

Doses received by DOE and DOE contractor employees and visitors have decreased dramatically over the last several years (see Table 4.7). For example, in 1985 the collective dose equivalent received by employees and visitors was 8,684 person-rem (86.84 person-Sv); in 1991, this value was

TABLE 4.13. Dose Equivalent by Dose-Equivalent Type (person-rem)<sup>(a)</sup>

	Total			Penetrating			Extr	Extremity
Facility Type	Effective Dose Equivalent	Internal	Total	Beta-Gamma	Neutron	Shallow	Arm & Hand	Leg & Foot
Accelerator	156	18	137	119	19	116	100	51
Fuel/Uran. Enrich.	39	က	36	35	-	94	48	24
Fuel Fab.	55	0	55	54	0	58	7	1
Fuel Process.	204	18	187	157	30	285	107	б
Maintenance & Support	423	66	311	250	09	405	596	127
Reactor	339	r.	334	328	9	361	133	27
Research, Gen.	441	59	340	237	103	309	471	131
Research, Fusion	, ,	0	9	9	0	9	0	9
Waste Proc./Mgmt.	128	19	107	7.7	30	121	212	88
Weapons Fab. & Testing	996	209	361	308	53	621	764	148
Other	214	10	203	163	41	263	113	32
Total	2,944	839	2,080	1,737	343	2,643	2,252	639

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

2,944 person-rem (29.44 person-Sv). Some of this decrease is attributable to the fact that the 1985 value was estimated from the numbers of individuals reported to have received doses in various dose-equivalent ranges. Previous to the 1987 reporting period, collective dose equivalents were calculated by multiplying the number of individuals who received dose equivalents in various dose-equivalent intervals by the midpoint of those intervals and summing the products. However, the majority of the decrease is attributable to other factors, such as the reduction of production tasks at DOE facilities and an increased emphasis on ALARA programs.

The most evident example of the recent dramatic decrease in collective doses is at the Richland Field Organization. In 1987, the collective dose equivalent to employees and visitors at Richland was 2,477 person-rem (24.77 person-Sv); in 1991, this value dropped by 89% to 275 person-rem (2.75 person-Sv). This decrease was primarily the result of both changes in the type of work performed and facility closures. Decreases also occurred from 1986 to 1991 at the Oak Ridge (-71%) and Savannah River (-69%) field organizations.

The 1991 data demonstrate that the significant decrease in collective dose equivalent is not attributable to fewer individuals being monitored, but to lower doses to those individuals who are monitored. Figure 4.12 illustrates the recent dramatic decrease in average annual dose equivalent per individual monitored who received a measurable exposure. Table 4.14 lists similar data for each facility type. Table 4.15 lists collective dose equivalent by facility type for the years 1980 through 1991.

One correlative effect of lower average individual dose equivalents is fewer employees who exceed various dose-equivalent levels. Figure 4.13 illustrates the number of employees who received dose equivalents greater than 0.5 rem (5 mSv), 1.0 rem (10 mSv), or 2.0 rem (20 mSv) from 1980 to 1991. As indicated in the figure, the numbers decreased significantly during the 1988-1991 time period. As a result, fewer employees are being exposed to doses that are significant fractions of the annual dose limit.

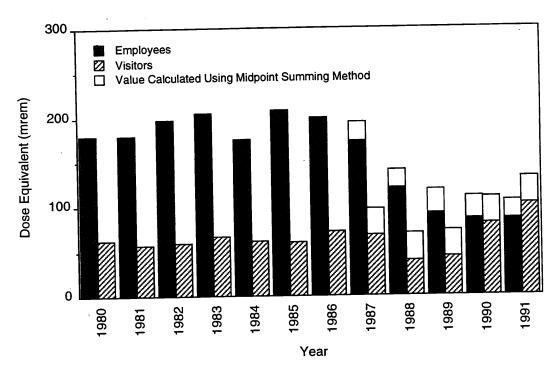


FIGURE 4.12. Average Dose Equivalent per Individual Who Received a Measurable Exposure, 1980-1991

**TABLE 4.14.** Average Dose Equivalent per Individual Who Received a Measurable Exposure by Facility Type, (a) 1980-1991 (mrem)

All Facilities	157	156	164	190	167	182	179	159	103	84	72	85	29	84
DOE Offices	57	29	62	57	62	63	65	30	19	21	16	22	12	12
Visitors	59	57	28	99	09	59	7.1	69	39	43	81	80	93	102
0ther	217	202	169	202	.164	188	185	173	100	69	65	89	59	64
Accelerator	209	228	509	219	196	175	129	86	114	116	87	87	89	101
Gen. <u>Re</u> search	120	140	168	169	154	193	211	150	124	97	06	102	85	6
Weapons Fab. & Test.	120	129	136	149	147	170	166	183	139	105	46	112	53	107
Uran. <u>Enrichment</u>	117	74	98	79	80	63	71	37	59	28	28	56	27	21
Fuel Proc.	442	412	362	298	294	318	314	267	217	259	170	176	119	124
Fuel Fab.	236	246	306	322	283	226	227	155	112	89	84	87	79	78
Reactor	278	270	302	313	323	323	300	239	104	92	61	89	52	52
Year	1980	1981	1982	1983	1984	1985	1986	1987	1988	1989	1990	1990 <sup>(b)</sup>	1991	1991 (b)

Throughout this report there may be minor variations in collective dose-equivalent values because of rounding. Beginning in 1987, three facility categories were added to those listed in the table: maintenance and support, fusion research, and waste processing/management. For this table, these facility categories are included in the "other" category for 1987-1989. Total effective dose equivalent for 1990. All other data in this table describe whole-body penetrating dose exposure. (a) (P)

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TABLE 4.15. Collective Dose Equivalent<sup>(a)</sup> by Facility Type, <sup>(b)</sup> 1980-1991 (person-rem)

All Facilities	8,024	7,483	7,879	8,158	8,423	8,684	8,465	6,353	3,901	3,375	2,545	3,327	2,079	2,944
DOE <u>Offices</u>	59	38	56	30	30	20	20	∞	2	က	က	4	2	2
Visitors	619	571	989	300	368	461	554	373	245	303	471	472	383	453
Other	1,773	1,813	1,293	1,522	1,944	2,025	2,117	2,260	1,195	928	777	849	422	487
Accelerator	412	348	254	273	248	262	232	169	194	184	127	127	121	139
Gen. <u>Research</u>	1,611	1,535	1,676	1,662	1,736	1,484	1,357	769	554	208	398	439	326	397
Weapons Fab. & <u>Testing</u>	869	982	1,056	1,399	1,672	1,851	1,802	1,028	797	512	197	839	341	946
Uran. <u>Enrichment</u>	156	62	30	31	28	56	39	41	32	41	47	22	33	36
Fuel Proc.	1,047	592	735	726	515	574	598	426	374	491	282	292	187	205
Fuel <u>Fab.</u>	323	267	411	434	264	265	356	271	171	7.7	29	63	59	59
Reactor	1,185	1,270	1,612	1,781	1,620	1,716	1,391	1,007	366	329	183	184	125	130
Year	1980	1981	1982	1983	1984	1985	1986	1987	1988	1989	1990	1990 <sup>(c)</sup>	1991	1991 <sup>(c)</sup>

Throughout this report there may be minor variations in collective dose-equivalent values because of rounding. Beginning 1987, three facility categories were added to those listed in the table: maintenance and support, fusion research, and waste processing/management. For this table, these facility categories are included in the "other" category for 1987-1989. Iotal effective dose equivalent for 1990. All other data in this table describe whole-body penetrating dose exposure. (a)

<sup>(</sup>၁)

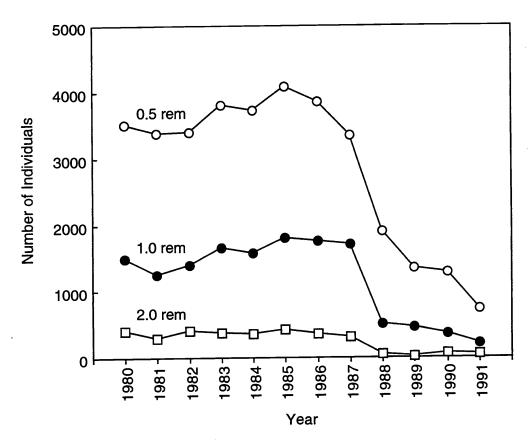


FIGURE 4.13. Number of Employees Who Received Dose Equivalents Greater Than 0.5 rem, 1 rem, and 2 rem, 1980-1991

## 5.0 ADDITIONAL DOSE REPORTING QUANTITIES

As mentioned earlier, this report is the second to report the complete data for all dose reporting quantities required in DOE Order 5484.1. These dose reporting quantities include total effective dose equivalent, annual internal dose equivalent, arm and hand extremity dose equivalent, and leg and foot extremity dose equivalent. This section will highlight and compare these dose quantities to the whole-body penetrating dose equivalent quantity.

The total effective dose equivalent quantity is the sum of the whole-body penetrating dose equivalent and annual internal dose equivalent. In past annual reports previous to 1990, the whole-body penetrating dose equivalent quantity was the main one reported and analyzed. Previous to 1990, only internal depositions that exceeded 50% of the annual standard were reported.

# 5.1 COMPARISON OF TOTAL EFFECTIVE DOSE EQUIVALENT, PENETRATING DOSE EQUIVALENT, AND INTERNAL DOSE EQUIVALENT

Figures 5.1 through 5.9 highlight the total effective dose equivalent and internal dose equivalent quantities. These quantities are compared to the penetrating dose equivalent primarily reported in the past. The average value for these quantities is shown for the age, sex, occupation, and facility categories described in Section 4.0.

### 5.1.1 Comparison by Age Range and Sex

Comparisons of total effective dose equivalent, penetrating dose equivalent, and internal dose equivalent by age range and sex are shown in Figures 5.1 through 5.3. Figure 5.1 illustrates the average values for the dose equivalent quantities by age range for all DOE and DOE contractor employees and visitors. The average quantities are shown in Figures 5.2 and 5.3 for male and female employees and visitors, respectively. Average total effective dose equivalent and penetrating dose equivalent values are generally highest for employees and visitors in the age ranges 30 to 40 and 50 to 65 and greater. Older male employees have much higher average internal dose equivalent values due to past internal uptakes of radioactive material. A similar trend is seen for internal dose to female employees. The higher internal dose averages for older employees accounts for the increase in

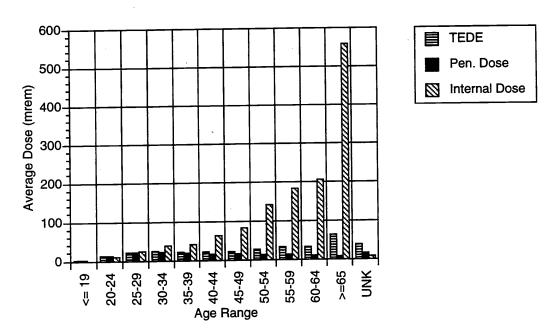


FIGURE 5.1. Comparison of Average Total Effective Dose Equivalent, Average Penetrating Dose Equivalent, and Average Internal Dose Equivalent by Age Range for All Employees and Visitors, 1991

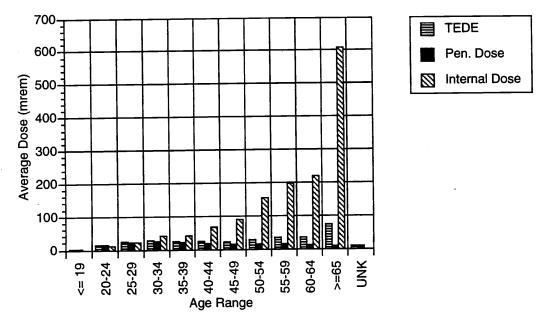


FIGURE 5.2. Comparison of Average Total Effective Dose Equivalent, Average Penetrating Dose Equivalent, and Average Internal Dose Equivalent by Age Range for Male Employees and Visitors, 1991

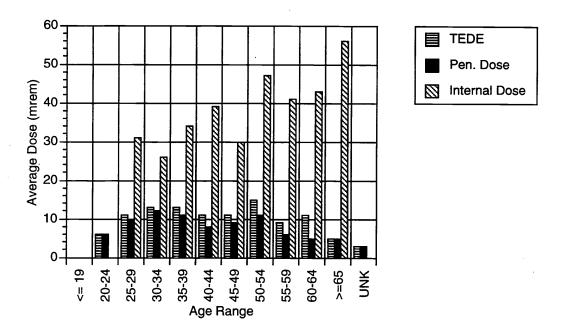


FIGURE 5.3. Comparison of Average Total Effective Dose Equivalent, Average Penetrating Dose Equivalent, and Average Internal Dose Equivalent by Age Range for Female Employees and Visitors, 1991

total effective dose equivalent for older age groups. The penetrating dose equivalent average generally decreases for all employees over the age of 40.

#### 5.1.2 Comparison by Occupation and Sex

Figure 5.4 illustrates the average dose equivalent quantities by occupation for all employees. Production workers had the highest overall average total effective dose equivalent (60 mrem (0.60 mSv)) and penetrating dose equivalent (42 mrem (0.42 mSv)). Scientists had the highest overall average internal dose equivalent (108 mrem (1.08 mSv)) for known occupation categories. The Unknown category had the highest overall average internal dose equivalent for all cartegories (147 mrem (1.47 mSv)). Employees classified as agricultural workers had the lowest average total effective, penetrating, and internal dose equivalent values (< 1 mrem (< 0.01 mSv)). Similar data trends are shown for male and female workers in Figures 5.5 and 5.6, respectively.

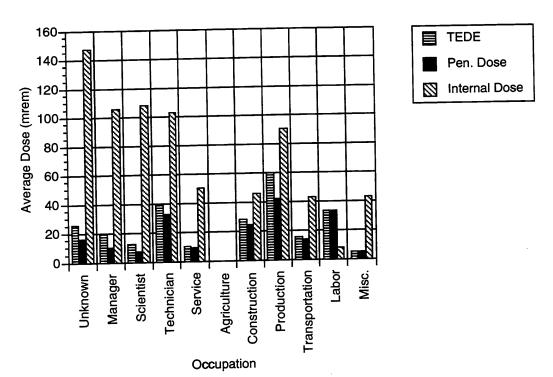


FIGURE 5.4. Comparison of Average Total Effective Dose Equivalent, Average Penetrating Dose Equivalent, and Average Internal Dose Equivalent by Occupation for All Employees and Visitors, 1991

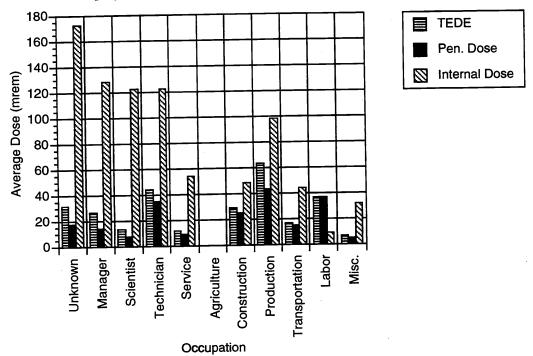


FIGURE 5.5. Comparison of Average Total Effective Dose Equivalent, Average Penetrating Dose Equivalent, and Average Internal Dose Equivalent by Occupation for Male Employees and Visitors, 1991

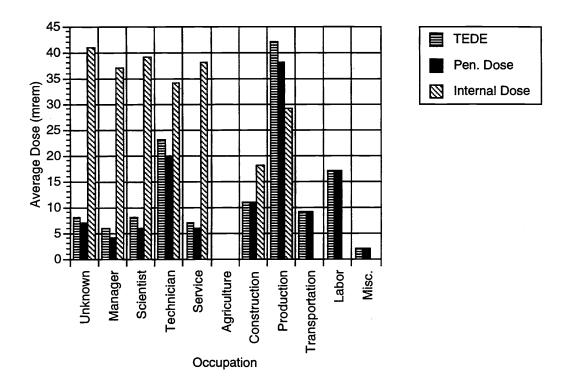


FIGURE 5.6. Comparison of Average Total Effective Dose Equivalent, Average Penetrating Dose Equivalent, and Average Internal Dose Equivalent by Occupation for Female Employees and Visitors, 1991

#### 5.1.3 Comparison by Facility Type and Sex

Average dose equivalent values are shown for DOE facility types in Figures 5.7 through 5.9. Data shown for all employees in Figure 5.7 reveal that those working at fuel processing facilities received the highest average total effective dose equivalent (57 mrem (0.57 mSv)) and penetrating dose equivalent (56 mrem (0.56 mrem)). Employees at general research facilities received the highest average internal dose equivalent (156 mrem (1.56 mSv)). Fuel and uranium processing employees received the lowest average total effective (4 mrem (0.04 mSv)) and penetrating (4 mrem (0.04 mSv)) dose equivalent values. Fusion research and fuel fabrication employees had the lowest internal dose equivalent values (< 1 mrem (< 0.01 mSv)). Accelerator facility employees had the highest average internal dose (1636 mrem (16.36 mSv)). This high value was due to one individual (out of 11 reported) who had an internal dose equivalent exceeding 5 rem (50 mSv). The individual's exposure was due to an uptake of <sup>238</sup>Pu in 1971. The other individuals had internal dose equivalent values of

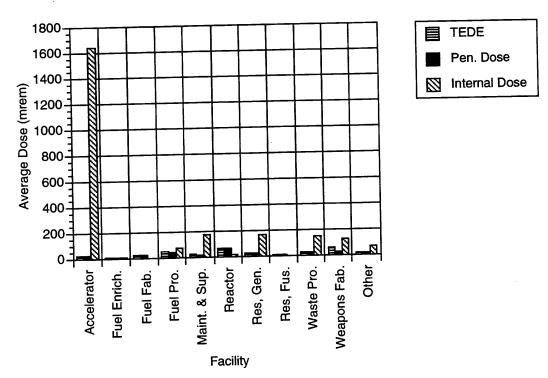


FIGURE 5.7. Comparison of Average Total Effective Dose Equivalent, Average Penetrating Dose Equivalent, and Average Internal Dose Equivalent by Facility Type for All Employees and Visitors, 1991

less than 100 mrem (1.00 mSv). Again, similar data trends were observed for the male and female components of the DOE population (Figures 5.8 and 5.9).

# 5.2 COMPARISON OF PENETRATING DOSE EQUIVALENT, HAND AND ARM EXTREMITY DOSE EQUIVALENT, AND FOOT AND LEG EXTREMITY DOSE EQUIVALENT

Figures 5.10 through 5.18 highlight the hand and arm extremity dose equivalent and foot and leg dose equivalent quantities. These quantities are compared to the whole-body penetrating dose equivalent. Again, the average value for these quantities is shown for age, sex, occupation, and facility categories.

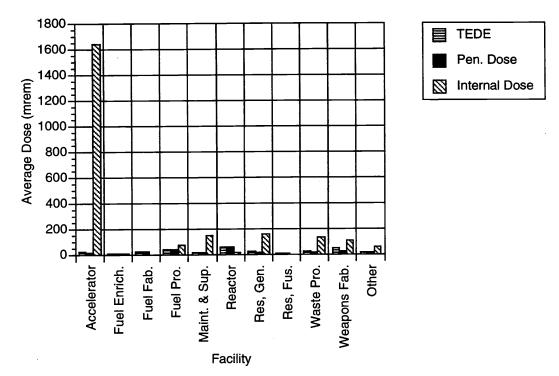


FIGURE 5.8. Comparison of Average Total Effective Dose Equivalent, Average Penetrating Dose Equivalent, and Average Internal Dose Equivalent by Facility Type for Male Employees and Visitors, 1991

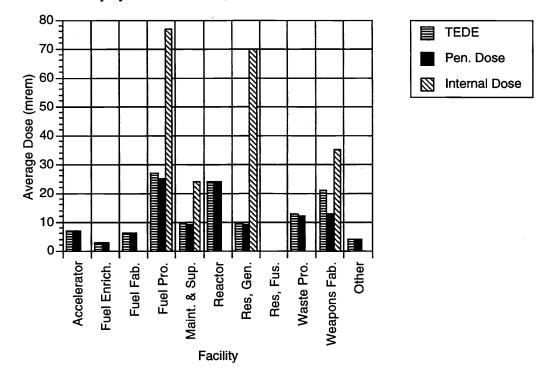


FIGURE 5.9. Comparison of Average Total Effective Dose Equivalent, Average Penetrating Dose Equivalent, and Average Internal Dose Equivalent by Facility Type for Female Employees and Visitors, 1991

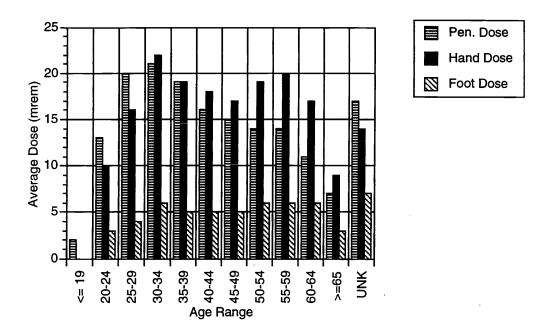


FIGURE 5.10. Comparison of Average Penetrating Dose Equivalent, Average Hand and Arm Extremity Dose Equivalent, and Average Foot and Leg Extremity Dose Equivalent by Age Range for All Employees and Visitors, 1991

## 5.2.1 Comparison by Age Range and Sex

Average hand and foot extremity dose equivalent values were highest for employees between the ages of 30 and 60. There is very little variation between the data shown for all employees in Figure 5.10 and male and female employees shown in Figures 5.11 and 5.12, respectively. Also, there is little variation in the extremity exposure of the maximally exposed age groups. The average hand extremity dose equivalent value was approximately 18 mrem (0.18 mSv), and the average foot extremity dose equivalent value was approximately 5 mrem (0.05 mrem).

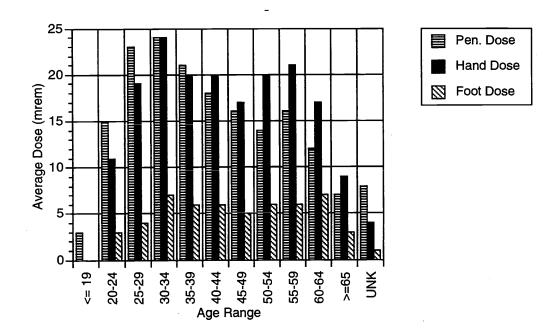


FIGURE 5.11. Comparison of Average Penetrating Dose Equivalent, Average Hand and Arm Extremity Dose Equivalent, and Average Foot and Leg Extremity Dose Equivalent by Age Range for Male Employees and Visitors, 1991

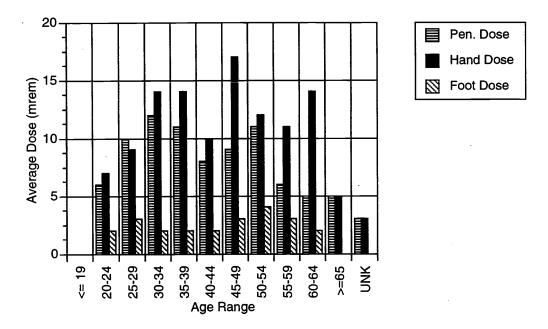


FIGURE 5.12. Comparison of Average Penetrating Dose Equivalent, Average Hand and Arm Extremity Dose Equivalent, and Average Foot and Leg Extremity Dose Equivalent by Age Range for Female Employees and Visitors, 1991

#### 5.2.2 Comparison by Occupation and Sex

Figure 5.13 illustrates that production employees received the highest average hand extremity dose equivalent (66 mrem (0.66 mSv)) and foot extremity dose equivalent (21 mrem (0.21 mSv)). Employees in the algirulture occupation category received the lowest average hand extremity dose equivalent (< 1 mrem (< 0.01 mSv)) and foot extremity dose equivalent (< 1 mrem (< 0.01 mSv)). Figures 5.14 and 5.15 illustrate the similar trends for the male and female employees, respectively.

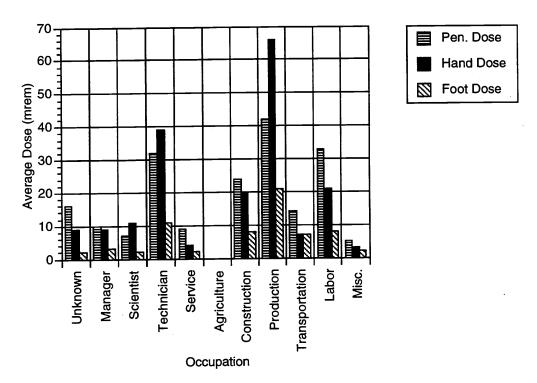


FIGURE 5.13. Comparison of Average Penetrating Dose Equivalent, Average Hand and Arm Extremity Dose Equivalent, and Average Foot and Leg Extremity Dose Equivalent by Occupation for All Employees and Visitors, 1991

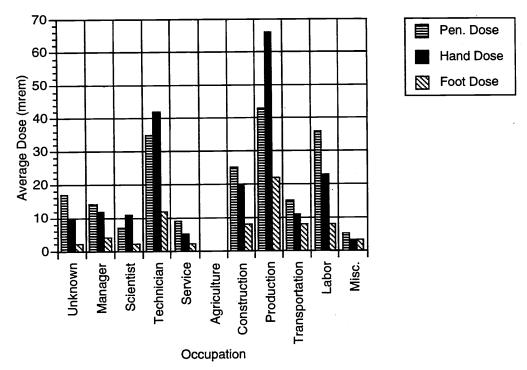


FIGURE 5.14. Comparison of Average Penetrating Dose Equivalent, Average Hand and Arm Extremity Dose Equivalent, and Average Foot and Leg Extremity Dose Equivalent by Occupation for Male Employees and Visitors, 1991

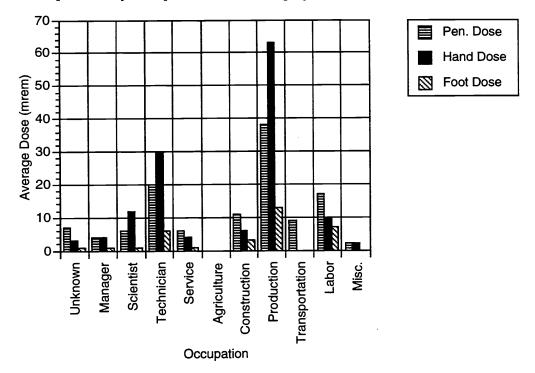


FIGURE 5.15. Comparison of Average Penetrating Dose Equivalent, Average Hand and Arm Extremity Dose Equivalent, and Average Foot and Leg Extremity Dose Equivalent by Occupation for Female Employees and Visitors, 1991

## 5.2.3 Comparison by Facility Type and Sex

As shown in Figure 5.16, individuals employed in weapons fabrication facilities received the highest average hand extremity dose equivalent (37 mrem (0.37 mSv)) and waste processing employees received the highest foot extremity dose equivalent (14 mrem (0.14 mSv)). Employees at fusion research facilities received the lowest average hand extremity dose equivalent (< 1 mrem (< 0.01 mSv)) and foot extremity dose equivalent (< 1 mrem (< 0.01 mSv)). Again, similar trends were seen for the male and female components of the population (Figures 5.17 and 5.18).

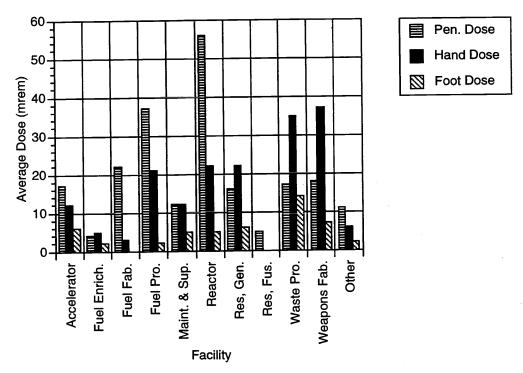


FIGURE 5.16. Comparison of Average Penetrating Dose Equivalent, Average Hand and Arm Extremity Dose Equivalent, and Average Foot and Leg Extremity Dose Equivalent by Facility Type for All Employees and Visitors, 1991

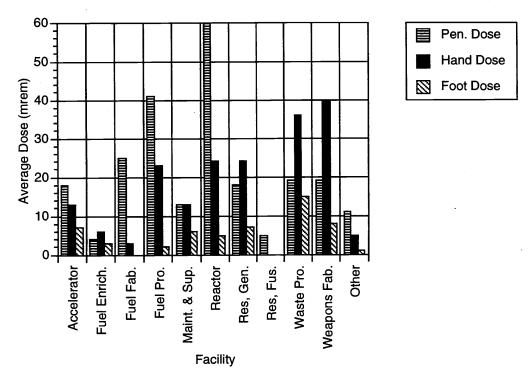


FIGURE 5.17. Comparison of Average Penetrating Dose Equivalent, Average Hand and Arm Extremity Dose Equivalent, and Average Foot and Leg Extremity Dose Equivalent by Facility Type for Male Employees and Visitors, 1991

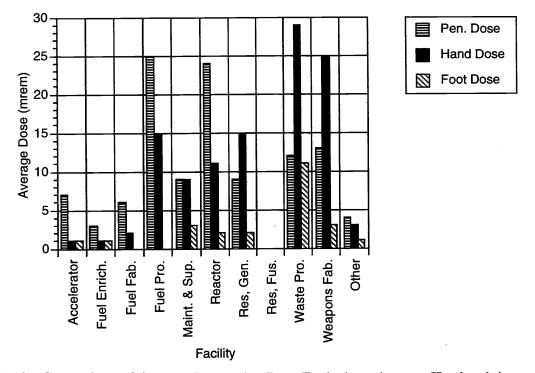


FIGURE 5.18. Comparison of Average Penetrating Dose Equivalent, Average Hand and Arm Extremity Dose Equivalent, and Average Foot and Leg Extremity Dose Equivalent by Facility Type for Female Employees and Visitors, 1991

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# 6.0 REPORTABLE RADIATION EXPOSURE INCIDENTS

In DOE Order 5484.1, the DOE has established criteria for classifying, reporting, and investigating radiation exposure incidents. Depending on the individual doses received, incidents involving exposure to radiation are classified as either Type A, Type B, or Type C occurrences. A Type A occurrence must be reported to DOE Headquarters immediately, and an investigation of the incident is conducted by a DOE Headquarters or field organization board. A Type B occurrence must be reported to DOE Headquarters within 72 hours, and an investigation of the incident is conducted by a DOE board appointed by the head of the field organization. A Type C incident is required to be reported by memo, and an investigation is conducted by DOE contractor personnel when their operations are involved, or by DOE personnel when Federal operations are involved.

Table 6.1 lists the criteria for classifying incidents involving radiation exposures at DOE facilities. Descriptions of such incidents are normally reported to the System Safety Development Center following submittal of the investigation report. No such incidents were reported to have occurred in calendar year 1991.

**TABLE 6.1.** Dose Criteria for Classification of Incidents Involving Occupational Radiation Exposures

	Dose Criter	ia_for_Inciden	t Type (rem)
Type of Exposure	A(a)	B(b)	
Whole-body	25	5	3
Skin of the whole-body	75	15	5
Thyroid	N/A	15	5
Forearms	150	30	10
Hands and feet	375	75	25
Internal dose	5 times annual standard	In excess of annual standard	N/A

<sup>(</sup>a) rem values pertain to a single exposure except for the value for the whole-body, which pertains to a single or annual cumulated exposure.

<sup>(</sup>b) rem values pertain to doses accumulated in one quarter.

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## 7.0 COMPARISON OF DOSES TO RISKS

Crucial to assessing the safety of DOE operations with respect to occupational radiation exposure is an assessment of the risks from doses received by DOE and DOE contractor employees. Section 4.0 of this report presented summaries of the radiation doses received by DOE and DOE contractor employees. Although the average doses were much lower than the DOE limits (indicating the impact of ALARA programs and changing missions at many DOE sites), comparison of employee doses to risks is appropriate for evaluating the magnitude of health effects, if any, that may be expected to occur. This section compares the doses received by DOE and DOE contractor employees in 1991 to risks based on published radiation risk coefficients and compares the calculated risks to other risks incurred both inside and outside the workplace.

Important considerations in assessing the relative significance of the risk of radiation doses received at DOE facilities are the doses received from sources other than working at the facilities. Everyone receives radiation doses regularly from various sources, including terrestrial radiation from naturally radioactive elements in the soil, cosmic radiation from space, radon in the air, and naturally radioactive potassium in our bodies. Other sources of radiation to which many of us are exposed include radiation from medical and dental procedures, cigarette smoke, fallout from past nuclear testing, and various food and other consumer products. Typical radiation doses received from each of these sources are listed in Table 7.1. By comparison to the values in Table 7.1, the average dose equivalent received by a DOE and DOE contractor employee who received a measurable occupational exposure during 1991 (82 mrem (0.82 mSv)) was less than the average dose equivalent received by an individual from non-work-related sources.

Although low doses of radiation have not been demonstrated to increase the incidence of cancer or other diseases, risk estimates have been developed by extrapolating from known effects at high doses and high dose rates to hypothetical effects at low doses and low dose rates. Based primarily on data from survivors of the atomic bombings at Hiroshima and Nagasaki, risk estimates have been developed that express the risk of death from cancer per unit whole-body dose equivalent of ionizing radiation. According to several sources, data published in 1980 suggest that a population distributed over all ages and both sexes would experience approximately 1 x 10<sup>-4</sup> cancer deaths per person per rem (NCRP 1987a, ICRP 1977, NAS 1980, UNSCEAR 1977). However, as detailed in the BEIR III

**TABLE 7.1.** Radiation Doses Received by Individuals in the U.S. from Sources Other than Occupational Exposures (adapted from NCRP Publication 93 (NCRP 1987b))

	Average Annual Effective Dose Equivalent
Source	per Member of the U.S. Population (mrem)
Natural sources Radon Cosmic Terrestrial In vivo	200 27 28 29
Nuclear Fuel Cycle	0.005
Consumer Products Domestic water supply Building materials Other	1 - 6 3.6 1 - 10
Medical Total <sup>(a)</sup>	53 ~360

<sup>(</sup>a) Value pertains to a nonsmoker. An additional 1300 mrem per year is estimated to be received by a typical smoker from inhalation of tobacco smoke.

report (NAS 1980), risk coefficients vary considerably depending on the age and sex of the exposed individual. Furthermore, the calculated risk to an individual exposed to low levels of ionizing radiation depends highly on the models chosen to extrapolate from the data on Hiroshima and Nagasaki, where excess deaths were observed only at relatively high doses delivered over a very short period of time.

More recently, both the United Nations Scientific Committee on the Effects of Atomic Radiation (UNSCEAR) and the Committee on the Biological Effects of Ionizing Radiations (BEIR) provided risk estimates based on a reassessment of the atomic bomb dosimetry as well as extended followups of the survivor data (UNSCEAR 1988, NAS 1990). In general, the associated risk estimates range from approximately 5 x 10<sup>-4</sup> per rem to 1 x 10<sup>-3</sup> per rem, depending on the age, sex, and risk projection model used; these estimates are based on acute exposures of at least 10 rem (100 mSv). For low doses and dose rates, both UNSCEAR and BEIR recognized the need to reduce these risk estimates by applying a dose rate effectiveness factor (DREF) of at least 2 to these values.

Figure 7.1 shows the estimated incidence of fatal cancers and the total numbers of person-years of life lost based on the whole-body ionizing radiation doses received at DOE facilities in 1991. These hypothetical data are based on age- and sex-specific risk equations provided in the BEIR V report (NAS 1990) and life table calculations as described by Bunger, Cook, and Barrick (1981) and Merwin, Traub, and Faust (1990).

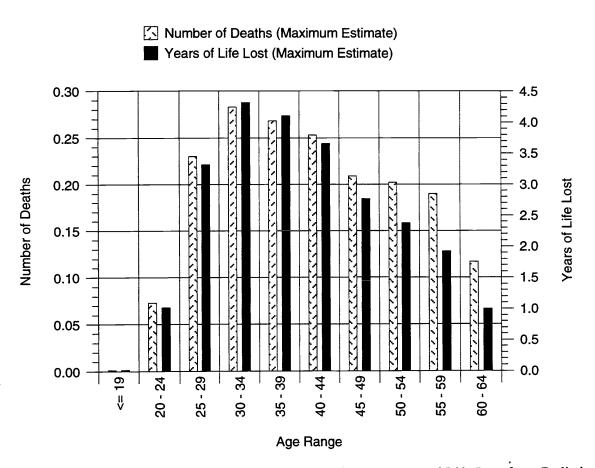


FIGURE 7.1. Estimated Maximum Number of Total Deaths and Years of Life Lost from Radiation Doses Received at DOE Facilities by Age Group in 1991—(The values indicated are maximum estimates; the actual values may be zero. See text for explanation.)

The values were calculated directly from the BEIR V risk equations and the doses received by employees and visitors at DOE facilities in 1989. Applying a DREF to these values would be appropriate (NAS 1990; UNSCEAR 1988) and would reduce the values by a factor of two or more. Furthermore, the BEIR V risk estimates were based on studies of individuals who received high doses. Consequently, the actual number of deaths and years of life lost from doses received at DOE

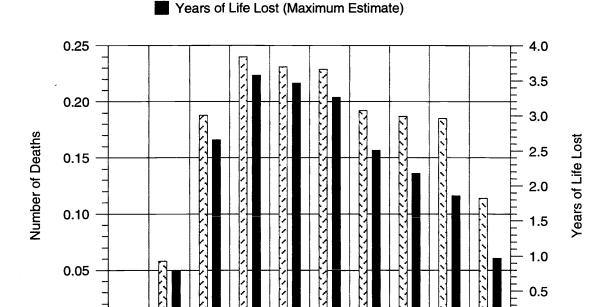
facilities may be zero. Figures 7.2 and 7.3 show the risk distribution by age range and sex. Because of their higher average dose, males in all age groups had higher risk values than females. Males between the ages of 30 and 44 had the highest estimated risk values.

Figures 7.4, 7.5, and 7.6 show risk values by facility type for all DOE/DOE employees, male employees, and female employees, respectively. The highest risk values were associated with weapons fabrication and testing facilities for male and female employees. The lowest risk values were obsrved at fusion research facilities. Similar risk trends were seen for male and female employees across all facility types.

Risk values are given by occupation type in Figures 7.7, 7.8, and 7.9. Again, the values for both sexes are shown followed by data for male and female employees. Technicians had the highest risk values for both sexes. Agriculture employees had the lowest values. Again, similar trends for both sex types were observed for all occupation types.

To put into perspective the calculated risks from ionizing radiation doses received at DOE facilities, it is important to review the risks associated with other activities. The primary purpose of this review is to indicate the effect of radiation doses received at DOE facilities on the health of workers relative to the effects of other hazards. Table 7.2 lists the estimated annual deaths per 100,000 persons in the U.S. population for various hazards.

As indicated in Table 7.2, reducing radiation doses at DOE facilities is only one way to improve the health of workers. Other effective methods may include anti-smoking campaigns, increased safety awareness, and the promotion of safe driving practices. Radiation doses received at DOE facilities do not significantly reduce the overall health or life expectancy of workers relative to the other risks encountered both in the workplace and as a part of everyday life.



Number of Deaths (Maximum Estimate)

FIGURE 7.2. Estimated Maximum Number of Total Deaths and Years of Life Lost from Radiation Doses Received at DOE Facilities by Age Group for Male Employees in 1991—(The values indicated are maximum estimates; the actual values may be zero. See text for explanation.)

35 - 39

40 - 44

Age Range

45 - 49

50 - 54

55 - 59

60 - 64

25 - 29

30 - 34

0.00

<= 19

Number of Deaths (Maximum Estimate)

Years of Life Lost (Maximum Estimate)

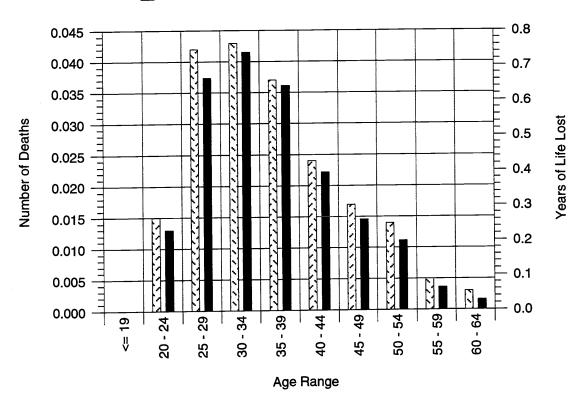
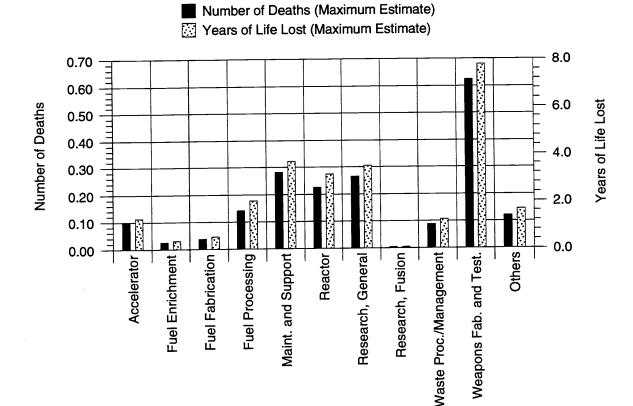
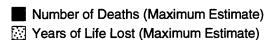


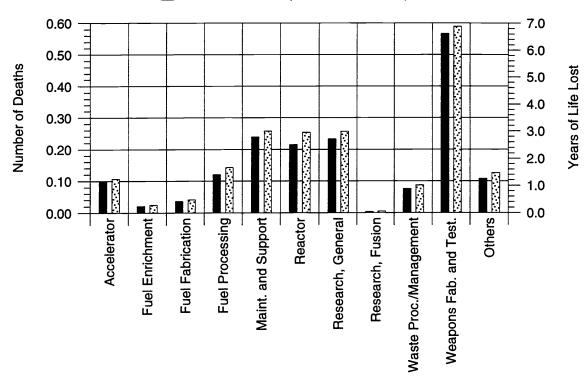
FIGURE 7.3. Estimated Maximum Number of Total Deaths and Years of Life Lost from Radiation Doses Received at DOE Facilities by Age Group for Female Employees in 1991—
(The values indicated are maximum estimates; the actual values may be zero. See text for explanation.)



Facility

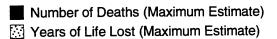
FIGURE 7.4. Estimated Maximum Number of Total Deaths and Years of Life Lost from Radiation Doses at DOE Facilities for All Employees in 1991—(The values indicated are maximum estimates; the actual values may be zero. See text for explanation.)

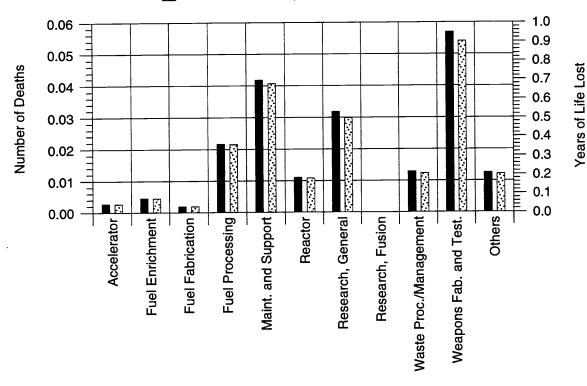




**Facility** 

FIGURE 7.5. Estimated Maximum Number of Total Deaths and Years of Life Lost from Radiation Doses at DOE Facilities for Male Employees in 1991—(The values indicated are maximum estimates; the actual values may be zero. See text for explanation.)





**Facility** 

FIGURE 7.6. Estimated Maximum Number of Total Deaths and Years of Life Lost from Radiation Doses Received at DOE Facilities for Female Employees in 1991—(The values indicated are maximum estimates; the actual values may be zero. See text for explanation.)

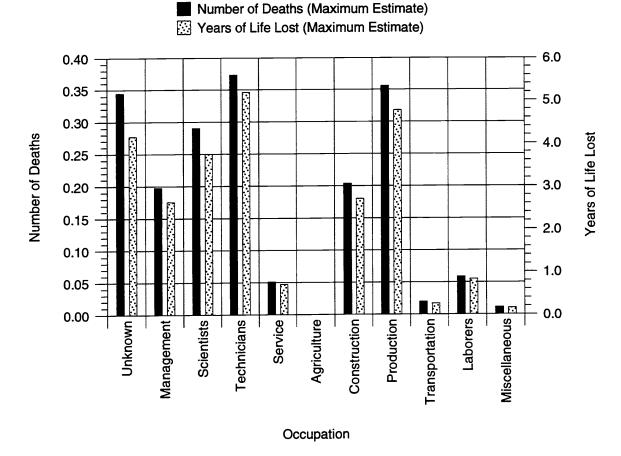


FIGURE 7.7. Estimated Maximum Number of Total Deaths and Years of Life Lost from Radiation Doses Received by Occupation Group (all employees) at DOE Facilities in 1991—(The values indicated are maximum estimates; the actual values may be zero. See text for explanation.)

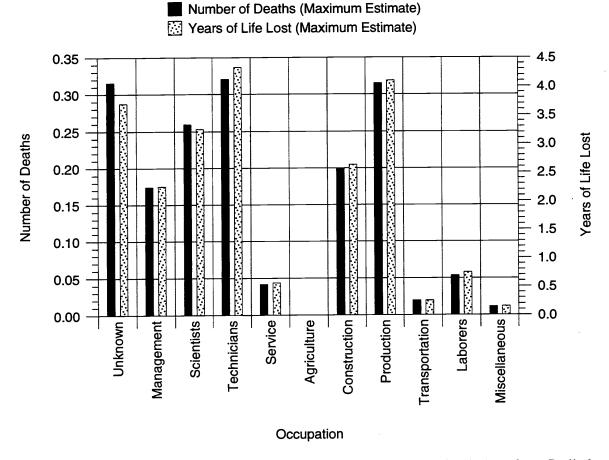


FIGURE 7.8. Estimated Maximum Number of Total Deaths and Years of Life Lost from Radiation Doses Received by Occupation Group (male employees) at DOE Facilities in 1991—(The values indicated are maximum estimates; the actual values may be zero. See text for explanation.)

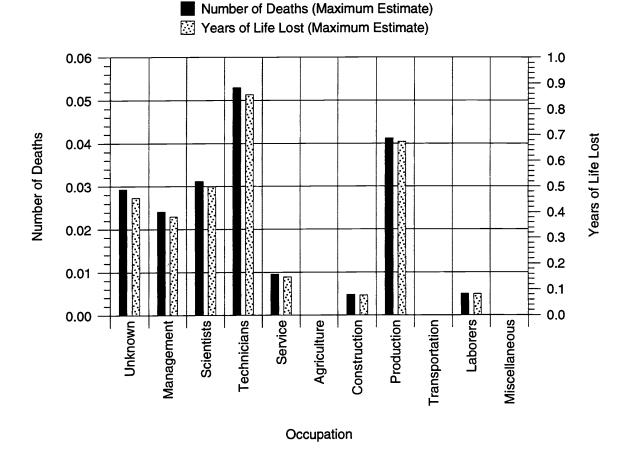


FIGURE 7.9. Estimated Maximum Number of Total Deaths and Years of Life Lost from Radiation Doses Received by Occupation Group (female employees) at DOE Facilities in 1991—(The values indicated are maximum estimates; the actual values may be zero. See text for explanation.)

TABLE 7.2. Estimated Annual Fatality Rates in the U.S. Attributable to Various Causes (a)

	Annual Number of Deaths per 100,000 People or Workers
Cause	per 100,000 reopie of workers
General Population	222
All causes	882
Heart disease	311
Cancer, all types	197
Lung cancer	56
Leukemia	7
Other cancer types	4
Accidents, all types	40
Motor vehicle accidents	20
Other accidents	20
Human Immunodeficiency Virus Infection	7
Other causes	327
Occupational	4.)
Industrial injuries and illnesses	4.8 <sup>(b)</sup>
Highway vehicles	1.6
Industrial vehicles or equipment	0.4
Falls	0.4
Heart attacks	0.3
Electrocutions	0.3
Caught between objects other than vehicles	0.3
or equipment	
Assaults	0.3
Aircraft crashes	0.2
Struck by objects other than vehicles	0.2
or equipment	
Explosions	0.2
Gas inhalation	0.1
Fires	0.1
Plant machinery operations	0.1
All other (including contact with carcinogenic	0.1
or toxic substances, drowning, train	
accidents, and various occupational illnesses)	
Estimated cancer fatalities from radiation doses	
received at DOE facilities	1.9 <sup>(c)</sup>

<sup>(</sup>a) Sources: General population data for the year 1988 from National Center for Health Statistics (1992); occupational data (except cancer fatalities from DOE radiation doses) for the years 1986 and 1987 from the Department of Labor (1989).

<sup>(</sup>b) Ranges from a low of 1.9 per 100,000 in the services industry to a high of 24 per 100,000 in the mining industry.

<sup>(</sup>c) Based on age- and sex-specific risk equations provided in the BEIR V report (NAS 1990). These equations were based primarily on the Japanese atomic-bomb survivor data, which represented acute exposures. The BEIR V committee recognized the need to apply a dose rate effectiveness factor for chronic exposures, which would reduce the risk estimate provided in the table by a factor of at least two. Value indicates deaths per 100,000 DOE workers.

**%** 

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## APPENDIX A

DISTRIBUTION OF ANNUAL TOTAL EFFECTIVE DOSE EQUIVALENT BY FACILITY TYPE FOR EACH FIELD ORGANIZATION, 1991

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TABLE A.1
Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>
Albuquerque Operations
1991

					Dose-E	Dose-Equivalent Ranges (rem)	ent Ra	nges	(rem	_							,
Facility Type	< Meas.	Meas	Meas 0.10- 0.25- < .10 0.25 0.50	0.25-	0.50-	0.50- 0.75-	!	2-3 3	4 4	-5 5-	-99	7 7-1	Total 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons	-10 >	10 P		lotal Person- rem
Accelerator	814	133	32	21	4	4	-								_	1,010	41
Maint. and Support	5,230	581	22	20	2	က	2									5,893	34
Reactor	80	26	9	2	1	2	4									121	6
Research, General	3,924	545	117	29	54	31	32	-	-	1						4,771	179
Research, Fusion	121	15														137	
Waste Proc./Management	980	26	က	2	-	1	-									1,044	ß
Weapons Fab. & Test.	1,884	291	48	24	2		-									2,250	26
Other	3,347	260	11	9	-											3,625	11
Visitors	2,058	370	22	4	2	2	2	2			1		2 1	-	1	2,471	82
DOE Offices	27															27	
Total Persons	18,495	2,274	295	146	29	43	46	၊ မ က		¦ ¦			2 1	1 1 1 1		21,379	! ! ! !
Total Person-rem		62	46	51	41	38	09	7	က	4	9	6 15	5 10	32	2		389

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE A.2
Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>
Chicago Operations
1991

				_	Jose-E	quivale	Dose-Equivalent Ranges (rem)	1	Total
Facility Type	< Meas.	Meas < .10	Meas 0.10- 0.25- 0.50- < .10 0.25 0.50 0.75	0.25-	0.50-	0.75-	1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons		Person- rem
Accelerator	2,739	682	114	52	23	ī.	3	3,619	79
Fuel Fabrication	47	10	က	1				61	-
Maint. and Support	1,467	171	18	4		1	1	1,663	11
Reactor	288	121	22	9				437	10
Research, General	2,055	206	47	16	5	1	2 2	2,332	56
Research, Fusion	504	142	80					654	2
Waste Proc./Management	82	5	4	1				36	-
Other	က	14	H					18	П
Visitors	1,575	893	51	9	4		1 2	2,530	38
DOE Offices	85	2	 	 	1 1 1 1	1 1 1		87	1 1 1 1
Total Persons	8,845	2,246	268	98	34	7	7	11,493	
Total Person-rem		68	40	30	20	9	8		173

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE A.3
Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>
DOE Headquarters
1991

		Dose-Equivalent Ranges (rem)	
Facility Type	Meas10	0.10- 0.25- 0.50- 0.75- 0.25 0.50 0.75 1.00 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons	Total Person- rem
Reactor	28	58	
Research, General	16	16	
Waste Proc./Management	15	15	
Weapons Fab. & Test.	12	12	
Other	23	23	
DOE Offices	713	857	
Total Persons	807	65	† ! !
Total Person-rem			

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE A.4
Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>
Idaho Operations
1991

				٥	ose-Eq	uivale	Dose-Equivalent Ranges (rem)		Total
Facility Type	< Meas.	Meas	Meas 0.10- 0.25- 0.50- 0.75- < .10 0.25 0.50 0.75 1.00	0.25-	0.25- 0.50-	0.75-	Total 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons		Person- rem
Fuel Processing	1,995	212	09	55	17	2	7	2,351	61
Maint. and Support	174	54	2					230	2
Reactor	741	146	61	56	4	4	1	983	33
Research, General	692	47	6		1	-		750	4
Waste Proc./Management	202	21	1					224	1
Other	2,308	305	20	2		2	1	2,668	22
Visitors	14	91	26	20	20	5	17	193	54
DOE Offices	3	1	\$ ! ! !	1 1 1 1	1 1 1			3	1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1
Total Persons	6,129	873	209	106	42	17	26	7,402	
Total Person-rem		30	35	38	26	15	34		177

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE A.5
Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>
Nevada Operations
1991

				_	)ose-Equi	valen	Dose-Equivalent Ranges (rem)	To+01	
Facility Type	< Meas.	Meas	0.10-	0.25-	Meas 0.10- 0.25- 0.50- 0.75- < .10 0.25 0.50 0.75 1.00	75-	Meas 0.10- 0.25- 0.50- 0.75- < Meas. < .10 0.25 0.50 0.75 1.00 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons rem	Total Person- ersons rem	
Accelerator	1,003	24	11	-			1,039	က	
Weapons Fab. & Test.	66	2					101		
Visitors	7	1	н					6	
DOE Offices	47	1 1 1 1		!	! !	1	.4	1	
Total Persons	1,156	27	12	-			1,196	<b>~</b>	
Total Person-rem		П	2					က	

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE A.6
Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>
Oak Ridge Operations
1991

				_	Dose-E(	quivale	Dose-Equivalent Ranges (rem)	To+01
Facility Type	< Meas.	Meas 0.10- 0.25- 0.50- 0.75- < .10 0.25 0.50 0.75 1.00	0.10-	0.25-	0.50-	0.25- 0.50- 0.75- 0.50 0.75 1.00	Total 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons	<u> </u>
Fuel/Uran. Enrichment	6,941	1,636	31	2			8,610	0 32
Fuel Fabrication	932	259	89	14	1	က	1 1,278	8 27
Fuel Processing	11	41	21	6			80	82 8
Research, General	5,549	273	78	28	2	1	2 5,936	6 39
Waste Proc./Management	200	12					212	2
Weapons Fab. & Test.	5,803	1,374	88	2			7,207	7 23
Visitors	2,314	743	43	24	6	9	3,142	.2 42
Total Persons	21,750	21,750 4,338	269	79	15	10	6 26,467	7:
Total Person-rem		77	42	27	6	6	8	172

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE A.7
Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>
Pittsburgh N.R. Office
1991

				_	ose-Equiva	Dose-Equivalent Ranges (rem)	Total	
Facility Type		Meas	0.10-	0.25- 0.50	Meas 0.10- 0.25- 0.50- 0.75- < .10 0.25 0.50 0.75 1.00	Meas 0.10 0.25 0.50 0.75 1.00 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons rem	Person	<u>.</u> !
Reactor	101	367	21	21	. e	513		19
Research, General	208	926	154	53	က	1,374		63
0ther	18	30	1	2		51		-
Visitors	165	86	9			257		2
Total Persons	492	492 1,439 182	182	9/	9	2,195		!
Total Person-rem		25	25 30	27	က		~	84

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE A.8

Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>

Richland Operations
1991

					Dose-Ec	quivale	ent Rar	Dose-Equivalent Ranges (rem)		To+01
Facility Type	< Meas.	Meas 0.10- < .10 0.25	0.10-		0.25- 0.50- 0.75- 0.50- 0.75- 0.50 0.75 1.00	0.75-	1-2 8	Total 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons	tal F sons	Person- rem
Accelerator	ĸ	4							6	
Fuel Fabrication	13	8	က						24	1
Fuel Processing	22	13	9	4	4	2	-		52	80
Maint. and Support	2,843	924	129	74	16	∞	15	1 4.	4,010	108
Reactor	438	195	31	13	4	1			682	19
Research, General	810	433	51	25	13	ω	5	1,	1,345	47
Waste Proc./Management	1,592	662	127	73	10	2	က	2 2,	2,474	81
Other	417	113	11	9	3	2			552	10
Visitors		12	<del>د</del>						15	-
DOE Offices	506	35	: ! ! !	! ! !	! ! !	!	1		241	1
Total Persons	6,346	2,399	361	195	50	26	24	3	9,404	
Total Person-rem		62	57	99	30	22	30	7		275

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE A.9
Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>
Rocky Flats Operations
1991

Total		7,452 885	906 16	8,358	905	nding.
	Total Persons	7,		æ		f rou
	) >10	-		-	15	nse o
	<pre>Meas 0.10- 0.25- 0.50- 0.75- </pre> <pre> &lt; Meas. &lt; .10 0.25 0.50 0.75 1.00 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 &gt;10 Persons </pre>	1		-	∞	values beca
	5-6 6-7 7-8	-	;	-	7	lent
	5 5-6	9	1	9	80	quiva
(rem)	3-4 4-5		; ;		2 97	ose-e
ges (	2-3 3-	68 16 8	 	68 16 8	91 40 26 28	ve d
ıt Ran	1-2 2	89			91	Necti
Dose-Equivalent Ranges (rem)	0.75-	09	; ; ;	09	52	in co
ose-Eq	0.50-0.75-0.75-0.75	115	1	485 116	200 166 70	ations
	0.25-	480	ן ט		166	ır vari
	0.10-	,309	20	1,329	200	oe mind
	Meas 0.10- 0.25- 0.50- 0.75- < .10 0.25 0.50 0.75 1.00	615 4,772 1,309 480 115	780	715 5,552 1,329	200	e may h
	< Meas.	615	100	715		report ther
	Facility Type	Weapons Fab. & Test.	Visitors	Total Persons	Total Person-rem	(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding

TABLE A.10
Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>
San Francisco Operations
1991

					Dose-E	quivale	ent Rai	Dose-Equivalent Ranges (rem)	(m:		
Facility Type	< Meas.	Meas	Meas 0.10- < .10 0.25	0.25-	0.25- 0.50- 0.75- 0.50- 0.75- 0.50 0.75 1.00	0.75-	1-2 ;	2-3 3-4	Total 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons	Total O Persons	Person- rem
Accelerator	733	220	29	10	2					994	16
Fuel/Uran. Enrichment	682	11	1	2	-					701	4
Maint. and Support	3,917	43	12	80	S	1	4			3,991	18
Research, General	1,478	16	13	11	က	4	7			1,592	23
Research, Fusion	341	1								342	
Waste Proc./Management	114	1								115	
Weapons Fab. & Test.	1,289	38	12	2		-				1,342	5
Other	1,302	19	-			2			1	1,325	7
Visitors	19	51	10	2		1				83	4
DOE Offices	134	8	1	 						137	!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!
Total Persons	10,009	463	78	38	11	6	12	1	1	10,622	
Total Person-rem		14	13	13	7	80	16	2	5		11

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE A.11
Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>
Savannah River Operations
1991

					Dose-E	quivale	ent Rar	Dose-Equivalent Ranges (rem)		To+01
Facility Type	< Meas.	Meas	Meas 0.10- 0.25- 0.50- 0.75- < .10 0.25 0.50 0.75 1.00	0.25-	0.50-	0.75-	1-2 8	Total 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons		Person- rem
Fuel Processing	1,306	797	254	111	27	6	2		2,506	128
Maint. and Support	5,438	3,253	396	92	6	2	2	2	9,197	182
Reactor	709	653	38						1,400	21
Research, General	888	264	16	9		1			1,175	10
Waste Proc./Management	751	380	91	28	4	2			1,256	38
Weapons Fab. & Test.	448	128	13	က					592	9
0ther	3,359	1,264	117	99	9	1			4,813	89
Visitors	930	276	2	2					1,213	9
DOE Offices	363	67	1			1 1 1 1			431	1
Total Persons	14,192	7,082	931	311	46	15	4	2	22,583	
Total Person-rem		165	141	104	27	13	9	5		459

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE A.12
Distribution of Annual Total Effective Dose Equivalent by Facility Type<sup>(a)</sup>
Schenectady N.R. Office
1991

					)ose-Ec	quivale	Dose-Equivalent Ranges (rem)	[ה+ם]
	:	Meas.	Meas 0.10 - 0.25 - 0.50 - 0.75 -	0.25-	0.50-	0.75-	Total	а-
Facility Type	< Meas.	.10	0.25 0.50 0.75 1.0	0.50	0.75	1.00	< Meas. < .10 0.25 0.50 0.75 1.00 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons	
Reactor	88	693	46	33			830	20
Research, General	471	396	2				872	9
0ther	7	14					21	
Visitors	198	451	66	100		38	57 1,008	207
Total Persons	764	1,554	150	103	65	38	57 2,731	
Total Person-rem		26	26 25	36	42	33	72	233
(a) Throughout this	report th	ere may	be min	or var	iation	s in co	(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding	ng.

#### APPENDIX B

DISTRIBUTION OF ANNUAL TOTAL EFFECTIVE DOSE EQUIVALENT BY CONTRACTOR, 1991

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# TABLE B.1 Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup> Albuquerque Operations 1991

					Dose-E	quivale	Dose-Equivalent Ranges (rem)		To+01
Contractor	< Meas.	Meas	0.10-	0.25-	0.50-	0.75-	1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10	Total 0 >10 Persons	Person- rem
Albuquerque Office Subs Employees Visitors	38	2	4	2	 	 		σ.	2
Total	1	2	4	2	!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!!	] ] [		6	2
Albuquerque Transporta Employees Visitors	rtation Division 23	ision 1						24	
Total	23	1	! ! !	 	! !	! ; ! !		24	
Allied-Signal, Inc. ( Employees Visitors	(Bendix Div. 196 1	iiv.)						207	! 1 1 ! !
Total	197	11	}   	! ! ! !	! ! !			208	
EG&G Mound Applied Tec Employees Visitors	Technologies 1,837 42	306 306	13	2			2	2,160	8
Total	1,879	308	13	2	1 1 1 1 1	 	2	2,204	80
<pre>6.E Pinellas Employees Visitors</pre>	241	41	1					283	-
Total	241	41	1					283	1
Inhalation Toxicology Employees Visitors	Research 253	Inst. 3 22 2	5	1	1			282	5
Total	253	3 24	5	1	1			284	2

### TABLE B.1 (continued) Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup> Albuquerque Operations 1991

306 16 1622 22 Total Person-Total rem 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons 1,613 1,613 6,226 6,909 2,255 111 2,366 989 47 47 53 53 989 2 Dose-Equivalent Ranges (rem) 35 40 Meas.- 0.10- 0.25- 0.50- 0.75-< .10 0.25 0.50 0.75 1.00 38 39 ~ 60 2 62 95 56 56 12 12 97 165 15 180 32 32 2 39 39 683 155 838 135 154 135 154 172 172 14 14 5,640 2,034 111 2,145 1,395 548 1,395 Los Alamos National Laboratory Employees 5,145 Visitors 495 548 45 45 33 39 < Meas. Mason & Hanger - Amarillo MK-Ferguson Subs - UMTRA MK-Ferguson Co. - UMTRA Employees Visitors Johnson Controls, Inc. Employees Jacobs-Weston Team Employees Visitors Employees Employees Visitors Visitors Visitors Contractor Total Total Total Total Total Total

TABLE B.1 (continued)

Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup>

Albuquerque Operations
1991

				ŏ	Dose-Equivalent Ranges (rem)	ıivaler	ıt Rang	es (	rem)								- + · + ·
Contractor	< Meas.	Meas 0.10- C < .10 0.25 C	0.10-	0.25- 0.50- 0.50 0.75	).50- ( ).75 1	0.75-	1-2 2-	2-3 3-4	4 4-5	5-6	7 1-9	7-8 8-	8-9 9-1	9-10 >10		Total P Persons	Person- rem
Mason & Hanger - Los Alamos Employees Visitors	amos 412	31		i !		! !	! ! !	!	! ! !	! ! !	!	i ! !	1	! !		443	1
Total	413	31			i !	; ;	1	!	1	1		i 	<u> </u>	,	1	444	1
Ross Aviation, Inc. Employees Visitors	78	10							:			1		; ;	i i	88	 
Total	78	10	1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	1	 	t 1 1	i ! !	! !	1 1 1	! !	! !					88	
Sandia National Laboratory Employees Visitors	tory 3,033 1,201	291 207	12	8 3	-1	2	4					1	1		1	3,346	17 8
Total	4,234	498	19			. 6	4	<u> </u>	!	!	! !	! !	! ! !	! !	   	4,764	56
Westinghouse (WIPP) Employees Visitors	803 165	17								! !			; ;	! !	! !	820	1 1 1 1 1
Total	896	21	, ! ! ! ! ! ! !		1 1 1 1 1 1 1 1			; ; ;			1	!	1	 	!	986	! 1 1
Albuquerque Operations Total	18,099	2,262	295	146	29	43	46	က	1 1	-	1	2	1.	1 2		20,971	388

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE B.2
Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup>
Chicago Operations
1991

				_	Jose-E	quivale	Dose-Equivalent Ranges (rem)		
Contractor	< Meas.	Meas	0.10-	0.25-	0.50-	0.75-	1-2 2-3 3-4 4-5 5-6 6-7 7-8	Total 8-9 9-10 >10 Persons	Total Person- s rem
Ames Laboratory (Iowa Employees Visitors	owa State) 87	18	Ħ					106	1 1
Total	88	18		! !	! !	 		107	,1
Argonne National Laboratory Employees 3., Visitors	ratory 3,343	266	. 59 3	15	ľ	1	1	3,690 32	30 30
Total	3,343	295	29	15	1 10	-	1	3,722	31
Battelle Memorial Ins Employees Visitors	Institute - Columbus 121 37 24 23	olumbus 37 23	N 60	3	1			168 51	3 4 4
Total	145	09	8	4	1		1	219	9 6
Brookhaven National L Employees Visitors	Laboratory 1,453 522	434 360	101 28	55	21	4	1 5	2,074	4 70 0 19
Total	1,975	794	129	61	25	. 4	9	2,994	4 89
Chicago Office Subs Employees Visitors	39	56	4	2		-		73	3
Total	39	52	4	2	1	-		73	3
Fermilab Employees Visitors	1,436 826	390 448	37 16	4	2			1,869	9 19 0 15
Total	2,262	838	53	4	2			3,159	9 34

TABLE B.2 (continued)

Distribution of Total Effective Dose Equivalent by Contractor<sup>(a)</sup>

Chicago Operations
1991

				۵	ose-Equ	uivalen	Dose-Equivalent Ranges (rem)		Total
Contractor	< Meas.	Meas	Meas 0.10- 0.25- 0.50- 0.75- < .10 0.25 0.50 0.75 1.00	0.25-	0.50- (	). 75-  .00	Total 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons	Total 10 Persons	Person- rem
Mass. Inst. of Tech. Employees Visitors	240 125	47	4					291	2
Total	365	58	4		<b>.</b>	: :		427	2
National Renewable Energ Employees Visitors	Energy Lab (NREL)- CH 13 5	(NREL)- 5	Н					18	1 1 1 1 1
Total	13	5	! ! !	 	i ! ! !	: ! ! !		18	
Princeton Plasma Physic. Employees Visitors	Physics Laboratory 373 1 67	atory 124 22	1			i		503	1 5
Total	440	146	7					593	9
Chicago Operations Total	8,670	2,240	268	98	34	7	7	11,312	172

TABLE B.3
Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup>
DOE Headquarters
1991

	_	-uc	_	!		•	; ; ;		! ! !		
	Total	Perso	re	1 1 1 1			1		1		
Dose-Equivalent Ranges (rem)		Meas 0.10- 0.25- 0.50- 0.75- Total Person-	< .10 0.25 0.50 0.75 1.00 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons			94		94			4D
			< Meas.			94		94		Š	94
			Contractor		DOE Office Subs	Employees Visitors		Total		DOE Headquarters	lotal

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

TABLE B.4
Distribution of Annual Total Effetive Dose Equivalent by Contractor<sup>(a)</sup>
Idaho Operations
1991

				_	Jose-E	quival	Dose-Equivalent Ranges (rem)		To+a1
Contractor	< Meas.	Meas	0.10-	0.25-	0.50-	1.00	1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9	Total 9 9-10 >10 Persons	Person- rem
Babcock & Wilcox Idaho, Employees Visitors	. Inc. 466	30	1					497	-
Total	466	32	1		! ! !			499	1
Chem-Nuclear Geotech Employees Visitors	693	24 5		-				719	1
Total	693	53	1		1 1 1 1			724	-
EG&G Idaho, Inc. Employees Visitors	1,831	249 25	72 3	56	4	4	1	2,187	38
Total	1,834	274	75	26	4	4		2,218	39
Idaho Office Subs Employees Visitors	18 1	1				1		20	- I
Total	19	1	1 ! ! !	: 	1 1 1 1 1	1		21	1
MK-Ferguson Company - Employees Visitors	10 180	20 17	6	ოთ	9	2 2	5 9 9	213	19
Total	180	37	18	12	9	4		592	56
MK-Ferguson Subcontrac Employees Visitors	tractors -10 52 7	34	11	111	14	ε !	11	63	33
Total	59	41	13	12	14	က	12	154	35

TABLE B.4 (continued)

Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup>

Idaho Operations
1991

				۵	ose-Equ	uivaler	Dose-Equivalent Ranges (rem)	- -	-
Contractor	< Meas.	Meas	Meas. 0.10- 0.25- 0.50- 0.75-	0.25-	0.50- (	0.75-	Total Total 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons	Person- ns rem	!
Protection Technology - Employees Visitors	ogy - INEL 391	44						435	- !
Total	391	44	; ; ; ;	: : :	i ! !	i ! !		435	1
West Valley Nuclear Ser Employees Visitors	ar Services, Inc. 720	Inc. 198	44	ო				965	15
Total	720	198	44		1 1 1 1	1 ! ! !		965	15
Westinghouse Idaho Nucl Employees Visitors	Nuclear Co. 1,555	176 8	56	52	18	2	5.11,8	1,867	57
Total	1,558	184	. 56	52	18	1 20 1	1.5	1,878	57
Idaho Operations Total	5,920	840	208	106	42	17	26 7,	7,159	176

# TABLE B.5 Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup> Nevada Operations 1991

				ŏ	ose-Eq	uivale	Dose-Equivalent Ranges (rem)		,
Contractor	< Meas.	Meas	0.10- (0.25 (	0.25- (	0.50-	0.75-	1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >	Total >10 Persons	Person- rem
EG&G Amador Valley Ope Employees Visitors	y Operations 2							8	
Total	2	 	; ; ; ;	i ! !	!	! ! !		2	 
EG&G Las Vegas Employees Visitors	142							142	
Total	142	 	: : : : :	i   	!	!		142	 
EG&G Los Alamos Employees Visitors	2							2	
Total	2	! ! ! ! !	; !	i ! !	 	!		2	1 1 1 1 1 1
EG&G Santa Barbara Employees Visitors	51							51	
Total	51	f f 1 1 1 1	! ! ! !	i !	f 	! ! !		51	 
EG&G Special Technologies Laboratories Employees Visitors	ies Labora 13	atories						13	
Total	13	! ! ! !	i ! !	i ! !	1       	! ! !		13	1 1 2 1 8 8
EG&G Washington D.C. Employees Visitors	o							6	
Total	က         	1 1 1 1 1 1 1	         		  -  - 	! ! !		6	1 1 1 1 1

# TABLE B.5 (continued) Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup> Nevada Operations 1991

					_	Dose-Equivalent Ranges (rem)		Total
Contractor	< Meas		Meas (	0.10-	0.25-	0.50- 0.75- Total 0.75 1.00 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons		Person- rem
	1	:	:	1			! ! ! !	1 
Fenix & Scisson, Inc. Employees Visitors	J	63	2	6	1		78	2
Total		63	. 2	6	-		78	2
Holmes & Narver, Inc., Employees Visitors	ESD	32	-				33	1 8 1 1 1
Total	1	32		: ! !	!		33	
Nevada Miscellaneous Contractors Employees 53 Visitors 1	ontrac	tors 53 1			ļ		53	 
Total	1 8 1 1	54	  -  -				54	
Raytheon Services - Ne Employees Visitors	Nevada	30	-				31	         
Total	! ! !	30		! !	 		31	
Reynolds Elec. & Engr. Employees Visitors	. 60	635 6	18	1			655	1
Total	9	641	18	က			299	П

TABLE B.5 (continued)

Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup>

Nevada Operations
1991

,	lotal Total Person- 9-10 >10 Persons rem	23	23	
Dose-Equivalent Ranges (rem)	lotal			
Equivalent	- 0.75- 1.00 1-			
Dose-	0.25- 0.50 0.50 0.75			
	0.10-	<b>≥</b>		
	Meas < .10	Corp	1	
	< Meas.	s Internt'l 22	22	
	Contractor	Science Applications Internt'l Corp NV Employees 22 1 Visitors	Total	Nevada Operations

# TABLE B.6 Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup> Oak Ridge Operations 1991

				0	ose-Eq	puivale	Dose-Equivalent Ranges (rem)				Total
Contractor	< Meas.	Meas	0.10-	0.25-	0.50-	0.75-	1-2 2-3 3-4 4-5	5-6 6-7 7-8 8-9	9 9-10 >10	Total Persons	Person- rem
				1 1 1 1	1	!	; ; ; ; ; ; ;	1 1 1 1 1 1 1 1		. 1	
Becntel National, INC. Employees Visitors	- (103K) 68 211	,, ) 8 26	-							238	-
Total	279	34	1	1 1 1				1 1 1 1 1 1 1 1 1	1 1 1 1 1 1 1	314	2
M.M. Portsmouth Subcontractors Employees Visitors 404	tractors 404	81	က			'				488	2
Total	404	81	3	 	: ! ! !					488	2
Martin Marietta (K-25) Employees Visitors	3,511 117	341 69								3,852	1
Total	3,628	410	!	! ! !	 			1		4,038	2
Martin Marietta (ORNL) Employees Visitors	5,410	267	77	28 8	5		5	i 1 1 1 1 1	1	5,790	39
Total	5,688	306	83	36	9	2	2			6,123	45
Martin Marietta (Paducah) Employees Visitors	cah) 1,780 2	215	80					1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	; ;	2,003	5
Total	1,782	2 240	8	1 	1 1 1 1 1					2,030	2
Martin Marietta (Ports Employees Visitors	Portsmouth) 1,650	0 1,080	0 23	2			; ;	 		2,755	25
Total	1,652	2 1,080	0 23	2	1 1 1	 				2,757	, 25

# TABLE B.6 (continued) Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup> Oak Ridge Operations 1991

					Dose-Ec	quivale	Dose-Equivalent Ranges (rem)		Total
Contractor	< Meas.	Meas	- 0.10- 0.25	0.25-	0.50-	0.75-	1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10	Total Persons	Person-
Martin Marietta (Y-12) Employees Visitors	5,803	3 1,374 7 293	28	2 1	,   	! ! ! !		7,207	23 4
Total	5,860	1,667	32	3	1 1 1 1	1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1		7,562	28
Morrison-Knudsen (WSSRAP) Employees Visitors	AP) 132 257	4						136	
Total	389	9 4	 	! ! ! !	 	[         		393	
Oak Ridge Inst. for Sci Employees Visitors	. & Ed 13	uc. (ORISE) 9 6 4 62	SE)					146	-
Total	143	89 88	1	1 	! ! ! !	 		212	1
RMI Company Employees Visitors	111	1 41 3 1	21	თ				82	εο <u>:</u>
Total	14	4 42	21	6	! ! ! !	! 		98	80
Westinghouse Environ. I Employees Visitors	Mgmt. Co. 932 979	o. of Ohio 2 259 9 147	nio 68 29	14 15	8	വ	3	1,278	27 26
Total	1,911	1 406	97	29	6	ω ;	4	2,464	54
Oak Ridge Operations Total	21,750	0 4,338	3 269	79	15	10	9	26,467	172

TABLE B.7

Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup>

Pittsburgh N.R. Office
1991

				_	)ose-Eq	uivale	Dose-Equivalent Ranges (rem)	[0+0]	
Contractor	< Meas.	Meas	0.10-	0.25-	Meas 0.10- 0.25- 0.50- 0.75- < .10 0.25 0.50 0.75 1.00	0.75-	Total 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons	Person-	
Westinghouse Electric (BAPL) Employees 1:	(BAPL) 196 164	758	92	14	i ! ! !	1 ! !	994	t 20 5 2	C 01
Total	360	844	32	14		1	1,250	) 22	
Westinghouse Electric (NRF) Employees Visitors	(NRF) 104	537	145	09	9		852	2 61	- 1
Total	104	537	145	09	9	 	852	2 61	_
Westinghouse Plant App Employees Visitors	Apparatus Division 18 30 1	ivision 30	1	2			51		- I
Total	19	30		2			<u></u>	52	- 1
Pittsburgh N.R. Office Total	e 483	1,411	178	76	9		2,154	4 83	m

Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup> Richland Operations TABLE B.8 1991

Total Person-6,296 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons 1,508  $\frac{1,135}{2}$ 6,304 1,513 1,137 151 58 58 151 Dose-Equivalent Ranges (rem) က 2 2 12 12 0.50- 0.75-0.75 1.00 œ œ σ 6 6 6 23 23 15 15 12 12 Meas.- 0.10- 0.25-< .10 0.25 0.50 147 147 27 27 21 21 255 3 258 28 44 58 44 Hanford Environmental Health Foundation 478 5 279 1,558 5 483 14 Const 4,294 1,563 23 14 281 23 Westinghouse Hanford Service Subs Employees 127 Battelle Memorial Institute (PNL) 916 Kaiser Engineers Hanford - Cost Employees 759 Westinghouse Hanford Services Employees 4,294 759 < Meas. 44 127 Employees Visitors Employees Visitors Employees Employees Visitors Visitors Visitors Contractor Total Total Total Total Total

52

52

53

Total rem 53

(a) Throughout this report there may be minor variatibns in collective dose-equivalent values because of rounding.

n

24

56

20

195

361

6,140 2,364

Richland Operations

168

168

274

9,163

TABLE B.9
Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup>
Rocky Flats Operations
1991

				ā	Dose-Equivalent Ranges (rem)	uivale	nt Ran	ges	(rem)	_					1040
Contractor	< Meas.	Meas	0.10-	0.25-	0.50-	0.75-	1-2 2	2-3 3.	3-4 4-	4-5 5-6	8-7 7-9	8-9 9-10	>10	Total F Persons	Person- rem
EG&G Rocky FLats Services Employees Visitors	es 117	120										1	! !	237	- 1
Total	117	120	 	 	1		! !	! !	i !	! ! !				237	1
EG&G Rocky Flats Employees Visitors	242 100	3,621	1,143	449 5	112	57	29	15	<b>∞</b>	9			-	5,723	803
Total	342	4,401	1,163	454	113	57		15	i : ∞	9	1		1	6,629	820
EG&G Rocky Flats Subcon <sup>:</sup> Employees Visitors	ubcontractors 20	s S	1			1		!	1 1 1	! ! ! !	! ! !	 	!	29	
Total	20	. 80	-	! ! !										59	
J. A. Jones - Rocky Flats Employees Visitors	its 98	489	134	30	က	2	1	-			!	1	1 1 1	758	55
Total	86	489	134	30		5			 					758	55
Wackenhut Services - Ro Employees Visitors	- Rocky Flats 53	its 3 452	58	-						! ! !	1			535	24
Total	53	452	28	1		-	1			1	1 1		1	535	24
Rocky Flats Operations Total	630		5,470 1,326	485	116	09	89	16	∞	9				8,188	006

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

#### TABLE B.10 Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup> San Francisco Operations 1991

10 Ξ Total Total Personrem 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons 80 20 432 432 317 153 28 49 49 79 181 Dose-Equivalent Ranges (rem) S Ŋ Meas.- 0.10- 0.25- 0.50- 0.75-< .10 0.25 0.50 0.75 1.00 n e e 2 2 က က 145 27 Nevada 2 172 22 Energy Technology Engineering Center < Meas. 16 16 75 427 317 427 317 Lawrence Livermore Nat'l Lab. Employees 7, Visitors Lawrence Berkeley Laboratory Employees LLNL Subcontractors Employees Visitors LLNL Plant Services LLNL Security Employees Visitors Employees Employees Visitors Visitors Visitors Contractor Total Total Total Total Total Total

TABLE B.10 (continued)
Distribution of Annual Total Effective Dose Equivalent by Contractor
San Francisco Operations
1991

					<b>a</b>	ose-Εα	Dose-Equivalent Ranges (rem)	ent Ra	) saɓu	rem)				Total
Contractor	< Meas.	Meč	Meas 0.10- < .10 0.25	0.10- (	0.25- 0.50	0.25- 0.50- 0.75- 0.50 0.75 1.00	0.75-	1-2	2-3 3-	4 4-5	5-6 6-7 7-	Total 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons	Total	Person-
Lawrence Livermore Nat Employees Visitors	National Laboratory 8,385 143	aborat	 tory 143	58	23	. o		7	-			,	8,602	43
Total	8,385	1	143	- 82	23	. თ	. 2	7	1	1			8,602	43
Stanford Linear Accelerator Center Employees Visitors	erator Ce 603	enter 3	110	27	თ	2		! !	   	¦ !	1	1 1 1 1 1 1	751	13
Total	603	! ! !	110	27	် ဂ	5							751	13
U. of Cal./Davis, Radi Employees Visitors	Radiobiology Lab -LEHR 20 2	y Lab 0 2		-1					;	1	i ! !	 	21	
Total	22	2	; !	1	! !	1 1 1 1	1 1 1 1 1	 					23	
U. of Cal./SF - Lab of Radiobiology Employees 29 Visitors	f Radiob 23	iolog 9	17						i	;	i ! ! !	   1   1   1   1	30	
Total	2	29			1 ! 1 ! 1 !				i 	i i i	1 1 1	1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	30	
San Francisco Operations Total	ons 9,875	5	460	78	38	11	6	12	1		_		10,485	5 77

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

#### TABLE B.11 Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup> Savannah River Operations 1991

94 94 2 Total Total Person-32 32 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons rem 56 26 4,857 4,857 293 293 1,346 1,346 22 22 232 232 Dose-Equivalent Ranges (rem) 1-2 Meas.- 0.10- 0.25- 0.50- 0.75- < .10 0.25 0.50 0.75 1.00 4 9 9 45 45 22 22 208 208 2 2 28 28 1,946 1,946 486 77 17 486 SR 2,653 2,653 < Meas. American Telephone & Telegraph Employees 25 216 216 773 17 17 158 773 Miscellaneous DOE Contractors Employees Visitors Bechtel Construction - SR Employees Industrial Phases - SR Service America Employees Visitors Diversco Employees Employees Visitors Visitors Contractor Visitors Visitors Total Total Total Total Total Total

TABLE B.11

Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup>

Savannah River Operations
1991

				ŏ	ose-Equ	iivalen	t Rang	Dose-Equivalent Ranges (rem)		-
Contractor	< Meas.	Meas.~ < .10	0.10-	0.25- (	0.50- 0	0.75-	1-2 2-	2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Pt	Total Persons	Person- rem
Southern Bell Tel. & Tel Employees Visitors	l. 18	4							22	 
Total	18	4	 	!	: :	1 1 1	! ! ! !		22	
Univ. of Georgia Ecology Laboratory Employees 88 Visitors	y Labora 88	tory 14		,	·		i		102	
Total	88	14	1 	1 					102	
Wackenhut Services, Inc. Employees Visitors	SR 727	271	48	43			i !		1,089	26
Total	727	271	48	43					1,089	26
Westinghouse S.R. Subco Employees Visitors	Subcontractors 69 925	.s 26 261	5	2					95	9 !
Total	994	287	. 2	2	!	i 1 1	!		1,288	7
Westinghouse Savannah River Co. Employees 8,155 Visitors 5	liver Co. 8,155 5	3,837 15	609	199	36	13	4	2	12,855	296
Total	8,160	3,852	609	199	36	13	4	2	12,875	297
Savannah River Operations Total	ns 13,829	7,015	930	311	46	15	4	2	22,152	458

TABLE B.12
Distribution of Annual Total Effective Dose Equivalent by Contractor<sup>(a)</sup>
Schenectady N.R. Office
1991

					Dose-E	quivale	Dose-Equivalent Ranges (rem)		
Contractor	< Meas.	Meas	0.10-	0.25-	0.50-	0.75-	T-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Pe	Total Pe Persons	Total Person- rem
GE-KAPL - Kesselring Employees Visitors	81 25	546 83	38	2				667 108	15
Total	106	629	38	. 7	1	 		775	15
GE-KAPL - Kesselring - Employees	Electric Boat	Boat							
Visitors	53	299	66	100	92	38	57	711	206
Total	53	299	66	100	65	38	57	711	206
GE-KAPL - Knolls Employees Visitors	460 26	388 36	ιΩ					853 62	9
Total	486	424	. 52			1		915	9
GE-KAPL - Knolls Subs Employees Visitors	7 15	14 5						21 20	
Total	22	. 19		!		!		41	; ; !
GE-KAPL - Windsor Employees Visitors	1 79	144 28	7	-				153 107	J.
Total	80	172	,	-				260	5
Schenectady N.R. Office Total	747	1,543	149	103	65	38	57 2	2,702	233

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.

			•	
Color Calle Color (Color Special Special Special Special Color Color Color Special Color Special Color	1 VVA 2 MATA Application of the resource of the contract of the desire of the contract of the	and the second s	and the second of the Company of the	

#### APPENDIX C

DISTRIBUTION OF ANNUAL TOTAL EFFECTIVE DOSE EQUIVALENT FOR DOE EMPLOYEES AND VISITORS BY DOE ORGANIZATION, 1991

TABLE C.1
Distribution of Annual Total Effective Dose Equivalent for DOE Employees and Visitors by DOE Organization<sup>(a)</sup>
1991

					Dose-Er	quivale	Dose-Equivalent Ranges (rem)		
Organization	< Meas.	Meas	0.10-	0.25-	0.50-	0.75-	1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10	Total Persons	Total Person- rem
Albuquerque Field Office Employees Visitors	ie 196	∞						204	
Total	196	. 00	 	1	1			204	  -  -  -  -
Dayton Area Office Employees Visitors	40							40	
Total	40	 	 	!	 			40	
Kansas City Area Office Employees Visitors	12							12	
Total	12	! ! ! !	 		 			12	 
Los Alamos Area Office Employees Visitors	81	က						84	
Total	83	3	1 	1 	!	1 1 1 1		98	 
Pinellas Area Office Employees Visitors	1							1	
Total	1	1 1 1 1	 	 					 
Kirtland Area Office Employees Visitors	16			•				16	
Total	16	 	 	 	! !	' ! ! !		16	!

TABLE C.1 (continued)

Distribution of Annual Total Effective Dose Equivalent for DOE Employees and Visitors by DOE Organization (a)

						<del></del>	1991	
					Dose-1	Equival	Dose-Equivalent Ranges (rem)	otal
Organization	< Meas.	Meas 0.10- < .10 0.25	0.10	- 0.25	0.25- 0.50- 0.50 0.75	1.00	Total Persons 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons	rson-
UMTRA Project Office Employees Visitors	o						S	;
Total	6	1 1 1 1 1	) 1 1 1 1	! ! ! !			0	
WIPP Project Office Employees Visitors	39	1					40	1 1 1
Total	39	1	i ! !		)         	1 1 1 1	40	1
Albuquerque Operations Total	396	12					408	
Chicago Field Office Employees Visitors	85	2	6.1		,		87	!
Total	85	i i i i	2	! ! !			87	
Environmental Meas. La Employees Visitors	Lab. 32		2	1	i ! ! !	 	34	 
Total	32		2				34	

TABLE C.1 (continued)
Distribution of Annual Total Effective Dose Equivalent for DOE Employees and Visitors by DOE Organization<sup>(a)</sup>
1991

	3			0.25-	0.50-	0.75-								Total Person-
01 yanı (zatı on )	Meas.	. 10	62.0	0.50	0.75	1.00	1-2 2-3	3-4	4-5 5-6	6-7 7-8	8-9	9-10 >10	Persons	rem
New Brunswick Laboratory Employees Visitors	, 48 10	2											50 10	
Total	58	2											09	
Chicago Operations Total	175	9											181	
DOE Headquarters Employees Visitors	713	65											778	
Total -	713	65							!!!!				778	
DOE Headquarters Total	713	65											778	† 
Idaho Field Office Employees Visitors	209	33											243	
Total -	209	33	-										243	
Idaho Operations Total	209	33	1										243	

TABLE C.1 (continued)
Distribution of Annual Total Effective Dose Equivalent for DOE Employees and Visitors by DOE Organization<sup>(a)</sup>
1991

				Dose-Equivalent Ranges (rem)		Total
Organization	< Meas.	Meas	0.10-	0.25- 0.50- 0.75- 0.50 0.75 1.00 1-2 2-3 3-4 4-5 5-6 6	Total 6-7 7-8 8-9 9-10 >10 Persons	Person- rem
Nevada Field Office Employees Visitors	47	1			47	
Total	47	1	!		48	_
Defense Nuclear Agency Employees Visitors	Kirtland AFB 7	and AFB			7	
Total	7	! ! ! ! !			7	
Environmental Protection Agency (NERC) Employees Visitors	ion Agenc 41	y (NERC	<u></u>		41	1
Total	41	1			41	1
Nevada Operations Total	96		1		96	9
Pittsburgh N.R. Office Employees Visitors		9 28	28 4		4	41 1
Total		9 2 6	28 4		4	41 1
Pittsburgh N.R. Office Total		6	28 4		4	41 1

TABLE C.1 (continued)

Distribution of Annual Total Effective Dose Equivalent for DOE Employees and Visitors by DOE Organization<sup>(a)</sup>
1991

					Jose-E	.quival	Dose-Equivalent Ranges (rem)		,
Organization	< Meas.	Meas	0.10-	0.25-	0.50-	0.75-	1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 P		Total Person- rem
Richland Field Office Employees Visitors	206	35						241	1
Total	206	35						241	1
Richland Operations Total	206	35						241	Π
Rocky Flats Office Employees Visitors	85	82	က					170	2
Total	85	82						170	2
Rocky Flats Operations Total	85	82	က					170	2
San Francisco Field Office Employees Visitors	ice 134	က						137	
Total	134							137	
San Francisco Operations Total	s 134	က						137	 

TABLE C.1 (continued)

Distribution of Annual Total Effective Dose Equivalent for DOE Employees and Visitors by DOE Organization<sup>(a)</sup>
1991

				Dose-Equivalent Ranges (rem)	Total
Organization	< Meas.	Meas 0.10- 0 < .10 0.25 0	0.10-	.25- 0.50- 0.75- 1.00 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons 1.00 1-2 2-3 3-4 4-5 5-6 6-7 7-8 8-9 9-10 >10 Persons 1.00 1-2 1-2 1-2 1-2 1-2 1-2 1-2 1-2 1-2 1-2	Person- rem
S.R. Forest Station Employees Visitors	62	15		11	
Total	95	15		7.1	
Savannah River Field Office	fice 301	52		354	
Total	301	52	1	354	4
Savannah River Operations Total	ons 363	29	1	431	_
Schenectady N.R. Office Employees	e 17	111		59	6
Total	17	11 11	1	32	29
Schenectady N.R. Office Total	1	7 11			59

(a) Throughout this report there may be minor variations in collective dose-equivalent values because of rounding.