

Regulatory Guide Update Program - Phase 4

RG	DG	Title	Status
1.4		Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss of Coolant Accident for Pressurized Water Reactors	
1.8		Qualification and Training of Personnel for Nuclear Power Plants	
1.12		Nuclear Power Plant Instrumentation for Earthquakes	
1.33		Quality Assurance Program Requirements (Operation)	
1.39		Housekeeping Requirements for Water-Cooled Nuclear Power Plants	
1.54		Service Level I, II, and III Protective Coatings Applied to Nuclear Power Plants	
1.77		Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized Water Reactors	
1.79		Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water Reactors	
1.82		Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant Accident	
1.89		Environmental Qualification of Certain Electric Equipment Important to Safety for Nuclear Power Plants	
	1151	An Approach for Plant-Specific, Risk-Informed Decisionmaking for Digital Systems	

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RG	DG	Title	Status
1.98		Assumptions Used for Evaluating the Potential Radiological Consequences of a Radioactive Offgas System Failure in a Boiling Water Reactor	
1.101	1174	Emergency Planning and Preparedness for Nuclear Power Plants	
1.110		Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power Reactors	
1.111		Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors	
1.113		Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I	
1.117		Tornado Design Classification	
1.122		Development of Floor Design Response Spectra for Seismic Design of Floor-Supported Equipment or Components	
1.133		Loose-Part Detection Program for the Primary System of Light-Water-Cooled Reactors	
1.134		Medical Evaluation of Licensed Personnel at Nuclear Power Plants	
1.137		Fuel-Oil Systems for Standby Diesel Generators	

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RG	DG	Title	Status
1.143		Design Guidance for Radioactive Waste Management Systems, Structures, and Components Installed in Light-Water-Cooled Nuclear Power Plants	
1.149		Nuclear Power Plant Simulation Facilities for Use in Operator Training and License Examinations	
1.154		Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis Reports for Pressurized Water Reactors	
1.155	1201	Station Blackout	
1.157		Best-Estimate Calculations of Emergency Core Cooling System Performance	
1.159		Assuring the Availability of Funds for Decommissioning Nuclear Reactors	
1.167		Restart of a Nuclear Power Plant Shut Down by a Seismic Event	
1.179		Standard Format and Content of License Termination Plans for Nuclear Power Reactors	
1.180		Guidelines for Evaluating Electromagnetic and Radio-Frequency Interference in Safety-Related Instrumentation and Control Systems	
1.183		Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors	
1.186		Guidance and Examples for Identifying 10 CFR 50.2 Design Bases	

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RG	DG	Title	Status
1.187		Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiments	
	New-005	High-Energy Line Leak-Before-Break Methodology	
	1177		
	New-011	Implementation of Voluntary 10 CFR 50.61 Entry Criteria	
2.2		Development of Technical Specifications for Experiments in Research Reactors	
2.3		Quality Verification for Plate-Type Uranium-Aluminum Fuel Elements for Use in Research Reactors	
2.4		Review of Experiments for Research Reactors	
2.5		Quality Assurance Program Requirements for Research Reactors	
2.6		Emergency Planning for Research and Test Reactors	
3.13		Guide for Acceptable Waste Storage Methods at UF6 Production Plants	
3.14		Seismic Design Classification for Plutonium Processing and Fuel Fabrication Plants	
3.17		Earthquake Instrumentation for Fuel Reprocessing Plants	

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3.31		Emergency Water Supply Systems for Fuel Reprocessing Plants	
3.39		Standard Format and Content of License Applications for Plutonium Processing and Fuel Fabrication Plants	
3.48		Standard Format and Content for the Safety Analysis Report for an Independent Spent Fuel Storage Installation or Monitored Retrievable Storage Installation (Dry Storage)	
3.50		Standard Format and Content for a License Application To Store Spent Fuel and High-Level Radioactive Waste	
3.55		Standard Format and Content for the Health and Safety Sections of License Renewal Applications for Uranium Hexafluoride Production	
3.60		Design of an Independent Spent Fuel Storage Installation (Dry Storage)	
3.61		Standard Format and Content for a Topical Safety Analysis Report for a Spent Fuel Dry Storage Cask	
3.62		Standard Format and Content for the Safety Analysis Report for Onsite Storage of Spent Fuel Storage Casks	
3.63		Onsite Meteorological Measurement Program for Uranium Recovery Facilities - Data Acquisition and Reporting	
3.67		Standard Format and Content for Emergency Plans for Fuel Cycle and Materials Facilities	

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3.71		Nuclear Criticality Safety Standards for Fuels and Material Facilities	
4.4		Reporting Procedure for Mathematical Models Selected To Predict Heated Effluent Dispersion in Natural Water Bodies	
4.7		General Site Suitability Criteria for Nuclear Power Stations	
4.13		Performance, Testing, and Procedural Specifications for Thermoluminescence Dosimetry: Environmental Applications	
4.16		Monitoring and Reporting Radioactivity in Releases of Radioactive Materials in Liquid and Gaseous Effluents from Nuclear Fuel Processing and Fabrication Plants and Uranium Hexafluoride Production Plants	
4.20		Constraint on Releases of Airborne Radioactive Materials to the Environment for Licensees other than Power Reactors	
5.4		Standard Analytical Methods for the Measurement of Uranium Tetrafluoride (UF ₄) and Uranium Hexafluoride (UF ₆)	
5.10		Selection and Usage of Pressure-Sensitive Seals on Containers for Onsite Storage of Special Nuclear Material	
5.13		Conduct of Nuclear Material Physical Inventories	
5.27		Special Nuclear Material Doorway Monitors	
5.28		Evaluation of Shipper-Receiver Differences in the Transfer of Special Nuclear Materials	

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RG	DG	Title	Status
5.51		Management Review of Nuclear Material Control and Accounting Systems	
5.57		Shipping and Receiving Control of Strategic Special Nuclear Material	
5.58		Considerations for Establishing Traceability of Special Nuclear Material Accounting Measurements	
5.67		Material Control and Accounting for Uranium Enrichment Facilities Authorized To Produce Special Nuclear Material of Low Strategic Significance	
6.6		Acceptance Sampling Procedures for Exempted and Generally Licensed Items Containing Byproduct Material	
7.4		Leakage Tests on Packages for Shipment of Radioactive Materials	
7.6		Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels	
7.8		Load Combinations for the Structural Analysis of Shipping Casks for Radioactive Material	
7.11		Fracture Toughness Criteria of Base Material for Ferritic Steel Shipping Cask Containment Vessels with a Maximum Wall Thickness of 4 Inches (0.1 m)	
7.12		Fracture Toughness Criteria of Base Material for Ferritic Steel Shipping Cask Containment Vessels with a Wall Thickness Greater Than 4 Inches (0.1 m) But Not Exceeding 12 Inches (0.3 m)	

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RG	DG	Title	Status
8.2		Guide for Administrative Practices in Radiation Monitoring	
8.4		Direct-Reading and Indirect-Reading Pocket Dosimeters	
8.5		Criticality and Other Interior Evacuation Signals	
8.9		Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program	
8.10		Operating Philosophy for Maintaining Occupational Radiation Exposures as Low as Is Reasonably Achievable	
8.11		Applications of Bioassay for Uranium	
8.13		Instruction Concerning Prenatal Radiation Exposure	
8.15		Acceptable Programs for Respiratory Protection	
8.18		Information Relevant to Ensuring that Occupational Radiation Exposures at Medical Institutions Will Be as Low as Is Reasonably Achievable	
8.19		Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants - Design Stage Man-Rem Estimates	
8.20		Applications of Bioassay for I-125 and I-131	
8.21		Health Physics Surveys for Byproduct Material at NRC-Licensed Processing and Manufacturing Plants	

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8.22		Bioassay at Uranium Mills	
8.23		Radiation Safety Surveys at Medical Institutions	
8.24		Health Physics Surveys During Enriched Uranium-235 Processing and Fuel Fabrication	
8.25		Air Sampling in the Workplace	
8.26		Applications of Bioassay for Fission and Activation Products	
8.27		Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power Plants	
8.28		Audible-Alarm Dosimeters	
8.29		Instruction Concerning Risks from Occupational Radiation Exposure	
8.32		Criteria for Establishing a Tritium Bioassay Program	
8.33		Quality Management Program	
8.35		Planned Special Exposures	
8.36		Radiation Dose to the Embryo/Fetus	
8.37		ALARA Levels for Effluents from Materials Facilities	

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8.38		Control of Access to High and Very High Radiation Areas of Nuclear Plants	
8.39		Release of Patients Administered Radioactive Materials	
10.1		Compilation of Reporting Requirements for Persons Subject to NRC Regulations	
10.12		Preparation of Petitions for Rulemaking Under 10 CFR 2.802 and Preparation and Submission of Proposals for Regulatory Guidance Documents	
	New-025	Maximum Hurricane Wind Speeds	
	5028		