

June 10, 2004

**Summary**

*Track 8: Radiation Protection and Shielding, Nuclear Criticality Safety, and Reactor Physics  
8c: Data, Analysis, and Operations for Nuclear Criticality Safety [NCSD] (C)*

Nuclear Science and Technology Division (94)

## **Impact of Benchmarks on Potential MOX Throughput**

**Sedat Goluoglu and C. M. Hopper**

Oak Ridge National Laboratory,\*  
P.O. Box 2008, MS-6170  
Oak Ridge, TN 37831-6170 USA  
Telephone: (865) 574-5255  
FAX: (865) 576-3513  
Email: goluoglus@ornl.gov

Submitted to the  
*American Nuclear Society*  
*2004 Winter Meeting "Leadership Toward a Progressive, Integrated Nuclear Community*  
*– Going Forward Together",*  
November 14–18, 2004,  
Washington, D.C.

The submitted manuscript has been authored by a contractor of the U.S. Government under contract No. DE-AC05-00OR22725. Accordingly, the U.S. Government retains a nonexclusive, royalty-free license to publish or reproduce the published form of this contribution, or allow others to do so, for U.S. Government purposes.

---

\*Managed by UT-Battelle, LLC, under contract DE-AC05-00OR22725 with the U.S. Department of Energy.

# Impact of Benchmarks on Potential MOX Throughput

Sedat Goluoglu and C. M. Hopper

Oak Ridge National Laboratory, P.O. Box 2008, Oak Ridge, TN 37831-6170, USA [goluoglu@ornl.gov](mailto:goluoglu@ornl.gov)

## INTRODUCTION

The Oak Ridge National Laboratory (ORNL) has been developing specialized sensitivity and uncertainty computational analysis capabilities as part of the U.S. Department of Energy (DOE) Nuclear Criticality Safety Program (NCSP) Task entitled, Applicable Ranges of Bounding Curves and Data (AROBCAD). Within the AROBCAD task is a requirement to implement the developing capabilities to address DOE production/safety issues. The DOE has a vested interest in assuring cost effective, but safe, process operations with poorly-moderated plutonium and uranium mixed oxides (MOX) that will be encountered at the proposed U.S. MOX Fuel Fabrication Facility undergoing U.S. Nuclear Regulatory Commission licensing. In accordance with the ORNL AROBCAD commitment to DOE to apply these new capabilities, ORNL provided computational sensitivity and uncertainty evaluations in support of a recent Organization of Economic and Cooperative Development (OECD) Nuclear Energy Agency (NEA) *Workshop on the need for integral critical experiments with low-moderated MOX fuels* [1]. This workshop was held to address the need for integral experiments involving low-moderated MOX powder systems. Comparative sensitivity and uncertainty computations were performed for weapons-grade plutonium and uranium oxide (WGMOX) and reactor-grade plutonium and uranium oxide (RGMOX) systems as related to available and proposed critical experiments. This paper presents the results of computational studies to examine the sensitivities, uncertainties, and suggested computational limit penalties relative to MOX powder application characteristics provided in Table 1.

## ANALYSIS

The sensitivities of various applications' selected nuclide-reaction pairs were calculated using the TSUNAMI-1D [2] sequence of the SCALE [3] code system. TSUNAMI-1D utilizes the XSDRNPM [4] module to calculate the

adjoint and forward fluxes that are then used to calculate the sensitivity coefficients. The most important nuclide-reaction pairs were determined to be  $^{239}\text{Pu}$ -fission,  $^{240}\text{Pu}$ -capture,  $^{235}\text{U}$ -fission, and  $^{238}\text{U}$ -capture.

Table 1. MOX powder application characteristics

Parameter	RGMOX	WGMOX
PuO <sub>2</sub> content, wt %	12.5 – 30	6.5 – 22
Powder density, g/cm <sup>3</sup>	4.6 – 5.5	5.5
Water content, wt %	3 – 5	1 – 5
Uranium enrichment, wt %	$^{235}\text{U} \leq 1.2$	$^{235}\text{U} \leq 1.2$
Plutonium composition, wt %	$^{240}\text{Pu} \geq 17$	$^{240}\text{Pu} \sim 4$

Comparisons between selected applications and critical experiments were evaluated by using the integral indices  $g$  [5,6],  $c_k$  and  $E_{sum}$  [7]. The proposed experiment contains 30 wt% PuO<sub>2</sub> in the RGMOX (25.2 wt%  $^{240}\text{Pu}$ ) fuel rods that are placed in a 1.07-cm square-pitched 31 by 31 array inside a parallelepiped tank partially filled with water (characteristics of the proposed experiment have been defined in a white paper entitled "Project of Critical Experiments for Nuclear Criticality Codes Validation for Low-moderated 'MOX' Fissile Media" by Institut de Radioprotection et de Surete Nucleaire, Department of Prevention and Studies of Accidents). The comparisons included proposed as well as 95 selected benchmarks from the International Handbook of Evaluated Criticality Safety Benchmark Experiments [8].

## SAFE LIMITS

The effect of computational bias and uncertainty as well as the computational penalty on MOX throughput is provided by determining the variance in subcritical volumes as impacted by an accepted subcritical margin in  $k_{eff}$  for safety. The penalty assessment methodology used in this work is based on the assumption that a benchmark with a greater sensitivity for the

nuclide, reaction and energy group triplet of interest sufficiently covers the triplet in the application. The approach that is used in this method is to determine the differences between the application and benchmark sensitivities for all triplets that are not covered, and to quantify the importance of this noncoverage in terms of its final effect on the  $k_{eff}$  value of the application as a computational penalty after utilizing the cross section uncertainties. The percent decrease in volume of MOX powder as a function of percent safety limit (in units of calculated  $k_{eff}$ ) is shown in Figure 1 for selected RGMOX and WGMOX applications. Assuming the standard deviation of the data that are not in the covariance data file that has been used is 0.15 and standard deviation of  $\chi$  for  $^{238}\text{U}$ ,  $^{241}\text{Pu}$  and  $^{242}\text{Pu}$  is 0.05, computational penalty is 0.3% and 1.0%, respectively (compared against existing benchmarks and the proposed experiment). Corresponding reduction in MOX batch volumes for selected RGMOX and WGMOX applications are 3.9% and 11.5%, respectively. When the standard deviation value is increased from 0.15 to 0.25, the reduction in MOX batch volumes for RGMOX and WGMOX are 7% and 30%, respectively. From these results, it is clear that the bias and uncertainty in the cross sections, or calculated values, or computational penalty affect the volume of safe MOX powder batch considerably. The effects are greater for WGMOX as they are more sensitive to changes in cross sections due to higher  $^{239}\text{Pu}$  content.

## SUMMARY AND CONCLUSIONS

Based on the range of system parameter values that are characteristic for RGMOX and WGMOX, a total of 36 application configurations (corresponding to the ranges in Table 1) have been investigated for determining the applicability of the proposed experiment to the validation of the low-moderated MOX fuel blending operations.

In addition, the status of existing benchmarks for validation of MOX fuel blending processes has been assessed by determining the computational penalty for the nuclide-reaction sensitivities of the MOX powder systems. Even the highest computational penalty is <1% of the calculated  $k_{eff}$  value, because most of the cross sections for the nuclides in these systems are known with small uncertainty.

Larger safely subcritical  $k_{eff}$  limits affect allowable safe MOX powder volumes considerably. Experiments that can provide

good bases for similarity can reduce the computational penalty as well as provide additional sources for reducing bias and uncertainty associated with the calculation of the  $k_{eff}$  of MOX powder applications.

Finally, the tools that have been used to assess the utility of the proposed benchmark can also be used to improve any proposed experiments or evaluate other experiment designs in order to better fit the applications for validation purposes.

## REFERENCES

1. Organization of the Economic and Cooperative Development, Nuclear Energy Agency, Nuclear Science Committee, *Workshop on the need for integral critical experiments with low-moderated MOX fuels*, 14-15 April 2004 OECD Headquarters, Château de la Muette, Paris 16, France.
2. B. T. REARDEN, "Perturbation Theory Eigenvalue Sensitivity Analysis with Monte Carlo Techniques," *Nucl. Sci. Eng.* 146, 367-382 (2004).
3. *SCALE: A Modular Code System for Performing Standardized Computer Analysis for Licensing Evaluation*, NUREG/CR-0200, Rev. 6 (ORNL/NUREG/CSD-2/R6), Vols. I, II, and III, May 2000. Available from the Radiation Safety Information Computational Center at Oak Ridge National Laboratory as CCC-545.
4. *XSDRNPM: A One-dimensional Discrete-ordinates Code for Transport Analysis*, NUREG/CR-0200, Rev. 6 (ORNL/NUREG/CSD-2/R6), Vol. II, Section F3, May 2000. Available from the Radiation Safety Information Computational Center at Oak Ridge National Laboratory as CCC-545.
5. SEDAT GOLUOGLU, C. M. HOPPER, and B. T. REARDEN, "Extended Interpretation of Sensitivity Data for Benchmark Areas of Applicability," *Trans. Am. Nucl. Soc.*, **88**, 77 (2003).
6. SEDAT GOLUOGLU and C. M. HOPPER, *Assessment of Degree of Applicability of Benchmarks for Gadolinium Using KENO V.a and the 238-Group SCALE Cross-Section Library*, ORNL/TM-2003/106), Oak Ridge National Laboratory, December 2003.

7. B. L. BROADHEAD, B. T. REARDEN, C. M. HOPPER, J. J. WAGSCHAL, and C. V. PARKS, "Sensitivity- and Uncertainty-Based Criticality Safety Validation Techniques," *Nucl. Sci. Eng.* **146**, 340-366 (2004).
8. *International Handbook of Evaluated Criticality Safety Benchmark Experiments*, NEA/NSC/DOC(95)03/VI,

Nuclear Energy Agency, Organization for Economic Co-operation and Development, Paris, France (September 2001).

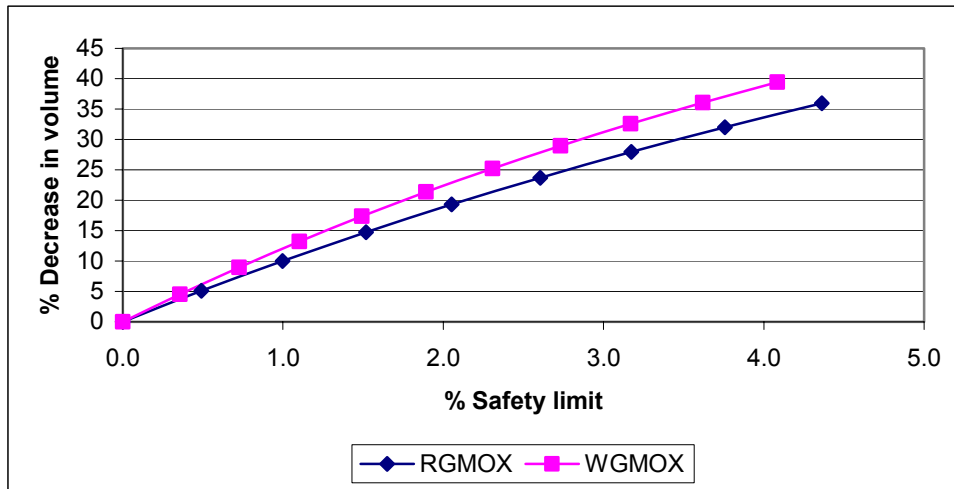


Figure 1. Percent decrease in volume as a function of percent safety limit