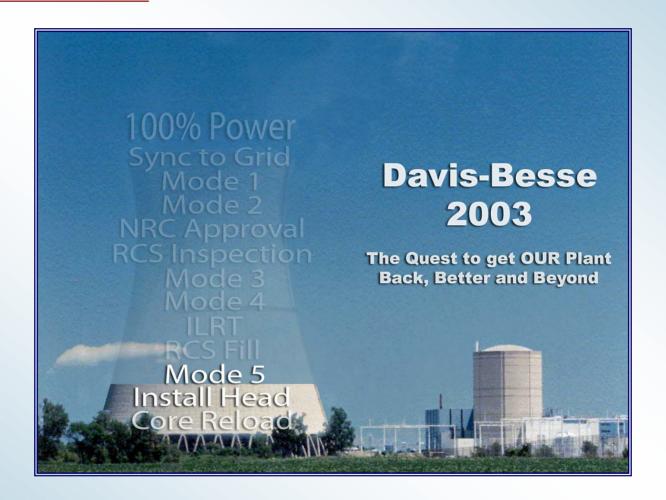
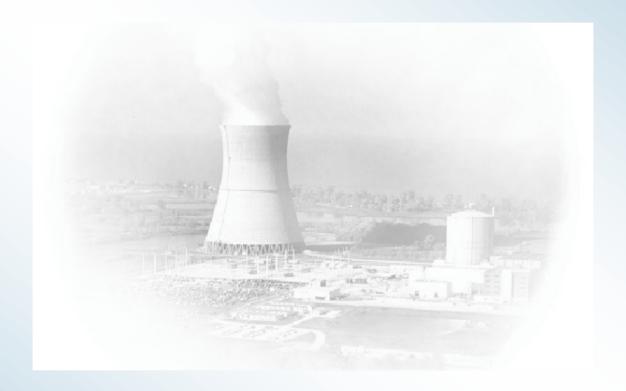
Davis-Besse Nuclear Power Station



IMC 0350 Meeting

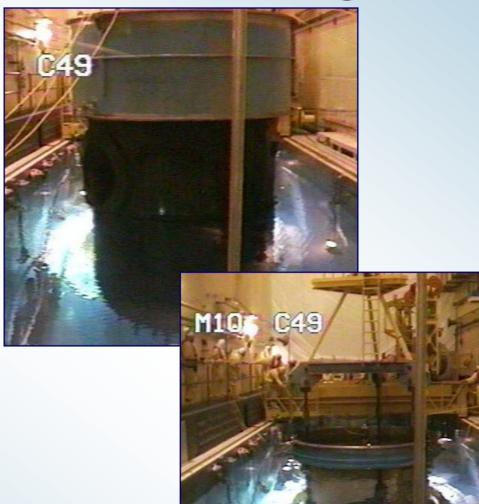
Desired Outcomes

Return to Service Plan Progress - Plant Programs and Scheduled ActivitiesLew Myers - FLUS Monitoring SystemCraig Hengge
Management and Human Performance - Problem Solving/Decision MakingLew Myers - Management Observation ProgramKathy Fehr
Quality Oversight - Safety Culture Survey; Safety Conscious Work Environment Survey; Quality Assurance ObservationsBill Pearce
Return to Service Building Blocks Progress - System Health
Integrated Schedule Progress - Upcoming Activities



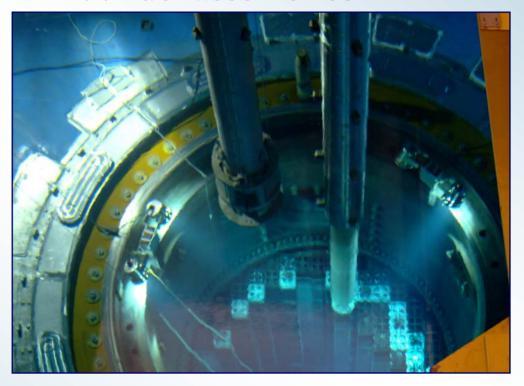
Lew Myers
Chief Operating Officer - FENOC

- Fuel Reload Preparation
 - Core support assembly
 - Upper Plenum
 - Vessel cleaned

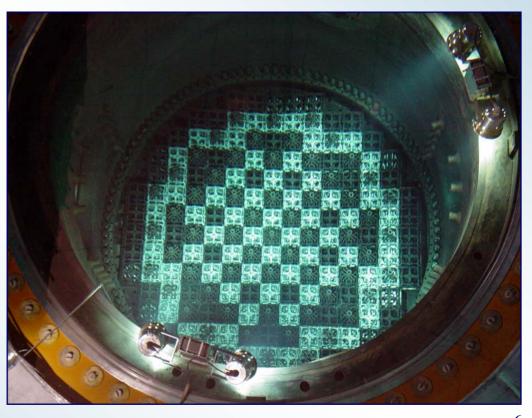




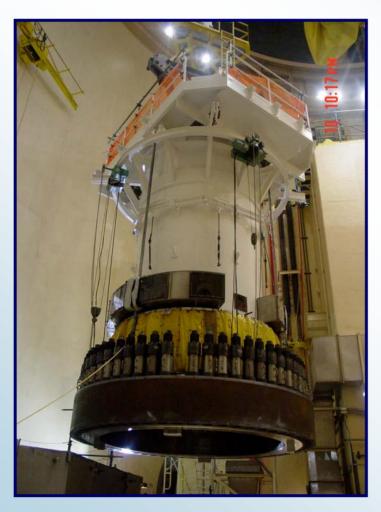
- Fuel Load Activity
 - Began Feb. 19
 - 177 fuel assemblies

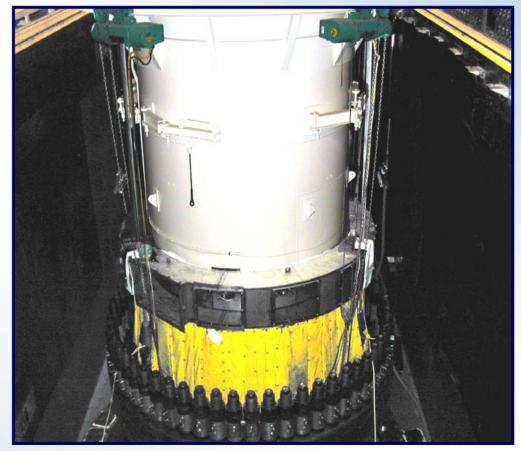


- Fuel Load Completed
 - Feb. 26, 2003



Reactor Head Installation Completed - March 10





- FLUS Under Vessel Monitoring System
 - Installation underway
 - Conduit, cabinets, and tubing



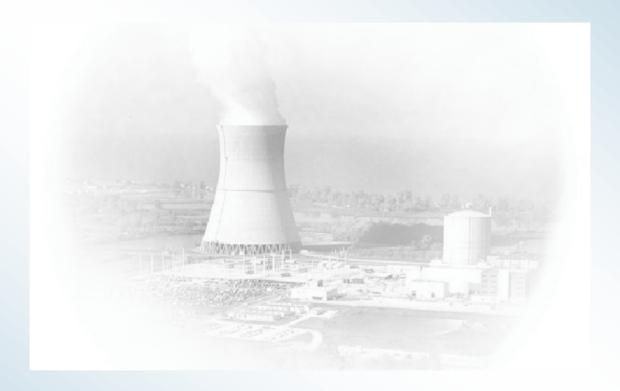
- Safety Features Actuation System Test
- Integrated Diesel Test
- Corrective Action Program Procedure Revision
 - Effective March 1, 2003
- Implementation of Decision Making/Problem Solving Procedure

- Containment Air Coolers Installed
 - Stainless steel coils



- Decay Heat Pit Tank Upgrade
- Mode 6 Restart
 Readiness Reviews
 Conducted





Craig Hengge Engineer - Plant Engineering

Framatome Leak Detection Testing

- Measure Leak Rate as a Function of Simulated Flaw Geometry
 - Identify residue deposit chemistry and any volatile chemicals that exit the crevice
 - Investigate the effect of annulus on leakage rates
- Verify Methods for Detecting Very Small Leaks
 - Visible evidence of boron residue
 - Other detectable residue

Framatome Leak Detection Testing

Test #	Duration	Boron	Leak Rate
1	8 Hours	2680 ppm	.015 gpm
2	8 Hours	2680 ppm	.0015 gpm
3	8 Hours	2680 ppm	.0004 gpm
4	8 Hours	1134 ppm	.0015 gpm
5	55 Hours	2680 ppm	.0006 gpm, 0 gpm after 47 Hrs

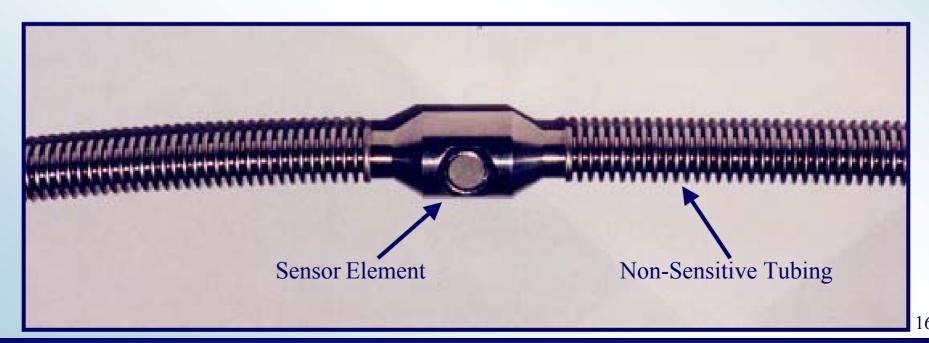
- All Tests Resulted in Visible Residue on Tube and Vessel Surface
- Significant Lithium Deposits Left at Tube/Vessel Surface



- State-of-the-Art System
- 10 Year Operational History
 - 12 installed in Europe and Canada
- Detect and Locate Under Vessel Leakage
 - Located between
 Reactor Vessel
 insulation and bottom of
 Reactor Vessel

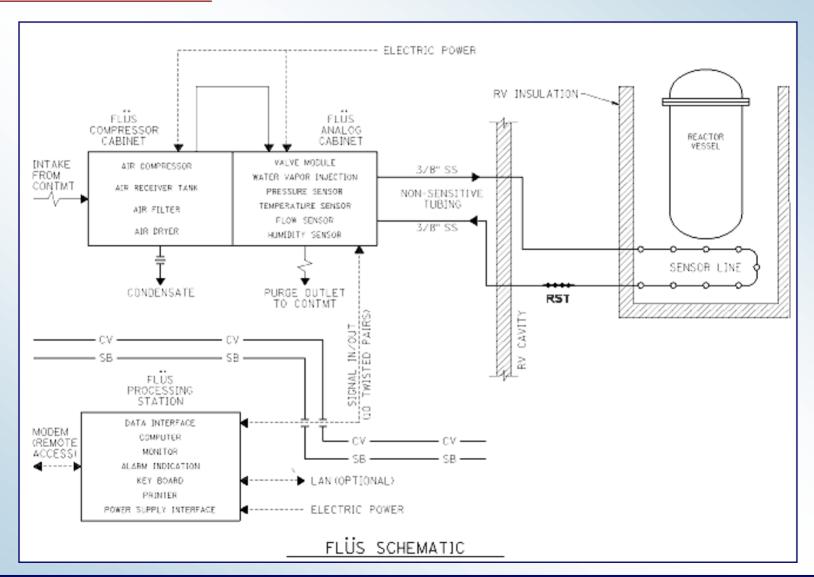
Installation

- Simple installation; conduit, cabinets, tubing
- Tubing is 3/8" diameter
- Approximate three weeks to complete

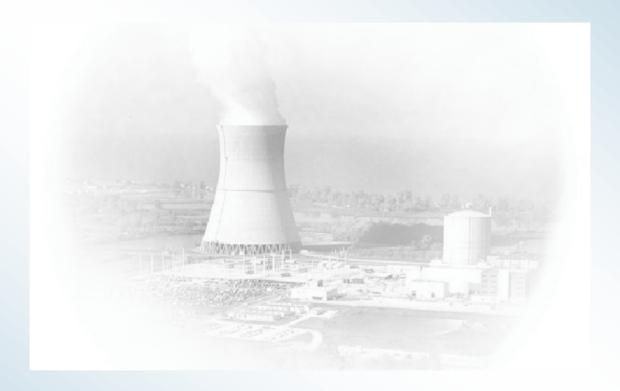


- Eight Channel System
 - Allows future upgrade
- Expected sensitivity:
 .004 to .02 gpm
- FLUS Sensitivity Test During Mode 3





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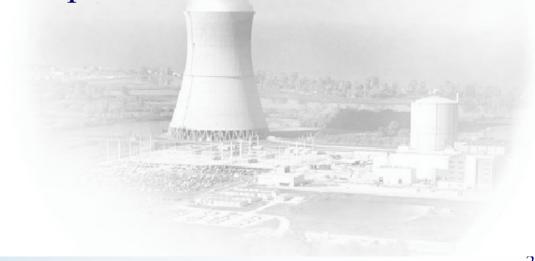


Lew Myers
Chief Operating Officer - FENOC

• Purpose

- Ensure plant issues are addressed consistently and effectively without consequences to plant safety and reliability
- Evaluate significance and potential impact on nuclear safety
- Determine level of management approval based on significance

- Low Significance
 - No personnel or radiological issues
 - Not likely to cause plant damage to components or systems
 - Not likely to effect operations or PSA Risk



- Medium Significance
 - Potential personnel or radiological concerns
 - Without controls could cause damage to plant equipment
 - Controls required to prevent undesirable change of state of components - no plant transients
 - Re-evaluation of PSA Risk

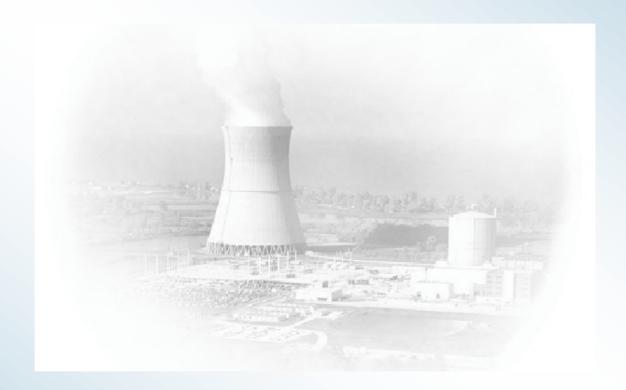
- High Significance
 - Without controls could cause damage to critical plant equipment
 - Could result in personnel or radiological safety issues
 - With proper controls will not result in reactor changes, generator runbacks or power reductions

Six Principles

- Conditions are recognized and reported using the Corrective Action Program
- Roles and responsibilities are clearly established for making and implementing decisions
- Consequences of operational challenges are clearly defined and rigorously evaluated
- Establish full understanding of short term and long term risk
- Develop implementation plans to establish actions,
 compensatory measures and contingencies
- Periodic evaluation of activities

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Management Observation Program



Kathy Fehr **Owner- Management Observation Program**

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Management Observation Program

Purpose

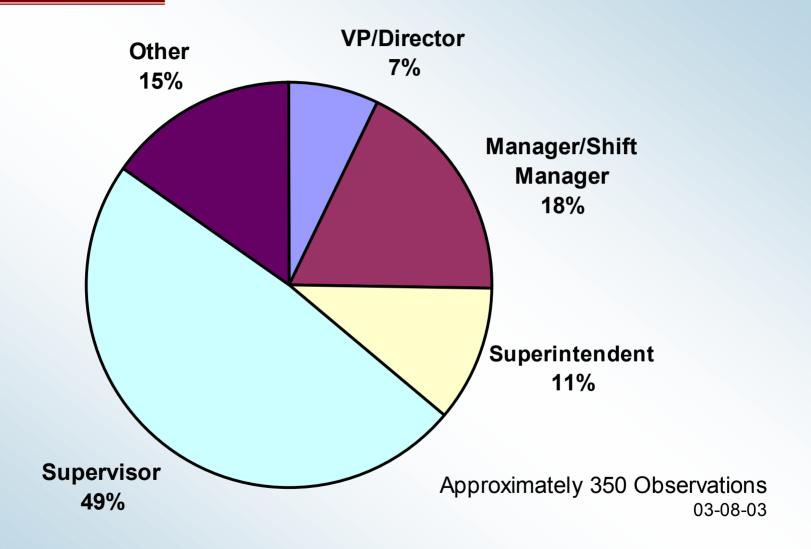
Management oversight on activities and influence desired behaviors

Definitions

- Satisfactory: Meets or exceeds expectations, no comments by the observer
- Satisfactory Coached: Meets or exceeds expectations, comments by the observer
- Unsatisfactory Coached: Provide feedback for area of improvement

Observation Percentage by Title

(February 2003)



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Management Observation Program

- Total Observations
 - 350 Observations
- Scheduled Observations
 - 90% average for February

- Condition Reports
 - 6.21% of February
 observations generated
 condition reports (20)
 - 5.98% for January

Management Observation Program

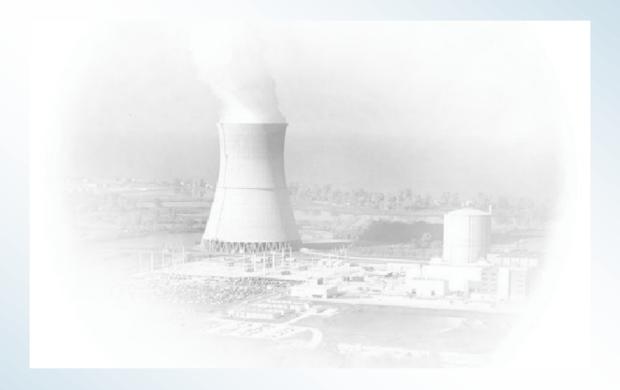
February Field

- Total Coached: 12.2%
 - 9.4% satisfactory coached
 - 2.8% unsatisfactory coached

January Field

- Total Coached: 10.9%
 - 7.9% satisfactory coached
 - 3% unsatisfactory coached

- Management observations have had a positive effective
- Improvement is needed and ongoing



Bill PearceVice President - FENOC Oversight

Safety Culture Assessment

- Activities Completed
 - Interviewed approximately 90 employees
 - Observed normal plant activities
 - Site meetings
 - Departmental interaction
 - Conducted pencil and paper survey
 - 80% participation
- Results Being Analyzed
 - "Convergent Validity"
 - Results within several weeks

Safety Conscious Work Environment

- Periodic Survey to be Conducted
 - Week of March 24
- Majority of Questions Remain the Same
 - Includes questions from NEI 97-05
- Rating Scale Remains the Same
- Survey to be Voluntary and Anonymous
 - Site personnel encouraged to participate
- Results in April

Quality Assurance Observations

- QA Audit Program Review
 - Areas in Need of Improvement
 - Implementation of commitments
 - Audit checklists
 - Use of Operating Experience in auditing
 - Training/qualification of auditors
 - Escalation of inadequate actions to audit findings
 - Interface issues with American Society of Mechanical Engineers (ASME) QA Program

Quality Assurance Observations

- Review Presented to Program Review Board
 - February 10
- Review Board Finding
 - Need for review of ASME QA Audit Program
 - Review Board will re-evaluate Audit Program
 - Completed March 10

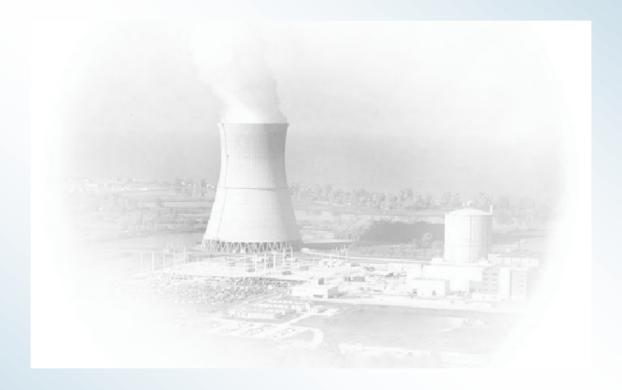
Quality Assurance Observations

- Oversight of New Reactor Head
- Restart Station Review Board
- Fuel Handling
- In-Service Inspection
- Quality Control
- Contractor Control
- Corrective Action
- Safety Culture
- Feedwater Flow Modification
- Quality of Work



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System Health Progress



Bob Schrauder

Director - Support Services

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System Health Progress

Collective Significance Reviews

- System Health Assurance Plan Reviews Identified Potentially Cross Cutting Issues
- Initial Collective Significance Review Identified Five Topical Areas Warranting Further Evaluation:
 - Seismic Qualification
 - Station Flooding
 - High Energy Line Break (HELB)
 - Environmental Qualification (EQ)
 - Appendix R Safe Shutdown Analysis

System Health Progress

Review Process

- Nuclear Operating Business Practice "Collective Significance Review"
 - Consistent process and format
- Evaluation Process
 - Condition Report (CR) database used to identify population of issues to evaluate
 - Bin CRs into common issues areas
 - Evaluate each issue to determine program significance
 - Conduct extent of condition evaluation where warranted

System Health Progress Seismic

- Actions to Support Restart
 - Evaluate impact of Cooling Tower Makeup pumps not in accordance with USAR
 - Resolve boundary conflicts between Seismic and Quality classification
 - HFA relays require adjustment to avoid potential chatter
 - Evaluate impact of boric acid on supports

System Health Progress Seismic

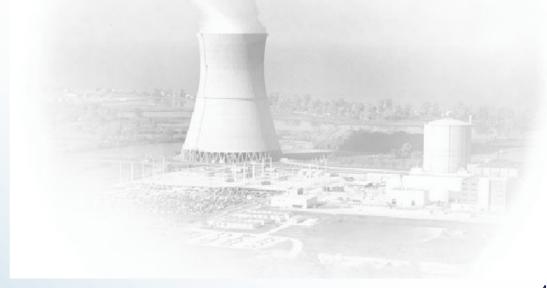
- Future Improvements
 - Eliminate confusion on boundary classifications
 - Improve procedural requirements for control of temporary equipment
 - Create a Seismic Program Summary document
 - Implement the SQUG New and Replacement Equipment program

System Health Progress Station Flooding

- Actions to Support Restart
 - Assess impact of Cooling Tower Makeup Pump seismic categorization on Service Water
 System
 - Evaluate impact of flooding service water tunnel on service water isolation valves to turbine building
 - Verify functionality of critical floor drains

System Health Progress Station Flooding

- Future Improvements
 - Install flood seals in conduit penetrations
 - Implement a formal inspection program for barriers with functions other than Fire Protection



System Health Progress High Energy Line Break

- Actions to Support Restart
 - Complete re-analysis of Turbine Building breaks
 - Revise calculation affecting line breaks/cracks
 - Verify Auxiliary Building free space volumes and openings

System Health Progress High Energy Line Break

- Future Improvements
 - Update model for Auxiliary Building environmental conditions
 - Revise calculations having no impact crack/break location
 - Revise USAR and Design Criteria Manual
 - Compile list of time critical Operator actions and bases and provide to Operations

System Health Progress Environmental Qualification

- Actions to Support Restart
 - Evaluate environmental effects on EQ equipment due to HELB reanalysis
 - Replace vendor supplied splices with EQ
 splices in Containment Limitorque actuators
 - Correct Raychem splice bend radius on four level transmitters
 - Replace containment purge inlet isolation
 solenoid valve due to accelerated thermal aging

System Health Progress Environmental Qualifications

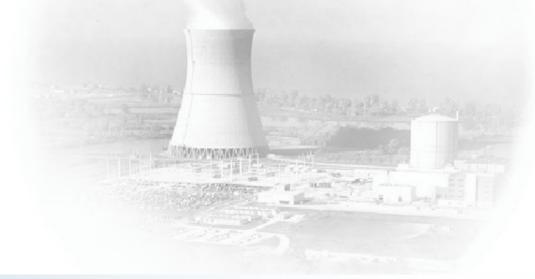
- Future Improvements
 - Revise guidance documents on installation of Limitorque actuators to ensure installation with qualified splices
 - Provide drainage configurations for containment conduit, as necessary
 - Maintenance Matrix of EQ files will be revised to require weep holes at low points of raceways in containment
 - Revise EQ program to provide guidance on recognizing components needing raceway drainage
 - Develop and conduct EQ refresher training

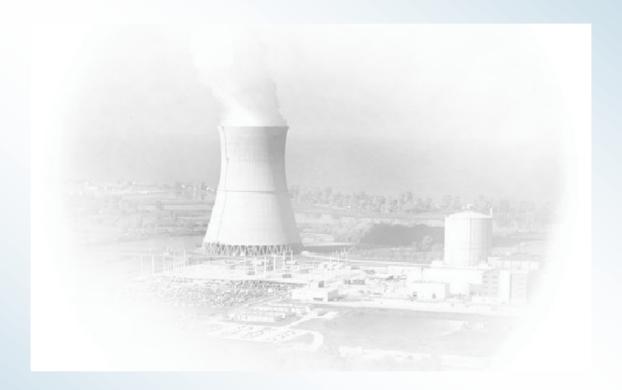
System Health Progress Appendix R - Safe Shutdown Analysis

- Actions to Support Restart
 - Complete Framatome transient calculation upgrade project
 - Complete Flow model of Component Cooling
 Water fire induced valve failure
 - Perform technical review of responses to requests for assistance (RFA)
 - Perform QA Triennial Audit

System Health Progress Appendix R - Safe Shutdown Analysis

- Future Improvements
 - Formalize informal calculation in Requests for Assistance (RFAs) where necessary
 - Complete safe shutdown procedures upgrade project





Lynn Harder

Containment Health Inspection Project Manager

- Ensuring Equipment Supports Safe and Reliable Plant Operation
- Inspection Procedures for Extent of Condition
 - "As found" conditions documented through
 Condition Reports
 - Electrical, structural, mechanical
 - More than 500 components inspected
 - 200 Alloy 600 components inspected

- Independent Evaluators Performed Cause Analysis on More Than 950 Condition Reports
 - More than 6,400 Corrective Actions developed to ensure remediation
 - Rework, replace, refurbish, and clean
 - More than 12,000 assets involved



Containment Air Cooler



Core Flood Tank





Service Water Piping/Support

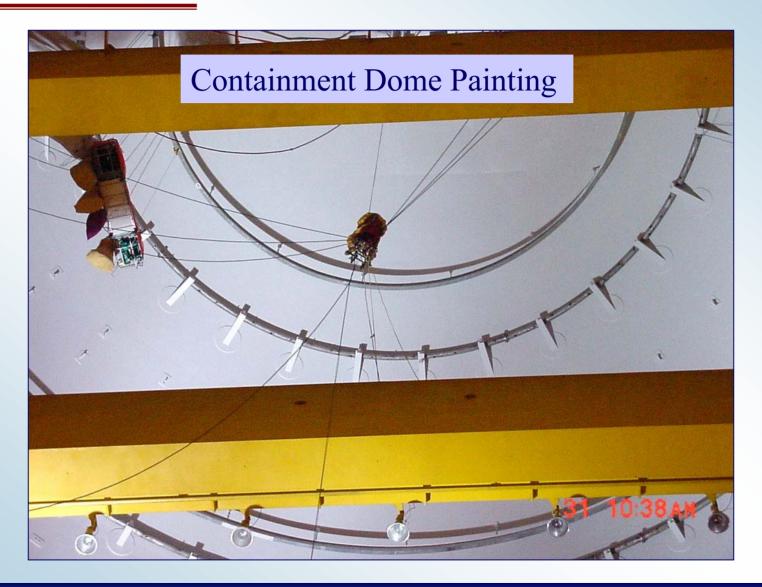
Plenum Sensing Line

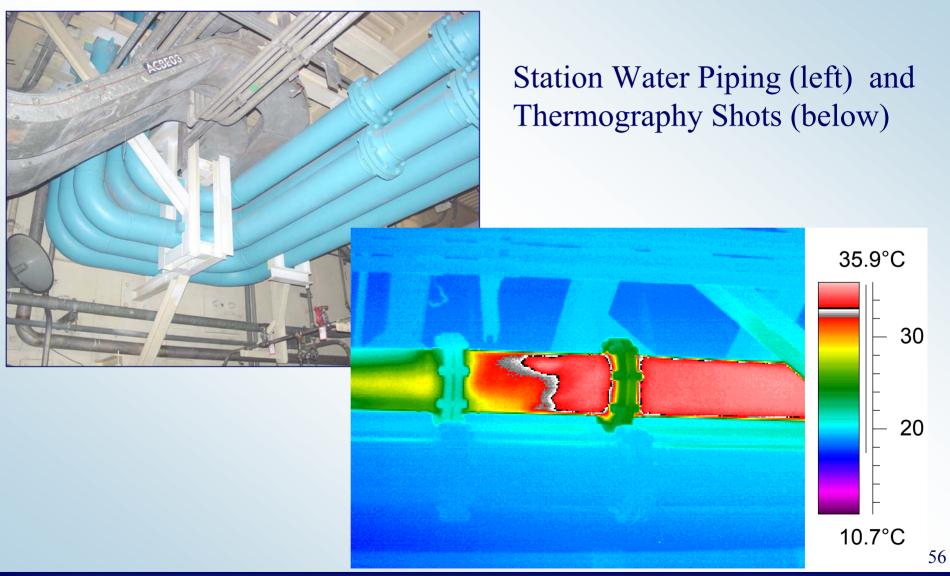


Containment Air Cooler Plenum

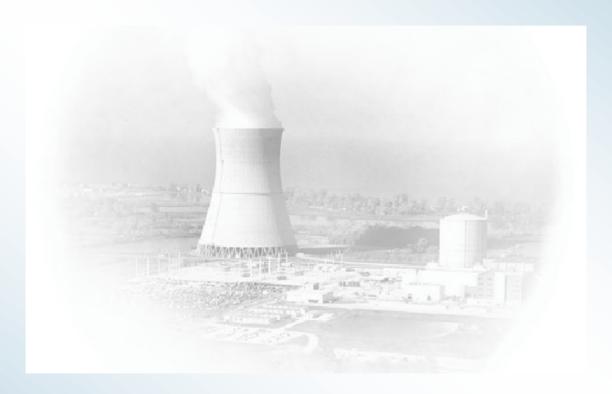


Containment Air Cooler Service Water Piping





- Containment Health Inspection Teams
 - Final "clean-up" of remaining work
- "As Left" Conditions
 - Documented by inspectors after cleaning
 - Assessed by independent evaluators
 - "As left" compared against "as found"
 - Final "as left" inspection
 - Baseline for Boric Acid Corrosion Control Program procedure
 - Provides for a systematic method
- Restart Test Plan
 - Four independent tests verify no Reactor Coolant System leakage



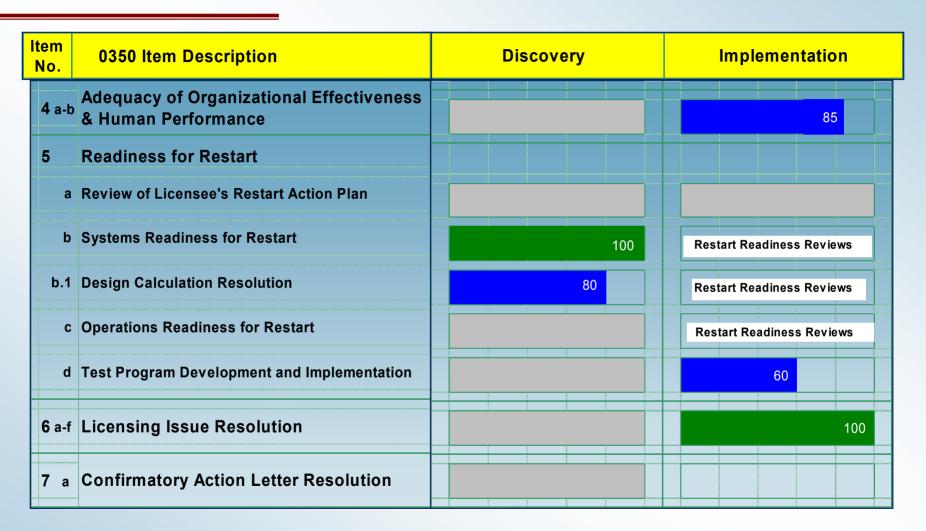
Clark Price
Owner - Restart Action Plan

- 0350 Checklist Items
- Restart Action Progress

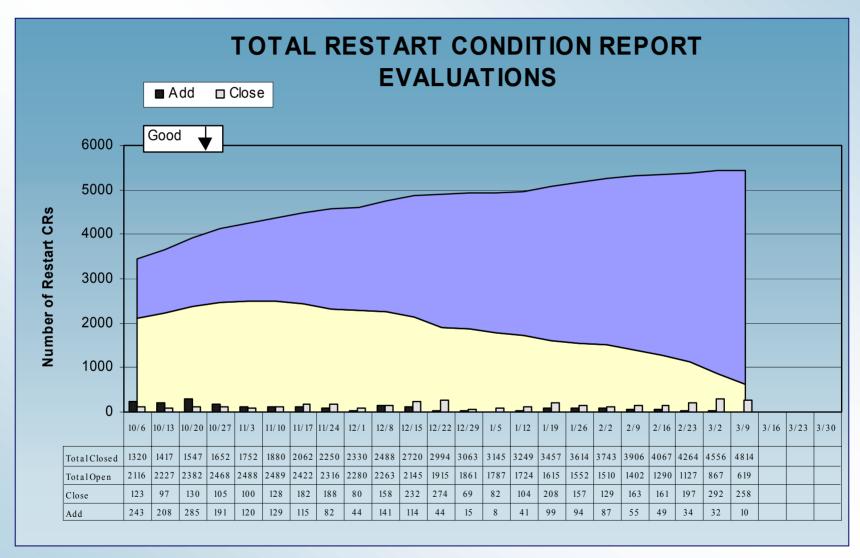
Item No.	0350 Item Description	Discovery	Implementation
1	Adequacy of Root Cause		
а	Penetration cracking and reactor pressure vessel corrosion	Technical Root Cause 02-0891	
b	Organizational, programmatic and human performance issues	95	
2	Adequacy of Safety Significant Structures, Systems and Components		
а	Reactor Pressure Vessel Head Replacement		97
b	Containment Vessel Restoration following RPV Head Replacement		90
С	Structures, Systems and Components Inside Containment	100	78
c.1	Containment Emergency Sump	100	84
d	Boric Acid in Systems Outside Containment	100	72

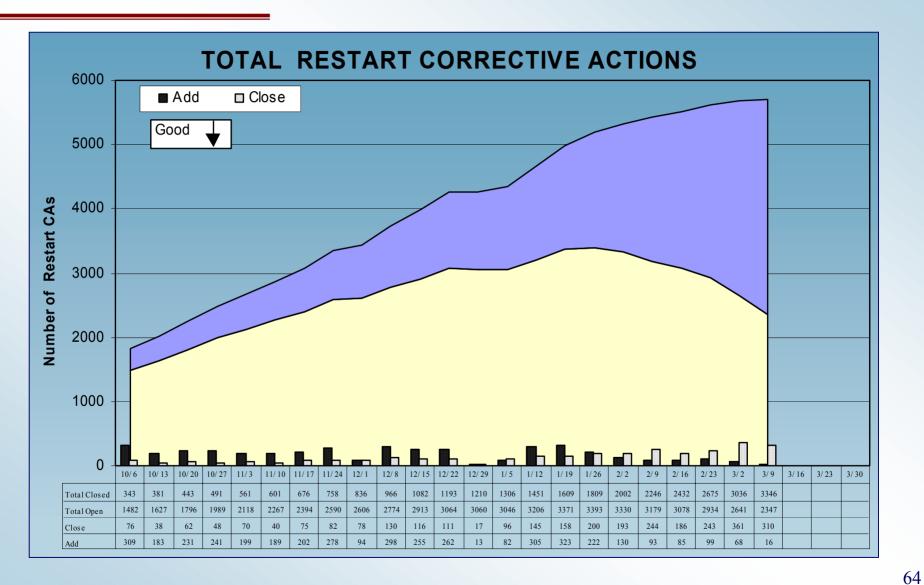




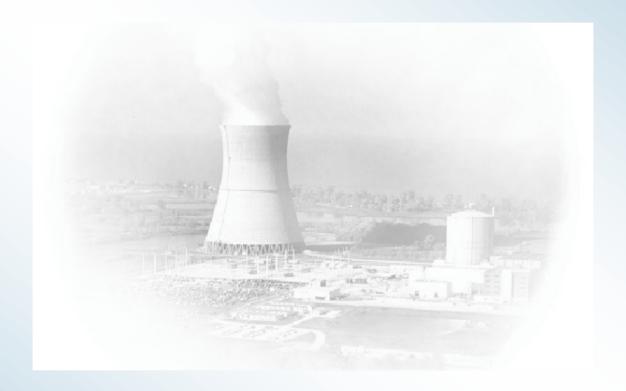






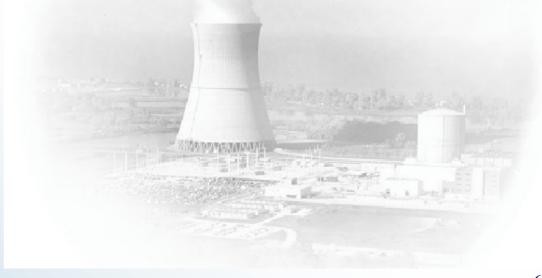






Jim Powers
Director - Nuclear Engineering

- Program Review Process Approval
- New Set Point Program Prior to Restart
- Programs Changes and Improvements
 - Involves more than 60 programs

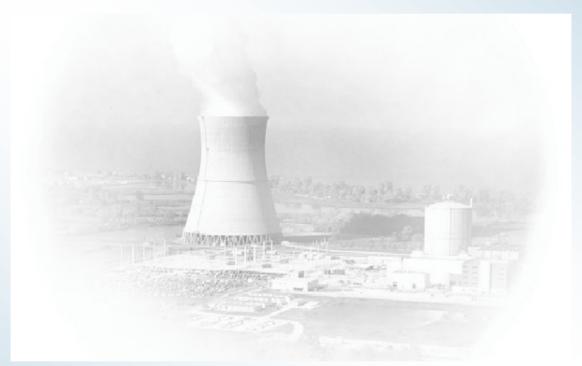


- Corrective Action Program
 - Issue: Inadequate implementation of the program
 - Resolution: Condition Report Analyst,
 Corrective Action Review Board
 improvements, and procedure changes
- Operating Experience (OE)
 - Issue: OE not properly evaluated and incorporated into activities
 - Resolution: Procedure changed to improve process and OE use promoted

- Boric Acid Corrosion Control
 - Issue: Did not address sources of boric acid outside containment and Reactor Vessel Head
 - Resolution: Revised program to include sources outside containment and Reactor Vessel Head
- In-Service Inspection Program
 - Issue: Program did meet the legal requirement,
 but not the intent
 - Resolution: Procedures have been revised and personnel refocused

- Plant Modification Program
 - Issue: Handoff of engineering package activities to planning and field relies on tribal knowledge to function
 - Resolution: Procedures are being developed and revised to address the issues
- Radiation Protection Program
 - Issue: Content of procedures does not contain requirements
 - Resolution: Procedures are being upgraded to incorporate requirements

Upcoming Activities



Greg Dunn

Manager - Outage Management & Work Control

Upcoming Activities

- Restart Milestones
 - Tension Studs/Mode 5
 - Drain Down/Nozzle Dams
 - Reactor Coolant System Fill
 - Integrated Leak Rate Test
 - Emergency Sump Lower Strainer Assembly
 - Containment Air Coolers
 - Mode 4 Restart Readiness Meeting
 - Reactor Pressure Test

Closing Remarks



Lew Myers
Chief Operating Officer - FENOC