

February 22, 2002

Mr. William R. McCollum, Jr.  
Vice President, Oconee Site  
Duke Energy Corporation  
7800 Rochester Highway  
Seneca, SC 29672

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2 AND 3 - RESPONSE TO BULLETIN 2001-01, "CIRCUMFERENTIAL CRACKING OF REACTOR PRESSURE VESSEL HEAD PENETRATION NOZZLES" (TAC NOS. MB2643, MB2644 AND MB2645)

Dear Mr. McCollum:

On August 3, 2001, the staff issued Bulletin 2001-01, "Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles," to the industry requesting that addressees provide information related to the structural integrity of the reactor pressure vessel head penetration (VHP) nozzles for their respective facilities, including the extent of VHP nozzle leakage and cracking that has been found to date, the inspections and repairs that have been undertaken to satisfy applicable regulatory requirements, and the basis for concluding that their plans for future inspections will ensure compliance with applicable regulatory requirements at their respective pressurized water reactor plants. You were requested to respond to Items 1 and 2 of the Bulletin within 30 days of its issuance.

You provided your Bulletin response by letter dated August 28, 2001, indicating that your plants are classified as having previously identified primary water stress corrosion cracking (PWSCC) in the VHP nozzles at your plants. The staff finds that you have provided the requested information. Your Bulletin response stated that you plan to perform a qualified visual examination of 100 percent of the VHP nozzles at your next refueling outage, consistent with the description for such an examination provided in Bulletin 2001-01. Although the proposed inspection method and schedule described in your Bulletin response are not consistent with the discussion in the Bulletin for plants which have identified PWSCC in their VHP nozzles, the staff has concluded that the most recent qualified visual examination of 100 percent of the VHP nozzles performed at your plants, in conjunction with your inspection plans, provides reasonable assurance that the public health and safety will be maintained through the next inspection at your plants. Since the proposed inspection scope and schedule described in your response were integral to the staff's finding, it is the staff's expectation that you will submit a revised response to the Bulletin if you make any substantive changes to the schedule and/or scope of future inspections for your plants. If warranted by such changes, the staff will reevaluate this issue for Oconee Nuclear Station, Units 1, 2 and 3.

Addressees are reminded that Item 5 of the Bulletin requested the following information within 30 days after plants restart following the next refueling outage:

- a. a description of the extent of VHP nozzle leakage and cracking detected at your plants, including the number, location, size, and nature of each crack detected;

Mr. W. R. McCollum

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- b. if cracking is identified, a description of the inspections (type, scope, qualification requirements, and acceptance criteria), repairs, and other corrective actions you have taken to satisfy applicable regulatory requirements. This information is requested only if there are any changes from prior information submitted in accordance with this bulletin.

Sincerely,

*/RA/*

Leonard N. Olshan, Senior Project Manager, Section 1  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, and 50-287

cc: See next page

- b. if cracking is identified, a description of the inspections (type, scope, qualification requirements, and acceptance criteria), repairs, and other corrective actions you have taken to satisfy applicable regulatory requirements. This information is requested only if there are any changes from prior information submitted in accordance with this bulletin.

Sincerely,

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Leonard N. Olshan, Senior Project Manager, Section 1  
 Project Directorate II  
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 Office of Nuclear Reactor Regulation

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