

**Regulatory Guides Not Planned To Be Updated by December 2010**

RG	Title
1.109	Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I
1.112	Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light-Water-Cooled Power Reactors
1.121	Bases for Plugging Degraded PWR Steam Generator Tubes
1.132	Site Investigations for Foundations of Nuclear Power Plants
1.138	Laboratory Investigations of Soils and Rocks for Engineering Analysis and Design of Nuclear Power Plants
1.14	Reactor Coolant Pump Flywheel Integrity
1.140	Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Normal Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants
1.142	Safety-Related Concrete Structures for Nuclear Power Plants (Other than Reactor Vessels and Containments)
1.152	Criteria for Digital Computers in Safety Systems of Nuclear Power Plants
1.161	Evaluation of Reactor Pressure Vessels with Charpy Upper-Shelf Energy Less Than 50 Ft-Lb
1.166	Pre-Earthquake Planning and Immediate Nuclear Power Plant Operator Postearthquake Actions
1.181	Content of the Updated Final Safety Analysis Report in Accordance with 10 CFR 50.71(e)
1.184	Decommissioning of Nuclear Power Reactors
1.185	Standard Format and Content for Post-Shutdown Decommissioning Activities Report
1.188	Standard Format and Content for Applications To Renew Nuclear Power Plant Operating Licenses
1.190	Calculational and Dosimetry Methods for Determining Pressure Vessel Neutron Fluence
1.191	Fire Protection Program for Nuclear Power Plants During Decommissioning and Permanent Shutdown
1.192	Operation and Maintenance Code Case Acceptability, ASME OM Code
1.195	Methods and Assumptions for Evaluating Radiological Consequences of Design Basis Accidents at Light-Water Nuclear Power Reactors
1.197	Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors
1.198	Procedures and Criteria for Assessing Seismic Soil Liquefaction at Nuclear Power Plant Sites
1.199	Anchoring Components and Structural Supports in Concrete
1.201	Guidelines for Categorizing Structures, Systems, and Components in Nuclear Power Plants According to Their Safety Significance
1.202	Standard Format and Content of Decommissioning Cost Estimates for Nuclear Power Reactors

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<b>RG</b>	<b>Title</b>
1.203	Transient and Accident Analysis Methods
1.204	Guidelines for Lightning Protection of Nuclear Power Plants
1.24	Assumptions Used for Evaluating the Potential Radiological Consequences of a Pressurized Water Reactor Radioactive Gas Storage Tank Failure
1.25	Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors
1.26	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants
1.29	Seismic Design Classification
1.3	Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss of Coolant Accident for Boiling Water Reactors
1.31	Control of Ferrite Content in Stainless Steel Weld Metal
1.32	Criteria for Power Systems for Nuclear Power Plants
1.35	Inservice Inspection of UngROUTed Tendons in Prestressed Concrete Containments
1.35.1	Determining Prestressing Forces for Inspection of Prestressed Concrete Containments
1.36	Nonmetallic Thermal Insulation for Austenitic Stainless Steel
1.41	Preoperational Testing of Redundant On-Site Electric Power Systems To Verify Proper Load Group Assignments
1.5	Assumptions Used for Evaluating the Potential Radiological Consequences of a Steam Line Break Accident for Boiling Water Reactors
1.52	Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Post-Accident Engineered-Safety-Feature Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants
1.53	Application of the Single-Failure Criterion to Nuclear Power Plant Protection Systems
1.60	Design Response Spectra for Seismic Design of Nuclear Power Plants
1.70	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (LWR Edition)
1.75	Physical Independence of Electric Systems
1.86	Termination of Operating Licenses for Nuclear Reactors
1.87	Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors
3.10	Liquid Waste Treatment System Design Guide for Plutonium Processing and Fuel Fabrication Plants
3.18	Confinement Barriers and Systems for Fuel Reprocessing Plants
3.19	Reporting of Operating Information for Fuel Reprocessing Plants
3.20	Process Offgas Systems for Fuel Reprocessing Plants

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<b>RG</b>	<b>Title</b>
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3.22	Periodic Testing of Fuel Reprocessing Plant Protection System Actuation Functions
3.26	Standard Format and Content of Safety Analysis Reports for Fuel Reprocessing Plants
3.27	Nondestructive Examination of Welds in the Liners of Concrete Barriers in Fuel Reprocessing Plants
3.28	Welder Qualification for Welding in Areas of Limited Accessibility in Fuel Reprocessing Plants and in Plutonium Processing and Fuel Fabrication Plants
3.29	Preheat and Interpass Temperature Control for the Welding of Low-Alloy Steel for Use in Fuel Reprocessing Plants and in Plutonium Processing and Fuel Fabrication Plants
3.3	Quality Assurance Program Requirements for Fuel Reprocessing Plants and for Plutonium Processing and Fuel Fabrication Plants
3.30	Selection, Application, and Inspection of Protective Coatings (Paints) for Fuel Reprocessing Plants
3.32	General Design Guide for Ventilation Systems for Fuel Reprocessing Plants
3.37	Guidance for Avoiding Intergranular Corrosion and Stress Corrosion in Austenitic Stainless Steel Components of Fuel Reprocessing Plants
3.40	Design Basis Floods for Fuel Reprocessing Plants and for Plutonium Processing and Fuel Fabrication Plants
3.54	Spent Fuel Heat Generation in an Independent Spent Fuel Storage Installation
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3.69	Topical Guidelines for the Licensing Support Network
3.7	Monitoring of Combustible Gases and Vapors in Plutonium Processing and Fuel Fabrication Plants
3.72	Guidance for Implementation of 10 CFR 72.48, Changes, Tests, and Experiments
3.73	Site Evaluations and Design Earthquake Ground Motion for Dry Cask Independent Spent Fuel Storage and Monitored Retrievable Storage Installations
4.17	Standard Format and Content of Site Characterization Plans for High-Level-Waste Geologic Repositories
4.18	Standard Format and Content of Environmental Reports for Near-Surface Disposal of Radioactive Waste
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5.11	Nondestructive Assay of Special Nuclear Material Contained in Scrap and Waste
5.15	Tamper-Indicating Seals for the Protection and Control of Special Nuclear Material
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5.22	Assessment of the Assumption of Normality (Employing Individual Observed Values)
5.23	In Situ Assay of Plutonium Residual Holdup
5.25	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material in Equipment for Wet Process Operations
5.26	Selection of Material Balance Areas and Item Control Areas
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5.31	Specially Designed Vehicle with Armed Guards for Road Shipment of Special Nuclear Material
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5.33	Statistical Evaluation of Material Unaccounted For
5.34	Nondestructive Assay for Plutonium in Scrap Material by Spontaneous Fission Detection
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5.38	Nondestructive Assay of High-Enrichment Uranium Fuel Plates by Gamma Ray Spectrometry
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5.48	Design Considerations - Systems for Measuring the Mass of Liquids
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5.5	Standard Methods for Chemical, Mass Spectrometric, and Spectrochemical Analysis of Nuclear-Grade Uranium Dioxide Powders and Pellets
5.52	Standard Format and Content of a Licensee Physical Protection Plan for Strategic Special Nuclear Material at Fixed Sites (Other than Nuclear Power Plants)
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5.59	Standard Format and Content for a Licensee Physical Security Plan for the Protection of Special Nuclear Material of Moderate to Low Strategic Significance
5.60	Standard Format and Content of a Licensee Physical Protection Plan for Strategic Special Nuclear Material in Transit
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5.8	Design Considerations for Minimizing Residual Holdup of Special Nuclear Material in Drying and Fluidized Bed Operations
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7.10	Establishing Quality Assurance Programs for Packaging Used in Transport of Radioactive Material
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8.30	Health Physics Surveys in Uranium Recovery Facilities
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