

# **Safety Evaluation Report**

Related to the License Renewal of the Brunswick Steam Electric Plant, Units 1 and 2

Docket Nos. 50-325 and 50-324

Carolina Power & Light Company

U.S. Nuclear Regulatory Commission Office of Nuclear Reactor Regulation Washington, DC 20555-0001



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Division of License Renewal Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, DC 20555-0001



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### **ABSTRACT**

This safety evaluation report (SER) documents the technical review of the Brunswick Steam Electric Plant (BSEP), Units 1 and 2, license renewal application (LRA) by the staff of the U.S. Nuclear Regulatory Commission (NRC) (the staff). By letter dated October 18, 2004, Carolina Power & Light Company (CP&L or the applicant) submitted the LRA for BSEP in accordance with Title 10, Part 54, of the Code of Federal Regulations (10 CFR Part 54). CP&L is requesting renewal of the operating licenses for BSEP Units 1 and 2, (Facility Operating License Numbers DPR-71 and DPR-62, respectively) for a period of 20 years beyond the current expiration dates of midnight September 8, 2016, for Unit 1 and midnight December 27, 2014, for Unit 2.

The BSEP units are located south of Wilmington, NC, at the mouth of the Cape Fear River in Brunswick County, NC, and two miles north of Southport, NC. The NRC issued the construction permits for Units 1 and 2 on February 7, 1970. The NRC issued the operating licenses for Unit 1 on November 12, 1976; and for Unit 2 on December 27, 1974. Units 1 and 2 are boiling water reactors (BWRs) with primary containments of the BWR Mark I design. Each unit has a nuclear steam supply system that is supplied by General Electric (GE) Nuclear Energy Company. The balance of the plant was originally designed and constructed by Brown & Root with the assistance of its agent, United Engineers & Constructors. Each unit operates at a licensed power output of 2923 megawatt thermal (Mwt), with a gross electrical output of approximately 1007 megawatt electric (Mwe).

This SER presents the status of the staff's review of information submitted to the NRC through December 6, 2005, the cutoff date for consideration in the SER. The staff will present its final conclusion on the review of the BSEP application in its update to this SER.

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### **ABBREVIATIONS**

AC alternating current

ACI American Concrete Institute

ACRS Advisory Committee on Reactor Safeguards

ACSR aluminum conductor steel reinforced
ADS automatic depressurization system
AERM aging effects requiring management

AFFF aqueous fire fighting foam

AFW auxiliary feedwater

AISC American Institute of Steel Construction

AISI American Iron and Steel Institute
AMP aging management program
AMR aging management review

AMSAC ATWS mitigating system actuation circuitry
ANSI American National Standards Institute
AOG augmented off-gas/auxiliary off-gas
AOO anticipated operational occurrence
API American Petroleum Institute
APRM average power range monitor

ARI alternate rod injection/alternate rod insertion

ARM area radiation monitor

ART adjusted reference temperature

AS&CR auxiliary steam and condensate recovery
ASCE American Society of Civil Engineers
ASME American Society of Mechanical Engineers

AST accident source term

ASTM American Society for Testing and Materials

ATWS anticipated transient without scram

ATWS-RPT anticipated transient without scram-recirculation pump trip

AWS American Welding Society

AWWA American Water Works Association

B&PV boiler and pressure vessel BNP Brunswick Nuclear Plant

BSEP Brunswick Steam Electric Plant

BTP branch technical position

BTRS boron thermal regeneration system

BWR boiling water reactor

BWROG Boiling Water Reactor Owners Group

BWRVIP Boiling Water Reactor Vessel and Internals Program

CAC containment atmospheric control CAD containment atmosphere dilution CAP Corrective Action Program

CASS cast austentitic steel

CB control board

CCW closed cooling water or component cooling water

CDD condensate deep bed demineralizer

CDF core damage frequency
CET core exit thermocouple
CF chemistry factor

CFD condensate filter demineralizer
CFR Code of Federal Regulations

CHRS containment heat removal system

CI confirmatory item CL chlorination

CLB current licensing basis

CMAA Crane Manufacturers Association of America

CP&L Carolina Power & Light Company, a Progress Energy Company

CR condition report CRD control rod drive

CRDH control rod drive housing CRDM control rod drive mechanism

CRGT control rod guide tube CRW clean radioactive waste

CS containment spray or carbon steel

CST condensate storage tank
CUF cumulative usage factor

CVCS chemical and volume control system

CW circulating water

DBA design-basis accident
DBE design-basis event
DC direct current
DG diesel generator

DGB diesel generator building

DOR Division of Operating Reactors (NRC)

D/P differential pressure
DRW dirty radioactive waste

DSCSS drywell and suppression chamber spray system

DW demineralized water
DWT demineralized water tank

ECCS emergency core cooling system

EDB equipment database

EDG emergency diesel generator EFPY effective full-power year

EOL end of life

EPRI Electric Power Research Institute

EQ environmental qualification ESF engineered safety feature

FAC flow-accelerated corrosion environmental fatigue factor

FERC Federal Energy Regulatory Commission

FHA fire hazards analysis

FO fuel oil

FOL facility operating license FOST fuel oil storage tank FP fire protection

FPP fire protection plan

FSAR Final Safety Analysis Report FSD functional system description FSER Final Safety Evaluation Report

FW feedwater

GALL generic aging lessons learned

GDC general design criteria or general design criterion

GE General Electric

GEIS generic environmental impact statement

GL generic letter

GSI general safety issue

HAZ heat-affected zone HCU hydraulic control unit

HD heater drains

HDFSS high density fuel storage system

HELB high-energy line break

HE/ME high energy/moderate energy
HEPA high efficiency particulate air
HJTC heated junction thermocouple
HMWPE high molecular weight polyethylene

HP high pressure

HPCI high pressure coolant injection

HPCS high pressure core spray (not an applicable system for BSEP)

HVAC heating, ventilation, and air conditioning

HWC hydrogen water chemistry

HX heat exchanger

IA instrument air

IAN non-interruptible instrument air

IASCC irradiation assisted stress corrosion cracking

I&C instrumentation and control

ID inside diameter

IE inspection and enforcement (former NRC Office of Inspection and Enforcement)

IEEE Institute of Electrical and Electronics Engineers

IGA intergranular attack

IGSCC intergranular stress corrosion cracking

ILRT integrated leak rate test (containment type A test)

IN information notice

INPO Institute of Nuclear Power Operations

IPA integrated plant assessment

IPCEA Insulated Power Cable Engineers Association

IR insulation resistance

IRM intermediate range monitor ISG interim staff guidance ISI inservice inspection

KV kilovolt

LBB leak before break LER Licensee Event Report

LO lubricating oil

LOCA loss-of-coolant accident LOOP loss of offsite power

LP low pressure

LPCI low pressure coolant injection LPCS low pressure core spray LPRM local power range monitor

LR license renewal

LRA license renewal application

—1 intended function (pressure boundary)

-2 intended function (filtration)

—3 intended function (flow restriction)

intended function (structural support/seismic integrity)

—5 intended function (heat transfer)

MCB main control board

MEAP material, environment, aging program

MeV million electron volts

MIC microbiologically induced corrosion

MOD motor operated disconnect

MS main steam

MSIV/LCS main steam isolation valve/leakage control system

MSL main steam line or mean sea level

MSLB main steam line break
MSR moisture separator reheater
MVD miscellaneous vents and drains

Mwe megawatt electric Mwt megawatt thermal

MWTS makeup water treatment system

NDE nondestructive examination
NDTT nil-ductility transition temperature

NEI Nuclear Energy Institute

NEPA National Environmental Policy Act of 1969

NFPA National Fire Protection Association

Ni nickel

NMS neutron monitoring system
NPAR nuclear plant aging research

NPS nominal pipe size

NRC U.S. Nuclear Regulatory Commission

NSR non-safety-related

NSSS nuclear steam supply system

NUREG designation of publications prepared by the NRC staff

OBE operating-basis earthquake

ODSCC outside-diameter stress-corrosion cracking

OE operating experience

OI open item

OLTP original licensed thermal power OPRM oscillation power range monitor

PASS post-accident sampling system PBDS period based detection system

PCB power circuit breaker

PCS primary containment structure
PEC Progress Energy Carolinas
PFM probabilistic fracture mechanics
pH concentration of hydrogen ion
P&ID piping and instrumentation diagram

PM preventive maintenance
PNS pneumatic nitrogen system
PORV power-operated relief valve
PRF penetration room filtration
PRM process radiation monitoring

PSRF non-safety-related that can prevent a safety-related function

P-T pressure-temperature

PTLR pressure-temperature limits report

PTS pressurized thermal shock

PVC polyvinyl chloride

PW pipe whip

PWS potable water system

PWSCC primary water stress-corrosion cracking

QA quality assurance

RAI request for additional information

RB reactor building

RBCCW reactor building closed cooling water

RBM rod block monitor

RCIC reactor core isolation cooling

RCP reactor coolant pump

RCPB reactor coolant pressure boundary

RCS reactor coolant system RFP reactor feedwater pump

RG regulatory guide RHR residual heat removal

Ri risk informed

RI-ISI risk-informed inservice inspection reactor manual control system radiation monitoring system

RMWST reactor makeup water storage tank

RNA reactor non-interruptible air
RPIS rod position information system
RPS reactor protection system
RPV reactor pressure vessel

RT<sub>NDT</sub> reference temperature nil-ductility transition

RT<sub>NDT/UI</sub> reference temperature nil-ductility transition (unirradiated)

RT<sub>PTS</sub> reference temperature pressurized thermal shock

RTS reactor trip system
RVI reactor vessel internals

RVLIS reactor vessel instrumentation system

RWCU reactor water cleanup system

RWM rod worth minimizer

RWST refueling water storage tank reactor building sampling system

SA service air

SAT startup auxiliary transformer

SBO station blackout

SC structure and component or suppression chamber

SCC stress-corrosion cracking SCW screen wash water SDV scram discharge volume

SE safety evaluation

SER Safety Evaluation Report

SFP spent fuel pool

SGBD steam generator blowdown
SGTS standby gas treatment system

SI safety injection SJAE steam jet air ejector SLC standby liquid

SLMS stator leak monitoring system
SMP structural monitoring program
SOC statement of consideration
SPDS safety parameter display system

SR safety-related

SRP Standard Review Plan

SRP-LR Standard Review Plan for Review of License Renewal Applications for Nuclear

**Power Plants** 

SRV safety relief valve SS stainless steel

SSC system, structure, and component

SSE safe-shutdown earthquake

SW service water

SWIS service water intake structure

TAC technical assignment control (internal NRC work management tool)

TASCS thermal stratification, cycling, and striping

TB turbine building

TBCCW turbine building closed cooling water trans-granular stress corrosion cracking

TID total integrated does
TIP traversing incore probe
TLAA time-limited aging analysis
TPNS total plant numbering system

TS technical specification
TSC technical support center
TSP trisodium phosphate
TT thermal transients

UAT unit auxiliary transformer

UFSAR updated final safety analysis report
USAS United States of America Standards

USE upper-shelf energy

UUSE unirradiated upper shelf energy

UT ultrasonic test

VAC Volts alternating current VDC Volts direct current

VFLD vessel flange leak detection

WANO World Association of Nuclear Operators

WCAP Westinghouse Commercial Atomic Power (report)

WOG Westinghouse Owners Group

XLPE cross-linked polyethylene