

Lawrence Livermore National Laboratory

**New LLNL evaluations of ^{237}U , ^{240}Am
and structural materials**



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New tools developed for creating evaluations

geft

- Set of python widgets to convert output of TALYS to ENDL format
- Can create ENDL files for
 - Channel cross sections and particle spectra
 - Inelastic and binary cross sections and angular distributions
 - Gamma multiplicities, continuum and discrete spectra

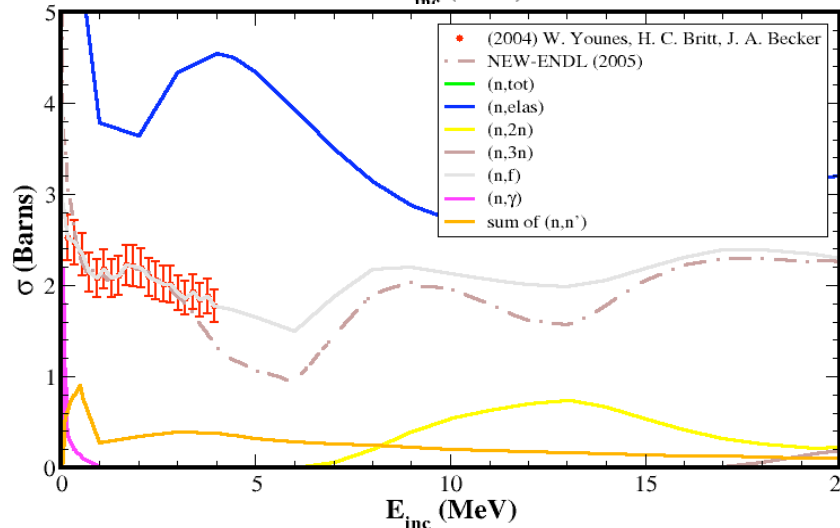
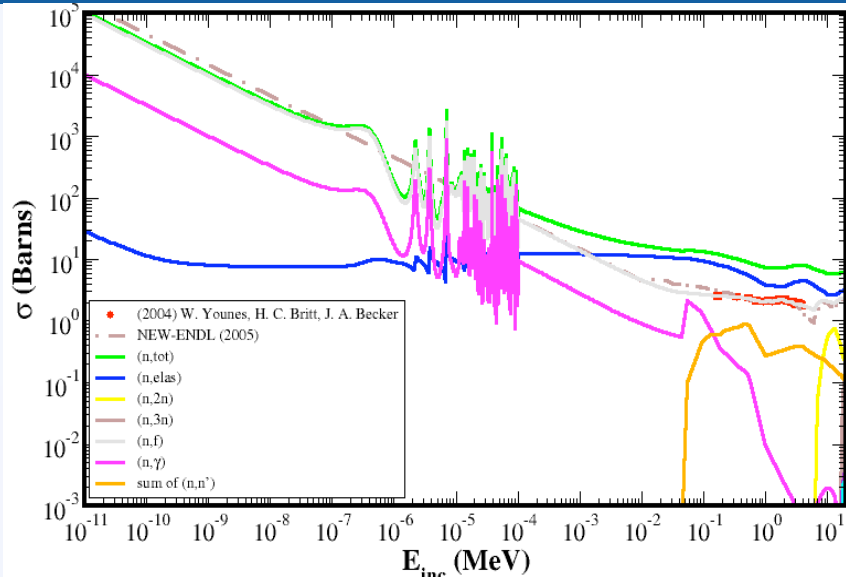
endl2endf

- Can read in an ENDF evaluation and chop it into channels
 - Allows copying untranslated ENDF sections between evaluations
 - ENDL has no equivalent for several reaction types (e.g. resonance region)
- Can reformat any ENDL data type into ENDF

Using combination of these tools, can create evaluations using TALYS and produce data in both ENDL and ENDF formats



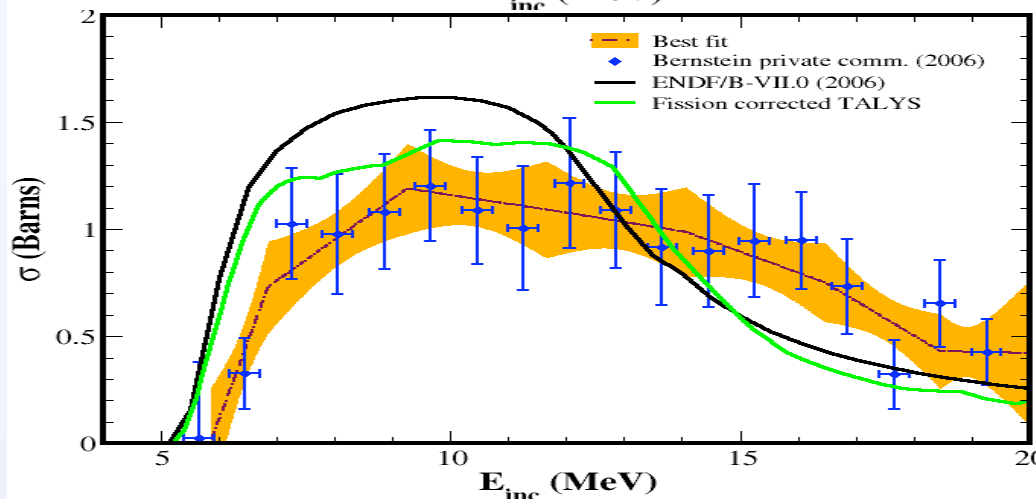
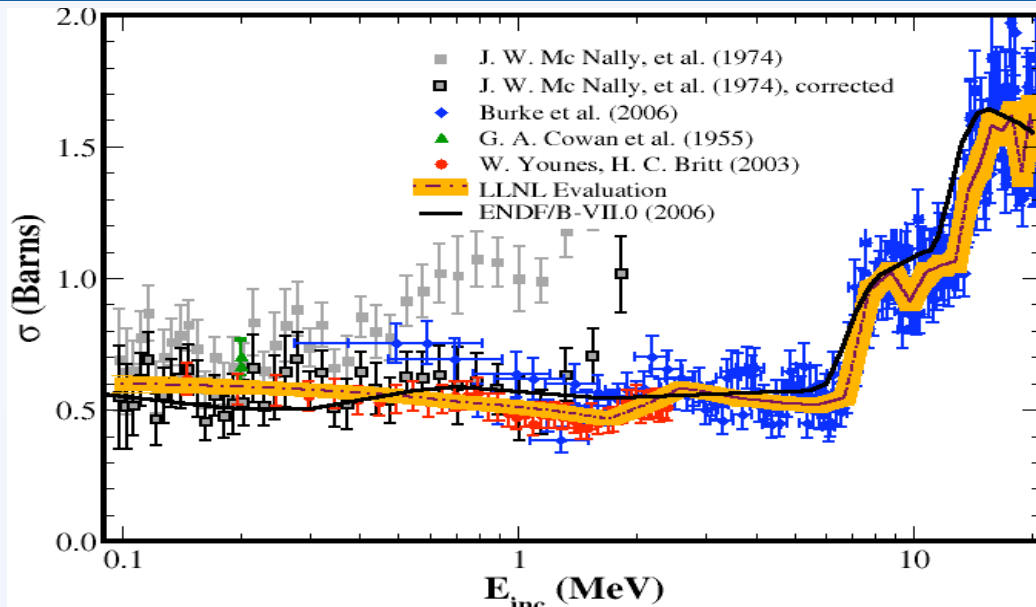
^{240}Am evaluation



- Used TALYS + `geft.py` + `endl2endf.py`
 - Soukhovitskii, Chiba et al. OMP
 - real coupled channel calc.
 - RIPL levels, masses, etc.
- Resonance data, ν and fission spectrum from ^{242}Am evaluation in ENDF/B-VII.0
- *Everything else* from TALYS:
 - σ 's
 - spectra
 - γ 's
 - angular distributions
- We tuned cross-sections:
 - swap in Younes, Britt (n,f) evaluation based on surrogate (t,pf)
 - match σ 's onto resonances



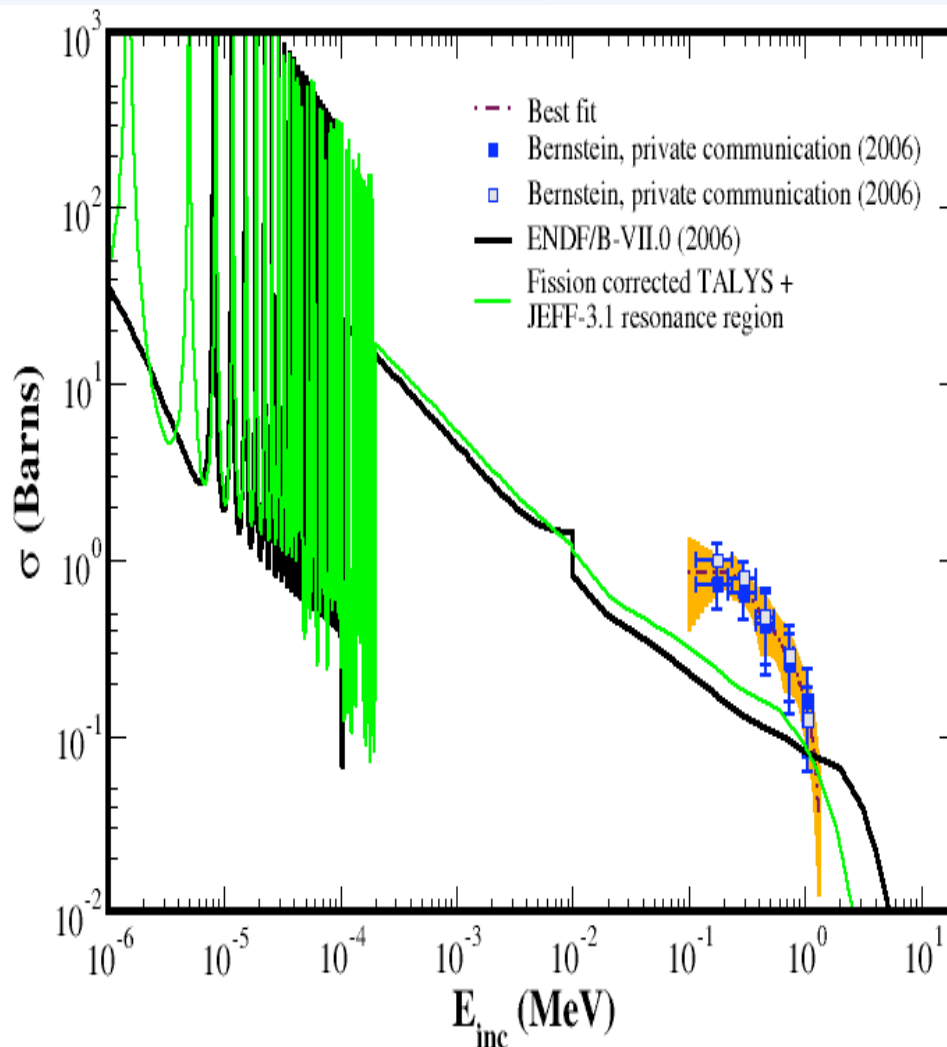
New ^{237}U evaluation based on surrogate data shows important differences with previous estimates



- Hauser-Feshbach modeling w/ TALYS
- Can't model fission, need other scheme:
 - Fit to $\sigma_{(n,f)}$
 - Scale other TALYS σ 's so consistent with $\sigma_{(n,f)}$ and non-elastic σ
- Outgoing particle distributions from TALYS
- Resonance data, ν , and fission spectrum, γ data from JEFF-3.1
- Matches LLNL surrogate (n,f), (n,2n)



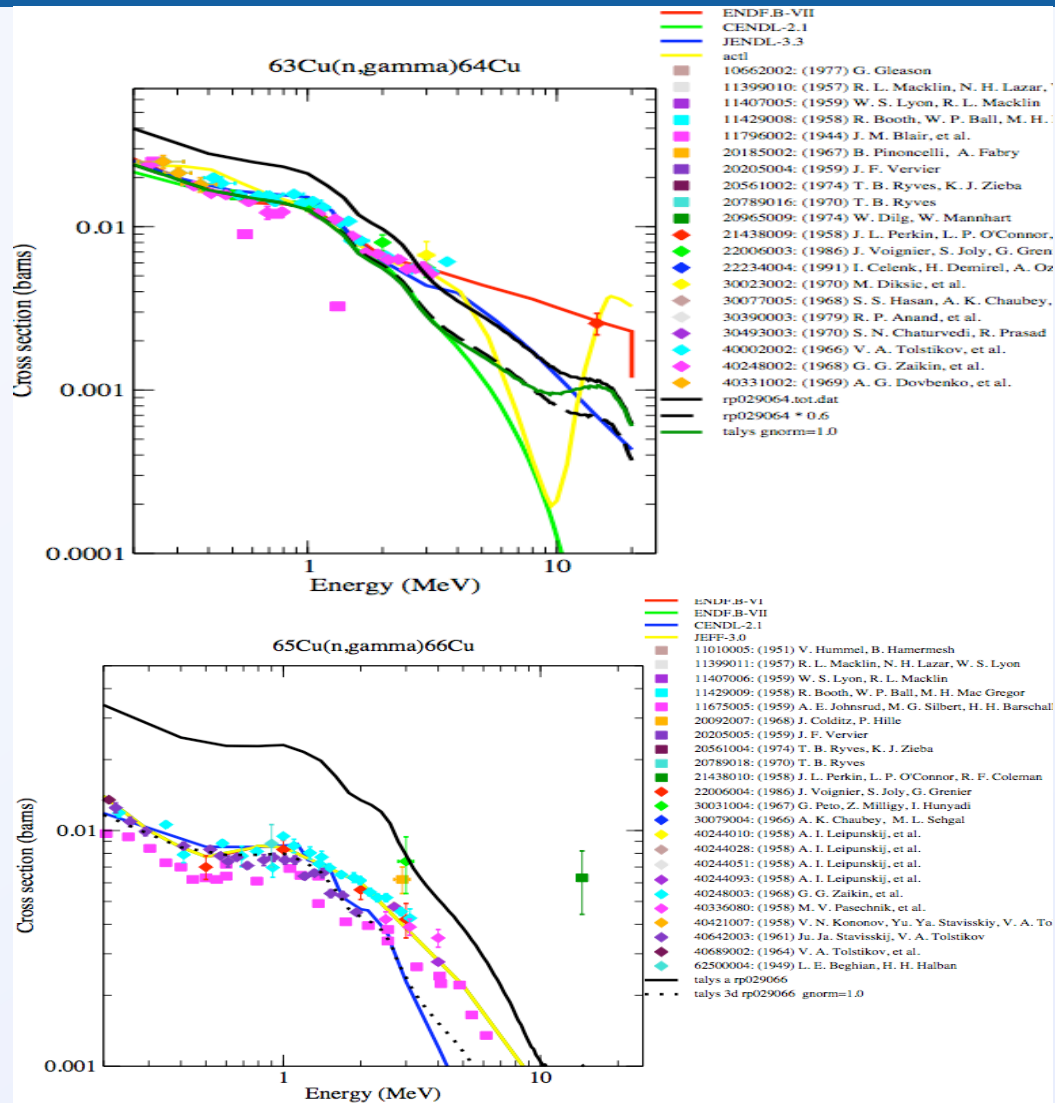
We have not attempted to match surrogate (n, γ) data



- Preliminary data of Bernstein *et al.* (2007)
- Could match by increasing
 - γ -ray strength function
 - absorption cross-section
- Important experimental questions yet to be resolved



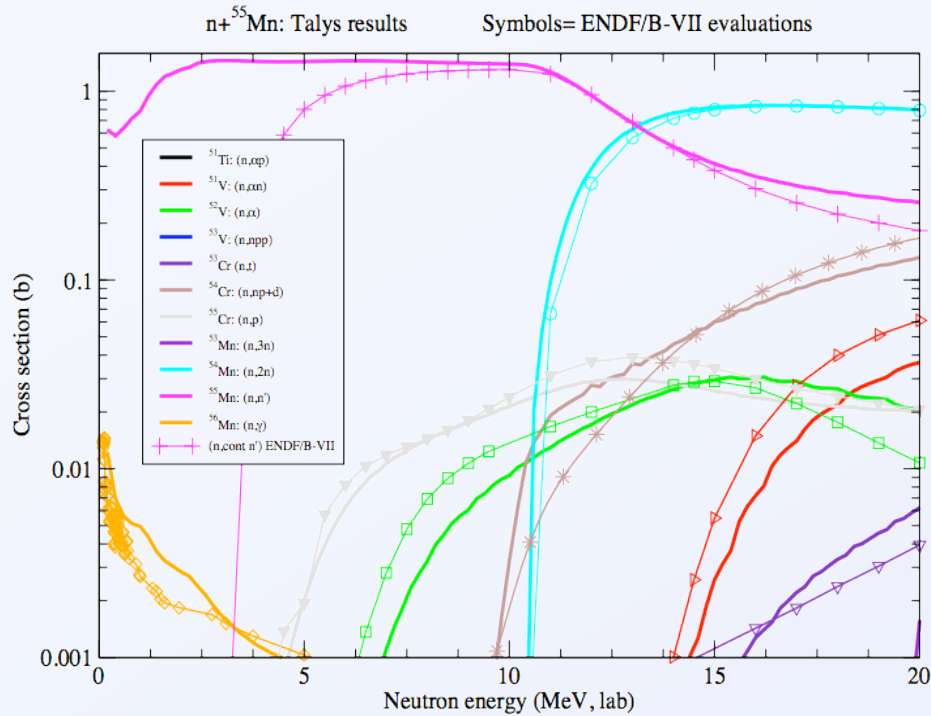
Structural Materials - $^{62-66}\text{Cu}$



- TALYS calculations for Cu isotopes overall fit data well
- Data only for odd isotopes: need to set gamma strength function (G_{norm}) to 1 to fit data (default behavior of scaling to fit GDR produced poor fits)
- Need method to produce resonance data

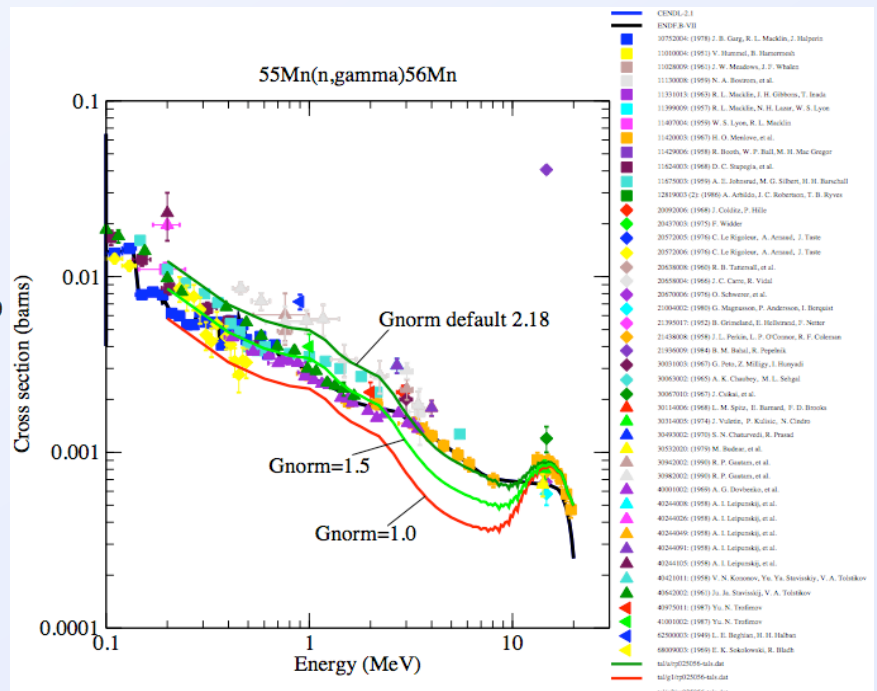


Structural Materials - $^{54,56,57}\text{Mn}$



- ENDF/B-VII satisfactory everywhere
 - except perhaps GDR in (n,γ)

- TALYS calculations satisfactory except for (n,γ) (a 'small' cross section)
 - cannot fit $E < 3$, $3 < E < 12$ and $12 < E < 15$ all at the same time by scaling G_{norm}

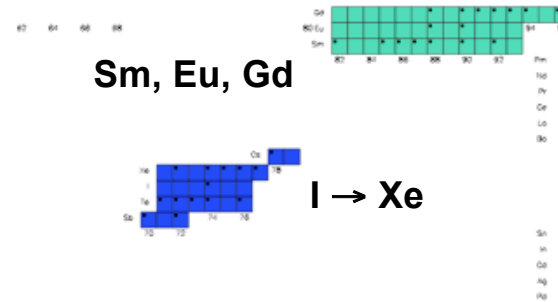
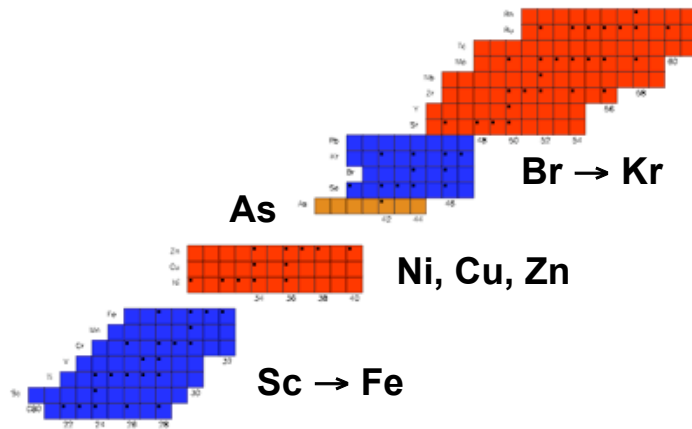


Hoffman radchem activation library

Hoffman & Kelley Radchem Evaluations – LLNL



- Radchem library from 2004-2006 has activation cross sections for 497 reactions
 - n,p,d incident on a range of targets from Sc to Gd



- Using `geft/endl2endf` can fill out evaluations to include particle spectra, elastic and inelastic cross sections and angular distributions
- TALYS used to calculate needed cross sections, renormalized to radchem library calculations from STAPRE
- Filled-out evaluations can then be used in transport codes (MCAPM)

