



# **Impact of ENDF/B-VII.0 for AECL**

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# AECL ENDF/B-VII.0 implementation status

- ***Kudos* to: Ramon Arcilla & NNDC, LANL & RSICC for processing & distributing ENDF/B-VII.0 as MCNP-useable ACE-format files**
  - Provides a standard reference library data set & facilitated early testing
- **Early testing of ENDF/B-VII.0 against ZED-2 D<sub>2</sub>O/air-cooled Natural Uranium (NU) critical experiments indicated** (see ND-2007 paper)
  - Systematic increase in MCNP  $k_{eff}$  by ~4.3 mk, improving agreement with measurement (except for ZEEP NU metal lattices); mainly due to <sup>238</sup>U changes
  - Small (0.4 mk), consistent reduction of MCNP D<sub>2</sub>O Coolant Void Reactivity (CVR) calculation bias when Thermal Scattering Law (TSL; S( $\alpha,\beta$ )) data for O&U in UO<sub>2</sub> available with ENDF/B-VII.0 are used
- **Additional comparisons against AECL critical measurements exemplify the benefits of ENDF/B-VII.0 & are considered further here** (summary submitted to PHYSOR-2008)
  - ZED-2: H<sub>2</sub>O/air-cooled Slightly Enriched Uranium (SEU; 0.95 wt% <sup>235</sup>U)/ Recovered Uranium (RU; 0.96 wt% <sup>235</sup>U)
  - MAPLE (Multipurpose Applied Physics Lattice Experiment): initial startup commissioning measurements
- **Current AECL ENDF/B-VII.0 implementation status**
  - Recommend early adoption of ENDF/B-VII.0 for all AECL applications: CANDU; ACR (Advanced CANDU Reactor); MAPLE; ZED-2; NRU (50-year old National Research Universal)
  - ***But* still need LEAPR input files or input specifications from LANL for TSL libraries: H in H<sub>2</sub>O; D in D<sub>2</sub>O; O & U in UO<sub>2</sub>; & Al to be able to generate multi-temperature data libraries with NJOY at the temperatures needed for specific applications & for accident analyses**



## Zero Energy Deuterium (ZED-2)

- Large D<sub>2</sub>O-moderated fuel lattice
- Cylindrical aluminum tank with 336 cm ID and 334 cm deep
- Surrounded by graphite reflector, 60 cm radially and 90 cm bottom
- Fuel rods and/or channels are suspended from steel beams at top of calandria at various spacings:
  - 28.6 cm for CANDU
  - 24.0 cm for ACR-1000
- Criticality achieved by adjusting moderator level within  $\pm 0.2$  cm
- Fuel channel consists of calandria tube, pressure tube, fuel assembly and coolant (D<sub>2</sub>O, H<sub>2</sub>O or air).

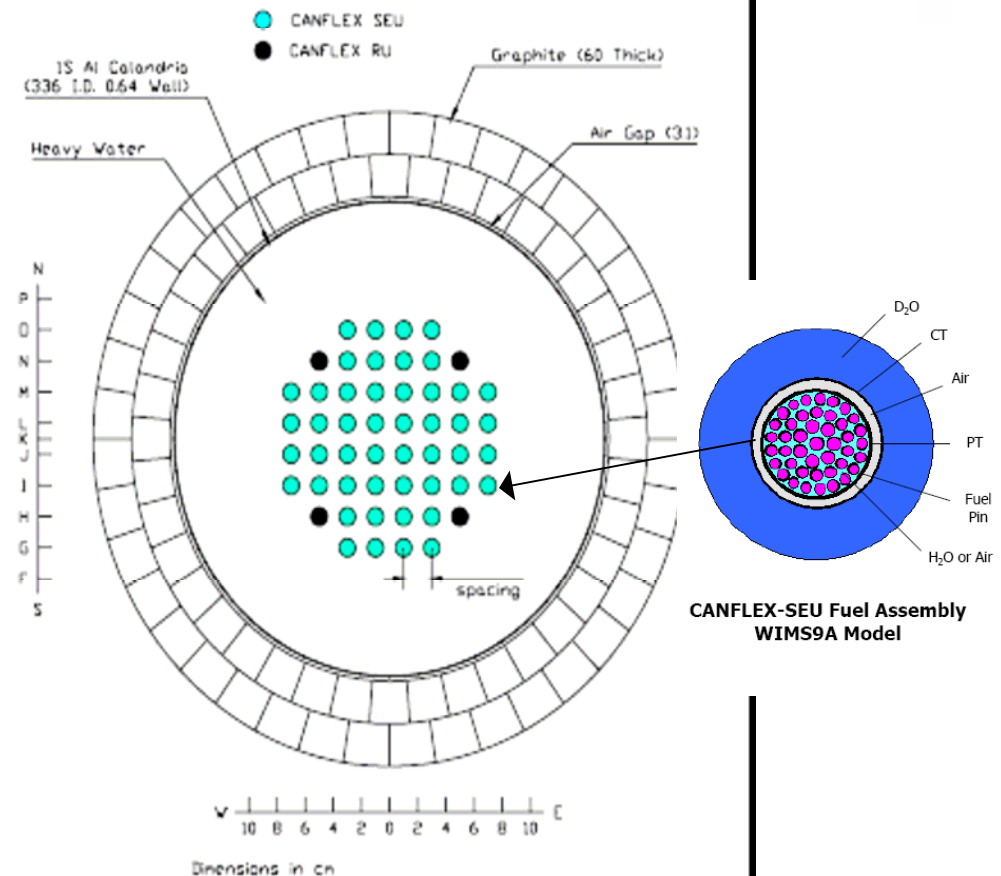


*ZED-2, Chalk River Labs*



# SEU/RU Flux-Map Experiments

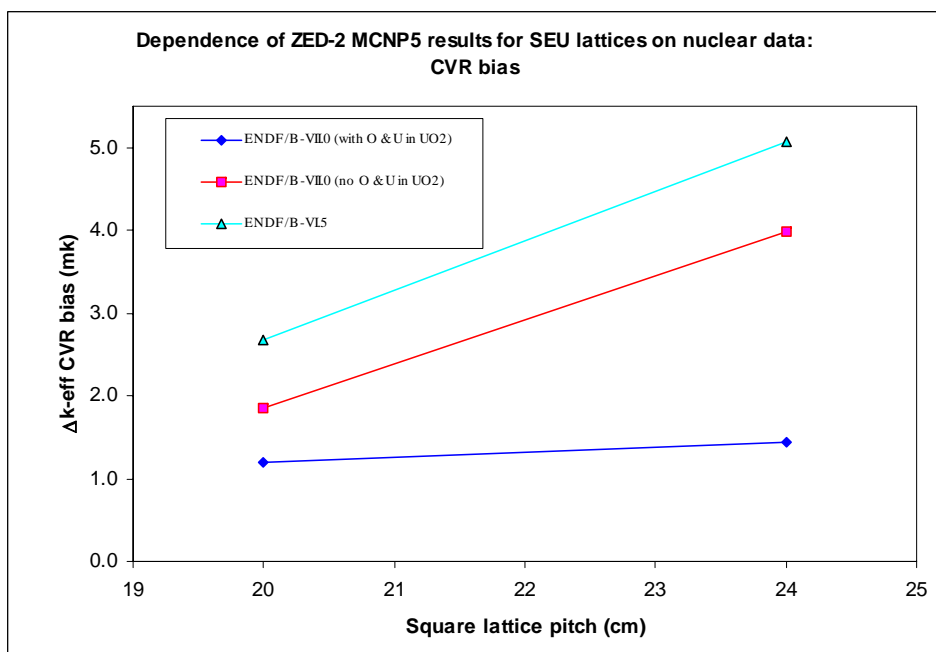
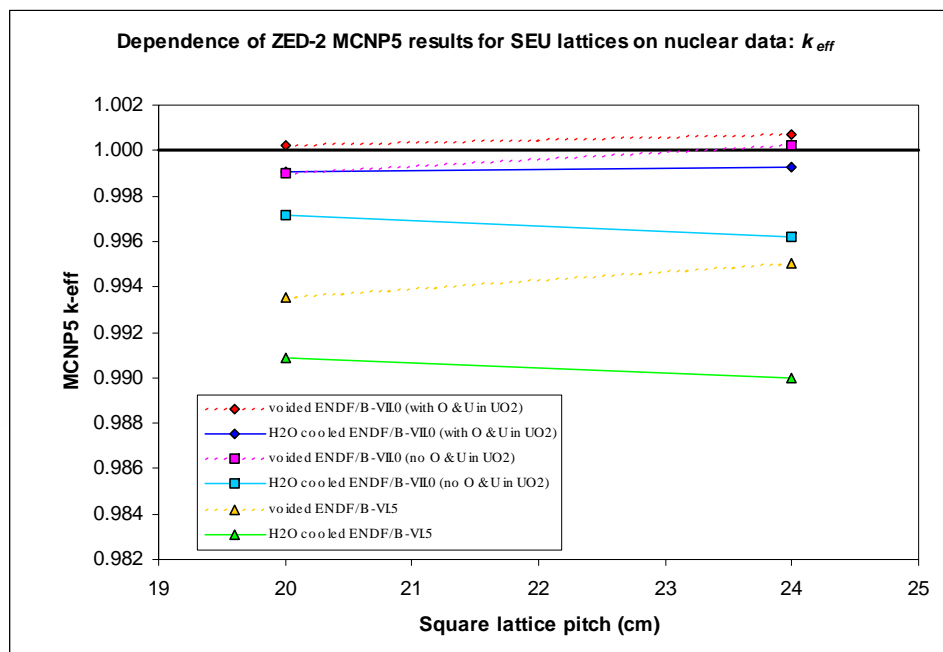
- 52 fuel channels arranged in a square lattice with a spacing of 20 cm or 24 cm.
- Channel consists of five 43-pin fuel assemblies surrounded by CANDU-sized aluminum PT/CT:
  - 48 channels use 0.95% SEU
  - 4 channels use 0.96% RU (recovered uranium)
- H<sub>2</sub>O or air (void) coolant.
- D<sub>2</sub>O moderator height adjusted to achieve criticality.
- Cu and In-Al foil activation measurements performed axially and radially.





# ZED-2 SEU/RU H<sub>2</sub>O/air-cooled critical measurements

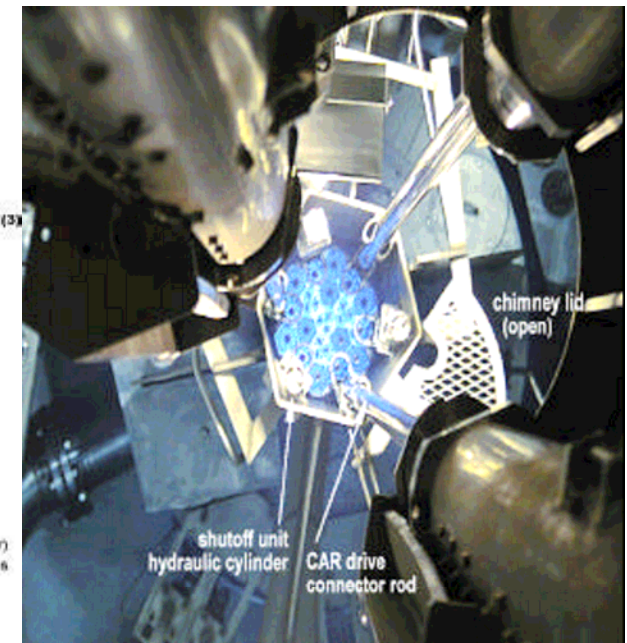
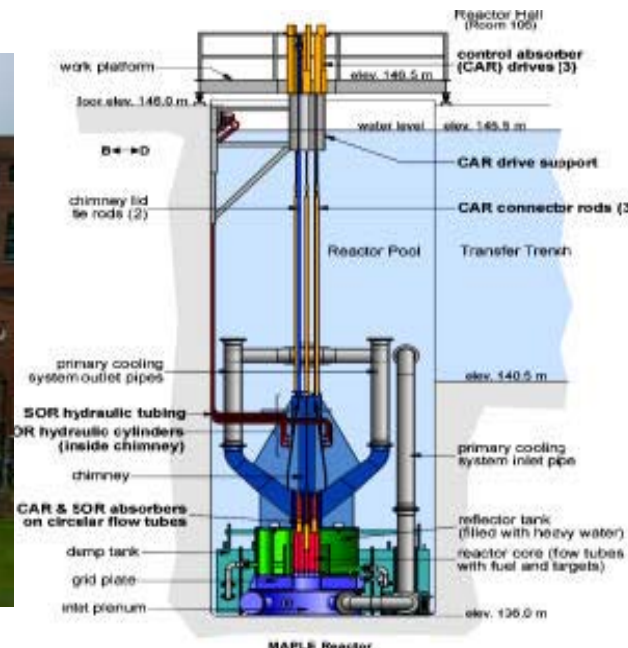
- Dramatic improvement in both  $k_{eff}$  & CVR bias when ENDF/B-VII.0 used with TSL data for O&U in UO<sub>2</sub>





# MAPLE dedicated isotope production reactors

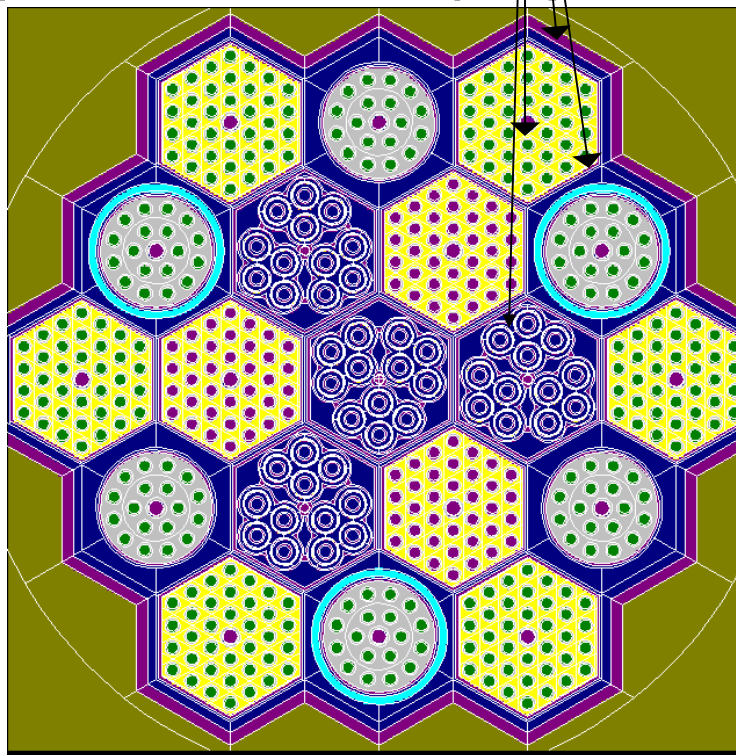
- Two 10-MW<sub>t</sub> MAPLE reactors to replace NRU mainly for fission-product <sup>99</sup>Mo production
- Complex, compact initial start-up core consisting of HEU (93.0 wt% <sup>235</sup>U) UO<sub>2</sub> target fuel assemblies, 36-element & 18-element LEU (19.75 wt% <sup>235</sup>U) driver fuel assemblies & 36-el. DU (Depleted Uranium) assemblies





## MAPLE 30°C isothermal critical results

- MCNP5  $k_{eff}$  within  $\sim 1$  mk using ENDF/B-VII.0
- $\sim 7\%$  of total n captures occur in Zr (zirc-4: core-reflector boundary; hex flow tubes; central support rods; HEU clad & structures)
- Reactivity impacts of major changes to  $^{90}\text{Zr}$  (51.5% abundance) &  $^{91}\text{Zr}$  (11.2% abundance) capture cross sections tend to cancel out

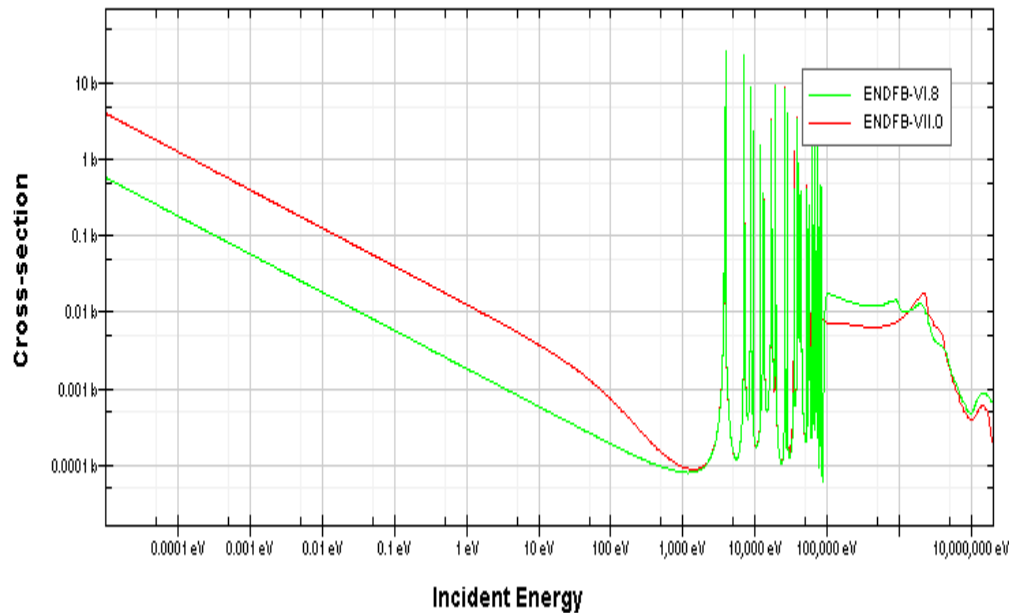




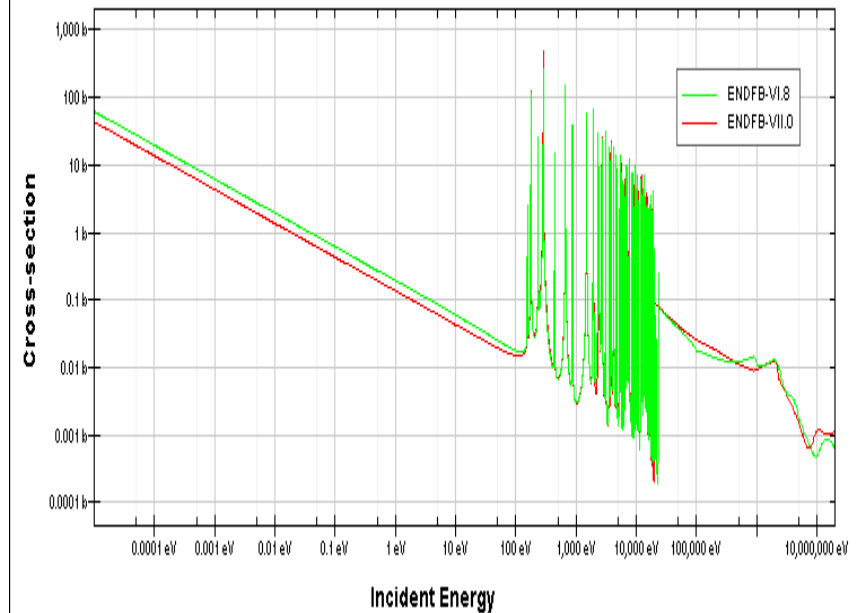
# Changes to Zr thermal captures

- Huge increase in  $^{90}\text{Zr}$  capture cross section, but low net reactivity impact due to opposing change to  $^{91}\text{Zr}$  capture cross section
- Less incentive to develop Zr materials enriched in  $^{90}\text{Zr}$

Incident neutron data // Zr90 / MT=102 : (z,g) radiative capture / Cross section



Incident neutron data // Zr91 / MT=102 : (z,g) radiative capture / Cross section







## MAPLE MCNP5 results for 30°C isothermal core

Case	Particle weight loss to $^{90}\text{Zr}$ captures	Particle weight loss to $^{91}\text{Zr}$ captures	Calculated $k_{eff}$	$\Delta k_{eff}$ (mk)
ENDF/B-VII.0 reference	8.81E-03	2.15E-02	$1.000725 \pm 0.000037$	-
ENDF/B-VI.5	2.06E-03	2.94E-02	$0.997460 \pm 0.000050$	$-3.26 \pm 0.06$
ENDF/B-VII.0 with $^{90}\text{Zr}$ from ENDF/B-VI.5	2.07E-03	2.18E-02	$1.002985 \pm 0.000053$	$2.26 \pm 0.06$
ENDF/B-VII.0 with $^{91}\text{Zr}$ from ENDF/B-VI.5	8.68E-03	2.87E-02	$0.997451 \pm 0.000100$	$-3.27 \pm 0.06$



## Other nuclear data items of potential interest

- 3-way collaboration (IRMM-GELINA, ORNL, AECL) to address new (n,d) energy-angle elastic scattering evaluation
  - AECL to fund additional nuclear theory calculations incorporating: alternate nuclear potential models, magnetic interaction & 3-body forces
- New RPI measurements for Gd, did not make it into ENDF/B-VII.0
  - Qualitative agreement with ZED-2 measurements; may confirm changes
- Some ZED-2 In (Indium) flux-scan measurements appear to show small systematic discrepancies relative to MCNP simulations; need to simulate with ENDF/B-VII.0



 **AECL**