# NUCLEAR REGULATORY COMMISSION

## Sunshine Act; Meetings

**DATES:** Weeks of February 16, 23; March 1, 8, 15, 22, 2004.

**PLACE:** Commissioners' Conference Room, 11555 Rockville Pike, Rockville, Maryland.

STATUS: Public and closed.
MATTERS TO BE CONSIDERED:

## Week of February 16, 2004

Wednesday, February 18, 2004

9:30 a.m. Briefing on Status of Office of the Chief Financial Officer Programs, Performance, and Plans (Public Meeting) (Contact: Edward L. New, 301–415–5646).

This meeting will be webcast live at the Web address—http://www.nrc.gov.

## Week of February 23, 2004—Tentative

Wednesday, February 25, 2004

9 a.m. Discussion of Security Issues (Closed—Ex. 1).

Thursday, February 26, 2004

9:30 a.m. Meeting with UK Regulators to Discuss Security Issues (Closed— Ex. 1).

1:30 p.m. Status of Davis Beese Lessons Learned Task Force Issues (Public Meeting) (Contact: Brendan Moroney, 301–415–3974).

This meeting will be webcast live at the Web address—http://www.nrc.gov.

### Week of March 1, 2004—Tentative

Tuesday, March 2, 2004

9:30 a.m. Meeting with Advisory Committee on the Medical Uses of Isotopes (ACMUI) and NRC Staff (Public Meeting) (Contact: Angela Williamson, 301–415–5030).

This meeting will be webcast live at the Web address—http://www.nrc.gov.

Wednesday, March 3, 2004

9:30 a.m. 25th Anniversary Three Mile Island (TMI) Unit 2 Accident Presentation (Public Meeting) (Location: TWFN Auditorium, 11545 Rockville Pike) (Contact: Sam Walker, 301–415–1965).

This meeting will be webcast live at the Web address—http://www.nrc.gov. 2:45 p.m. Discussion of Security Issues (Closed—Ex. 1).

Thursday, March 4, 2004

1:30 p.m. Briefing on Status of Office of Nuclear Material Safety and Safeguards (NMSS) Programs, Performance, and Plans—Waste Safety (Public Meeting) (Contact: Claudia Seelig, 301–415–7243). This meeting will be webcast live at the Web address—http://www.nrc.gov.

### Week of March 8, 2004—Tentative

Tuesday, March 9, 2004

9:30 a.m. Briefing on Status of Office of Nuclear Material Safety and Safeguards (NMSS) Programs, Performance, and Plans—Material Safety (Public Meeting) (Contact: Claudia Seelig, 301–415–7243).

This meeting will be webcast live at the Web address—http://www.nrc.gov. 1:30 p.m. Discussion of Security Issues (Closed—Ex. 1).

### Week of March 15, 2004—Tentative

There are no meetings scheduled for the week of March 15, 2004.

#### Week of March 22, 2004—Tentative

Tuesday, March 23, 2004

9:30 a.m. Briefing on Status of Office of Nuclear Regulatory Research (RES) Programs, Performance, and Plans (Public Meeting) (Contact: Alan Levin, 301–415–6656).

This meeting will be webcast live at the Web address—http://www.nrc.gov.

Wednesday, March 24, 2004

9:30 a.m. Briefing on Status of Office of Nuclear Reactor Regulation (NRR) Programs, Performance, and Plans (Public Meeting) (Contact: Mike Case, 301–415–1275).

This meeting will be webcast live at the Web address—http://www.nrc.gov.

Thursday, March 25, 2004

1:30 p.m. Briefing on Status of Office of Nuclear Security and Incident Response (NSIR) Programs, Performance, and Plans (Public Meeting) (Contact: Jack Davis, 301– 415–7256).

This meeting will be webcast live at the Web address—http://www.nrc.gov.

\* The schedule for Commission meetings is subject to change on short notice. To verify the status of meetings call (recording)—(301) 415–1292. Contact person for more information: Dave Gamberoni, (301) 415–1651.

The NRC Commission Meeting Schedule can be found on the Internet at: http://www.nrc.gov/what-we-do/policy-making/schedule.html.

This notice is distributed by mail to several hundred subscribers; if you no longer wish to receive it, or would like to be added to the distribution, please contact the Office of the Secretary, Washington, DC 20555 (301–415–1969). In addition, distribution of this meeting

notice over the Internet system is available. If you are interested in receiving this Commission meeting schedule electronically, please send an electronic message to dkw@nrc.gov.

Dated: February 12, 2004.

### Dave Gamberoni,

Office of the Secretary.

[FR Doc. 04–3553 Filed 2–13–04; 9:42 am]

BILLING CODE 7590-01-M

# NUCLEAR REGULATORY COMMISSION

## Correction to Biweekly Notice Applications and Amendments to Operating Licenses Involving No Significant Hazards Consideration

On February 3, 2004 (69 FR 5200), the **Federal Register** published the "Biweekly Notice of Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations." On page 5216, Southern Nuclear Operating Company, Inc., *et al.*, Vogtle Electric Generating Plant, Units 1 and 2, "Amendment Nos. 130 and 108" should read "Amendment Nos. 130 and 109."

Dated at Rockville, Maryland, this 10th day of February 2004.

For the Nuclear Regulatory Commission.

## Ledyard B. Marsh,

Director, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 04–3438 Filed 2–17–04; 8:45 am]

# NUCLEAR REGULATORY COMMISSION

[NUREG-1600]

# Revision of the NRC Enforcement Policy: Correction

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Policy statement: correction.

SUMMARY: This document corrects a notice appearing in the Federal Register on January 5, 2004 (69 FR 385)), that clarifies that enforcement action may be taken against non-licensees for violations of the Commission's regulations governing the packaging and transportation of radioactive material. This action is necessary to: (1) Include the deadline for submitting comments on the Enforcement Policy revision, which is March 19, 2004 and (2) correct the methods for providing comments.

### FOR FURTHER INFORMATION CONTACT:

Renée Pedersen, Senior Enforcement Specialist, Office of Enforcement, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, at (301) 415–2742 or e-mail rmp@nrc.gov.

### SUPPLEMENTARY INFORMATION:

1. The **EFFECTIVE DATE** entry is corrected to read as follows:

**EFFECTIVE DATE:** October 1, 2004. Submit comments by March 19, 2004.

2. The **ADDRESSES** entry is corrected to read as follows:

ADDRESSES: Submit written comments to: Michael T. Lesar, Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, Mail Stop: T6D59, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001. Hand deliver comments to: 11555 Rockville Pike, Rockville, Maryland, between 7:30 a.m. and 4:15 p.m., Federal workdays. Copies of comments received may be examined at the NRC Public Document Room, Room O1F21, 11555 Rockville Pike, Rockville, MD. You may also e-mail comments to nrcrep@nrc.gov.

Dated at Rockville, Maryland, this 11th day of February 2004.

For the Nuclear Regulatory Commission.

#### Michael T. Lesar,

Federal Register Liaison Officer. [FR Doc. 04–3442 Filed 2–17–04; 8:45 am] BILLING CODE 7590–01–P

## NUCLEAR REGULATORY COMMISSION

Notice of Clarification to Steam Generator Tube Integrity Event Reporting Guideline in NUREG-1022, "Event Reporting Guidelines 10 CFR 50.72 and 50.73"

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice of clarification in reporting guideline for steam generator tube integrity event.

SUMMARY: The U.S. Nuclear Regulatory Commission plans to make a clarification in the reporting guideline for serious steam generator tube degradation contained within Revision 2 to NUREG-1022, "Event Reporting Guidelines 10 CFR 50.72 and 50.73." The NRC intends to issue an errata to NUREG-1022, Revision 2. The purpose of this clarification is to ensure that the NRC receives timely notification of serious steam generator tube degradation.

**SUPPLEMENTARY INFORMATION:** In NUREG—1022, Revision 2, "Event Reporting Guidelines 10 CFR 50.72 and 50.73," steam generator tube degradation is characterized in Section 3.2.4(A)(3) as being seriously degraded

if the tubing fails to meet the following two performance criteria: (A) Steam generator tubing shall retain structural integrity over the full range of normal operating conditions (including startup, operation in the power range, hot standby, and cooldown and all anticipated transients included in the design specification) and design basis accidents. This includes retaining a margin of 3.0 against burst under normal steady state full power operation and a margin of 1.4 against burst under the limiting design basis accident concurrent with a safe shutdown earthquake. (B) The primary to secondary accident induced leakage rate for the limiting design basis accident, other than a steam generator tube rupture, shall not exceed the leakage rate assumed in the accident analysis in terms of total leakage rate for all steam generators and leakage rate for an individual steam generator. The licensing basis accident analyses typically assume a 1 gallon per minute primary to secondary leak rate per steam generator, except for specific types of degradation at specific locations where the tubes are confined, as approved by the NRC and enumerated in conjunction with the list of approved repair criteria in the licensee's design basis documents. The first performance criteria is commonly referred to as the structural integrity performance criteria and the second criteria is commonly referred to as the accident induced leakage performance criteria. As written, NUREG-1022, Revision 2 implies that the principal safety barrier (i.e., the steam generator tubes in this case) would not be considered seriously degraded if it had either structural or leakage integrity. This is contradictory to existing NRC regulations which require, in part, that the reactor coolant pressure boundary (which includes the steam generator tubes) be designed to permit periodic inspection and testing of important areas and features to assess both their structural and leak-tight integrity (refer to General Design Criterion 32 of Appendix A to 10 CFR part 50) and be designed and tested so as to have an extremely low probability of abnormal leakage, of rapidly propagating failure, and of gross rupture (refer to General Design Criterion 14 of Appendix A to 10 CFR part 50). The regulations, therefore, indicate that both structural and leakage integrity criteria must be satisfied and not meeting either one of the two performance criteria should constitute serious degradation of the principal safety barrier. Accordingly, steam generator tube degradation should be considered

serious if either of the two criteria specified in Section 3.2.4(A)(3) of NUREG-1022, Revision 2, are not satisfied

The intended clarification involves changing the wording in Section 3.2.4(A)(3) of NUREG-1022, Revision 2 (page 39) from "Steam generator tube degradation is considered serious if the tubing fails to meet the following two performance criteria" to "Steam generator tube degradation is considered serious if the tubing fails to meet either of the following two performance criteria."

The NRC will consider any comments it receives pertaining to this intended change in NUREG—1022, Revision 2.

**DATES:** Comment period expires March 19, 2004. Comments submitted after this date will be considered if it is practical to do so, but assurance of consideration cannot be given except for comments received on or before this date.

ADDRESSES: Submit written comments to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Mail Stop T6–D59, Washington, DC 20555–0001, and cite the publication date and page number of this Federal Register notice. Written comments may also be delivered to NRC Headquarters, 11545 Rockville Pike (Room T6–D59), Rockville, Maryland, between 7:30 a.m. and 4:15 p.m. on Federal workdays.

## FOR FURTHER INFORMATION, CONTACT:

Samuel Lee at (301) 415–1061 or by Email to *ssl@nrc.gov*, or Ken Karwoski at (301) 415–2752 or by e-mail to *kjk1@nrc.gov*.

Dated at Rockville, Maryland, this 10th day of February, 2004.

For the Nuclear Regulatory Commission.

### William D. Beckner,

Chief, Reactor Operations Branch, Division of Inspection Program Management, Office of Nuclear Reactor Regulation.

[FR Doc. 04–3441 Filed 2–17–04; 8:45 am] BILLING CODE 7590–01–P