

#### UNITED STATES NUCLEAR REGULATORY COMMISSION REGION IV 611 RYAN PLAZA DRIVE, SUITE 400 ARLINGTON, TEXAS 76011-4005

March 21, 2005

Joseph E. Venable Vice President Operations Waterford 3 Entergy Operations, Inc. 17265 River Road Killona, LA 70066-0751

# SUBJECT: WATERFORD STEAM ELECTRIC STATION, UNIT 3 - NRC RADIATION SAFETY TEAM INSPECTION REPORT 05000382/2005009

Dear Mr. Venable:

On March 4, 2005, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at your Waterford Steam Electric Station, Unit 3 facility. The enclosed report documents the inspection findings, which were discussed at the conclusion of the inspection with Mr. K. Walsh, General Manager, Plant Operations and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The team reviewed selected procedures and records, observed activities, and interviewed personnel. Specifically, the team evaluated the inspection areas within the Radiation Protection Strategic Performance Area that are scheduled for review every two years. These areas are:

- Radiation Monitoring Instrumentation
- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems
- Radioactive Material Processing and Transportation
- Radiological Environmental Monitoring Program and Radioactive Material Control
  Program

Based on the results of this inspection, the NRC has identified one self-revealing, noncited violation of very low safety significance (Green). However, because the finding is of very low safety significance and was entered into your corrective action program, the NRC is treating this finding as noncited violation consistent with Section VI.A of the NRC Enforcement Policy. If you contest the subject or severity of a noncited violation, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington DC 20555-0001; with copies to the Regional Administrator, U.S. Nuclear Regulatory Commission Region IV, 611 Ryan Plaza Drive, Suite 400, Arlington, Texas 76011-4005; the Director, Office of Enforcement, U.S. Nuclear Regulatory Commission, Washington DC 20555-0001; and the NRC Resident Inspector at the Waterford Steam Electric Station, Unit 3 facility.

Entergy Operations, Inc.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <a href="http://www.nrc.gov/reading-rm/adams.html">http://www.nrc.gov/reading-rm/adams.html</a> (the Public Electronic Reading Room).

Sincerely,

#### //RA//

Michael P. Shannon, Chief Plant Support Branch Division of Reactor Safety

Dockets: 50-382 Licenses: NPF-38

Enclosure: NRC Inspection Report w/attachment: Supplemental Information

cc w/enclosure: Senior Vice President and Chief Operating Officer Entergy Operations, Inc. P.O. Box 31995 Jackson, MS 39286-1995

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Director, Nuclear Safety & Regulatory Affairs Waterford 3 SES Entergy Operations, Inc. 17265 River Road Killona, LA 70066-0751

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SISP Review Completed: Yes ADAMS: Yes Do Initials: DRC Publicly Available Non-Publicly Available Sensitive Non-Sensitive

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# U.S. NUCLEAR REGULATORY COMMISSION REGION IV

Dockets:	50-382
Licenses:	NPF-38
Report:	05000382/2005009
Licensee:	Entergy Operations, Inc.
Facility:	Waterford Steam Electric Station, Unit 3
Location:	Hwy. 18 Killona, Louisiana
Dates:	February 28 through March 4, 2005
Inspectors:	Daniel R. Carter, Health Physicist, Plant Support Branch Bernadette D. Baca, Health Physicist, Plant Support Branch Larry T. Ricketson, P.E., Senior Health Physicist, Plant Support Branch Binesh K. Tharakan, Health Physicist, Plant Support Branch
Approved By:	Michael P. Shannon, Chief, Plant Support Branch Division of Reactor Safety

# SUMMARY OF FINDINGS

IR 05000382/2005009; 02/28/05 - 03/04/05; Waterford Steam Electric Station, Unit 3; Radioactive Material Transportation, Cross-Cutting Areas

The report covered a one week period of inspection on site by a team of four region-based inspectors. One finding of very low safety significance (Green) was identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using IMC 0609, "Significance Determination Process". Findings for which the Significance Determination Process does not apply may be "Green" or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

# A. NRC-Identified and Self-Revealing Findings

Cornerstone: Public Radiation Safety

• <u>Green</u>. The team reviewed a self-revealing, noncited violation of 10 CFR 71.5, which occurred when the licensee failed to ship radioactive material correctly. A radioactive shipment classified as an "excepted package-limited quantity" exceeded the external dose rate limitation of 0.5 millirem per hour on the surface of the package. The package recipient identified dose rates of 1.2 millirems per hour on the exterior surface of the package and notified the licensee of the problem.

The finding is greater than minor because it was associated with a Public Radiation Safety cornerstone attribute (human performance) and it affected the associated cornerstone objective because the failure to correctly ship radioactive material decreases the licensee's assurance that the public will not receive unnecessary dose. However, this finding cannot be evaluated by the Public Radiation Safety Significance Determination Process because it did not involve radioactive shipments classified as Schedule 5 through 11, as described in NUREG-1660, and it did not fit traditional enforcement. Therefore, the finding was reviewed by NRC management and determined to be of very low safety significance. Additionally, this finding had cross-cutting aspects associated with human performance. Licensee personnel directly contributed to the finding when they failed to ensure that the package did not exceed the dose rate limit. The finding was placed into the licensee's corrective action program as Condition Report WF3-2003-03514 (Section 2PS2).

# **REPORT DETAILS**

# 2. RADIATION SAFETY Cornerstones: Occupational Radiation Safety [OS] and Public Radiation Safety [PS]

# 2OS3 Radiation Monitoring Instrumentation and Protective Equipment (71121.03)

#### a. Inspection Scope

This area was inspected to determine the accuracy and operability of radiation monitoring instruments that are used for the protection of occupational workers and the adequacy of the program to provide self-contained breathing apparatus to workers. The team used the requirements in 10 CFR Part 20 and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- Calibration of area radiation monitors associated with transient high and very high radiation areas and post-accident monitors used for remote emergency assessment
- Calibration of portable radiation detection instrumentation, electronic alarming dosimetry, and continuous air monitors used for job coverage
- Calibration of whole body counting equipment and radiation detection instruments utilized for personnel and material release from the radiologically controlled area
- Self-assessments and audits
- Corrective action program reports since the last inspection
- Calibration expiration and source response check currency on radiation detection instruments staged for use
- The licensee's capability for refilling and transporting self-contained breathing apparatus air bottles to and from the control room and operations support center during emergency conditions, status of self-contained breathing apparatus staged and ready for use in the plant and associated surveillance records, and personnel qualification and training
- Qualification documentation for onsite personnel designated to perform maintenance on the vendor-designated vital components, and the vital component maintenance records for self-contained breathing apparatus units

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

Licensee Event Reports

Licensee action in cases of repetitive deficiencies or significant individual deficiencies

The inspector completed 9 of the required 9 samples.

b. Findings

No findings of significance were identified.

# 2PS1 Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems (71122.01)

a. Inspection Scope

This area was inspected to ensure that the gaseous and liquid effluent processing systems are maintained so that radiological releases are properly mitigated, monitored, and evaluated with respect to public exposure. The team used the requirements in 10 CFR Part 20, 10 CFR Part 50 Appendices A and I, the Offsite Dose Calculation Manual, and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- The most current radiological effluent release reports, changes to radiation monitor setpoint calculation methodology, anomalous sampling results, effluent radiological occurrence performance indicator incidents
- Gaseous and liquid release system component configurations
- Radioactive liquid and gaseous effluent release permits and dose projections to members of the public
- Changes made by the licensee to the ODCM, the liquid or gaseous radioactive waste system design, procedures, or operation since the last inspection
- Monthly, quarterly, and annual dose calculations
- Surveillance test results involving air cleaning systems and stack or vent flow rates
- Instrument calibrations of discharge effluent radiation monitors and flow measurement devices, effluent monitoring system modifications, effluent radiation monitor alarm setpoint values, and counting room instrumentation calibration and quality control
- Interlaboratory comparison program results
- Audits, self-assessments and corrective action reports performed since the last inspection

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

- Licensee event reports and special reports
- Abnormal releases

The inspector completed 10 of the required 10 samples.

b. Findings

No findings of significance were identified.

# 2PS2 Radioactive Material Processing and Transportation (71122.02)

a. Inspection Scope

This area was inspected to verify that the licensee's radioactive material processing and transportation program complies with the requirements of 10 CFR Parts 20, 61, and 71 and Department of Transportation regulations contained in 49 CFR Parts 171-180. The team interviewed licensee personnel and reviewed:

- The radioactive waste system description, recent radiological effluent release reports, and the scope of the licensee's audit program
- Liquid and solid radioactive waste processing systems configurations, the status and control of any radioactive waste process equipment that is not operational or is abandoned in place, changes made to the radioactive waste processing systems since the last inspection, and current processes for transferring radioactive waste resin and sludge discharges
- Radio-chemical sample analysis results for radioactive waste streams and use of scaling factors and calculations to account for difficult-to-measure radionuclides
- Shipping records for non-excepted package shipments
- Licensee audits, vendor audits, state agency reports, self-assessments, and corrective action reports performed since the last inspection

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

- Shipment packaging, surveying, labeling, marking, placarding, vehicle checking, driver instructing, and disposal manifesting
- Licensee event reports, and special reports

The inspector completed 6 of the required 6 samples.

b. Findings

<u>Introduction</u>. The team reviewed a Green self-revealing, noncited violation of 10 CFR 71.5, which occurred when the licensee failed to ship radioactive material correctly.

<u>Description</u>. On November 10, 2003, the licensee was notified about a problem with Shipment Number 03-3054. The radioactive shipment had been transported as an "excepted package-limited quantity." The notification came from the recipient, who identified that the contact dose rate on the external surface of the package exceeded the 0.5 millirem per hour limit allowed by regulation. The recipient measured a contact radiation level of 1.2 millirem per hour. The licensee's survey conducted before shipment indicated that the highest dose rate on the package was 0.2 millirem per hour. The package contained refueling equipment. The licensee was unable to get a second, verification survey of the package from the recipient as the items were already unloaded by the time the licensee was notified. The licensee's investigation into the issue could not conclusively determine the cause for the increased external radiation dose rate. The inconsistent measurements came from either: (1) the package contents had not been correctly blocked and braced and the contents shifted, (2) the package was not properly surveyed, or (3) an inappropriate survey instrument was used.

<u>Analysis</u>. The failure to ship limited quantity radioactive material in accordance with federal regulations is a performance deficiency. The finding is greater than minor because it was associated with a Public Radiation Safety cornerstone attribute (human performance) and it affected the associated cornerstone objective because the failure to correctly ship radioactive material decreases the licensee's assurance that the public will not receive unnecessary dose. However, this finding cannot be evaluated by the Public Radiation Safety Significance Determination Process because it does not involve radioactive shipments classified as Schedule 5 through 11, as described in NUREG-1660, "U.S. Specific Schedules of Requirements for Transport of Specified Types of Radioactive Material Consignments," and it does not fit traditional enforcement. Therefore, the finding was reviewed by NRC management and determined to be of very low safety significance.

Additionally, this finding had cross-cutting aspects associated with human performance. Licensee personnel involved with the shipment directly contributed to the finding when either the package was not correctly blocked-and-braced, properly surveyed, or an inappropriate survey instrument was used.

<u>Enforcement</u>. 10 CFR 71.5 states, in part, that each licensee who transports licensed material on public highways shall comply with applicable requirements of the DOT regulations in 49 CFR parts 170 through 189. Specifically, 49 CFR 173.421(a)(2) requires that excepted packages for limited quantities of radioactive material have a

Enclosure

radiation level at any point on the external surface of the package which does not exceed 0.5 millirem per hour. However, the licensee failed to ensure that the package did not exceed the dose rate limit. Because this violation was of very low safety significance and was entered into the licensee's corrective action program as Condition Report WF3-2003-03514, it is being treated as a non-cited violation, consistent with Section VI.A of the NRC Enforcement Policy: NCV 05000382/0509-01, Failure to ship radioactive material correctly.

# 2PS3 <u>Radiological Environmental Monitoring Program and Radioactive Material Control</u> <u>Program (71122.03)</u>

a. Inspection Scope

This area was inspected to ensure that the radiological environmental monitoring program verifies the impact of radioactive effluent releases to the environment and sufficiently validates the integrity of the radioactive gaseous and liquid effluent release program; and that the licensee's surveys and controls are adequate to prevent the inadvertent release of licensed materials into the public domain. The team used the requirements in 10 CFR Part 20, 10 CFR Part 50, Appendix I, the Offsite Dose Calculation Manual, and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- Annual environmental monitoring reports
- A sampling of air sampling stations and thermoluminescence dosimeter (TLD) monitoring stations
- Operability, calibration, and maintenance of meteorological instruments
- Each event documented in the Annual Environmental Monitoring Report which involved a missed sample, inoperable sampler, lost TLD, or anomalous measurement
- Significant changes made by the licensee to the ODCM as the result of changes to the land census or sampler station modifications since the last inspection
- Quality control program, interlaboratory comparison program results, and vendor audits
- Locations where the licensee monitors potentially contaminated material leaving the controlled access area and the methods used for control, survey, and release from these areas

- Type of radiation monitoring instrumentation used to monitor items released, survey and release criteria of potentially contaminated material, radiation detection sensitivities, procedural guidance, and material release records
- Audits, self-assessments and corrective action reports performed since the last inspection

Either because the conditions did not exist or an event had not occurred, no opportunities were available to review the following items:

- Licensee event reports and special reports
- Collection and preparation of environmental samples
- Calibration and maintenance records for air samplers, composite water samplers, and environmental sample radiation measurement instrumentation

The inspector completed 10 of the required 10 samples.

b. Findings

No findings of significance were identified.

# 4. OTHER ACTIVITIES

# 4OA2 Problem Identification and Resolution

a. <u>Inspection Scope</u>

The team evaluated the effectiveness of the licensee's problem identification and resolution process with respect to the following inspection areas:

- Radiation Monitoring Instrumentation (Section 20S3)
- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems (Section 2PS1)
- Radioactive Material Processing and Transportation (Section 2PS2)
- Radiological Environmental Monitoring Program and Radioactive Material Control Program (Section 2PS3)

# a. <u>Findings and Observations</u>

No findings of significance were identified.

#### 4OA4 Cross-Cutting Aspects of Findings

Section 2PS2 describes an issue with a human performance cross-cutting aspects which involved the failure of personnel to ship radioactive material correctly.

#### 4OA6 Management Meetings

## Exit Meeting Summary

On March 4, 2005, the team presented the inspection results to Mr. K. Walsh, General Manager, Plant Operations and other members of the staff who acknowledged the findings. The team confirmed that proprietary information was not provided or examined during the inspection.

# SUPPLEMENTAL INFORMATION

# **KEY POINTS OF CONTACT**

# <u>Licensee</u>

- J. Amato, Environmental/Effluent Supervisor (Acting)
- K. Cambre, Auxiliary Operator, Operations
- L. Dauzat, Supervisor, Radiation Protection Operations
- A. Dubois, REMP Technician, Chemistry
- L. Gaubert, Senior Health Physics Technician, Radiation Protection
- D. Haslauer, Radiation Monitoring Instrumentation Engineer
- J. Hornsby, Superintendent, Chemistry
- P. Kelly, Radiation Protection Supervisor, Radwaste/Radioactive Material Control
- S. Landry, Radiation Protection Specialist, Radwaste/Radioactive Material Control
- H. Landeche, Senior Instrument and Calibration Technician
- D. Newman, Supervisor, Radiation Protection
- L. Rome, Senior Instrument and Calibration Technician
- R. Sebring, Senior Health Physics Technician, Radiation Protection
- D. Stevens, Senior Health Physics/Chemistry Specialist
- K. Walsh, General Manager, Plant Operations
- G. Watson, Senior Instrument and Calibration Technician

# <u>NRC</u>

- R. Azua, Senior Resident Inspector
- G. Larkin, Resident Inspector

# LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

<u>Opened</u>

None

# Opened and Closed During this Inspection

05000382/200509-01 NCV Failure to ship radioactive material correctly (Section 2PS2)

# LIST OF DOCUMENTS REVIEWED

#### Section 2OS3: Radiation Monitoring Instrumentation and Protective Equipment

#### Audits, Self Assessments, and Surveillances

LO-WLO-2004-0001-0108 Radiation Monitoring Instrumentation, 11/29/04-12/07-04 SQA2003-00626 Analytics, Inc. (A427X) - Review of NUPIC Audit 18509, 8/13/2003

#### Condition Report/Disposition Requests

CR-WF3-2003-00649, CR-WF3-2003-01762, CR-WF3-2003-01987, CR-WF3-2003-03102, CR-WF3-2003-03249, CR-WF3-2004-00809, CR-WF3-2004-01940, CR-WF3-2004-02022, CR-WF3-2004-02149, CR-WF3-2004-02840, CR-WF3-2004-03318, CR-WF3-2004-03936, CR-WF3-2004-03944, CR-WF3-2004-03947, CR-WF3-2004-03949, CR-WF3-2005-00437

#### Procedures

HP-002-350	Operation of the Whole Body Counting System, Revision 7
HP-002-360	Operation of the Eberline Model BC-4 and SAC-4 Counters, Revision 2
HP-002-450	Calibration and Quality Assurance of the Whole Body Counting System,
	Revision 11
HP-002-470	Efficiency Determination and Statistical Check of Counter/Scalars, Revision 6
HP-002-482	Calibration of MSA Escort ELF Pump Air Sampler, Revision 2
RP-301	Radiation Protection Instrument Control, Revision 0
RP-302	Operation of Radiation Protection Instrumentation, Revision 1
RP-303	Source Checking of Radiation Protection Instrumentation, Revision 6
RP-304	Operation of Counting Equipment, Revision 2
ENS-RP-306	Operation and Calibration of Eberline PM-7, Revision 2
RP-307	Operation and Calibration of Eberline Personnel Contamination Monitors
	(PCM), Revision 3
DD 211	Electronic Alarming Desimptors, Povision 0

RP-311 Electronic Alarming Dosimeters, Revision U

Instrument Calibration Datasheets and Work Orders

Westside Access Controlled Access Area Whole Body Counter Calibration, February 18, 2005 SAC-4 Calibration, January 18, 2005 ASC-001 BC-4 Calibration, November 1, 2004 HP-CS-014 WR Telepole Calibration, November 9, 2004 HP-DR-344 Personnel Contamination Monitor Calibration, July 28, 2004 HP-DS-038

Small Tool Monitor Calibration, August 26, 2004 HP-DS-064

105013 Electronic Alarming Dosimeter Calibration, November 17, 2004

Maintenance Action Item # 50232129

Maintenance Action Item # 50232291

Maintenance Action Item # 50975879

# Section 2PS1: Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems

#### Audits and Self-Assessments

Snap Shot Assessment on REMP/Effluents dated March 24, 2003 Quality Assurance Audit of Effluent and Environmental Monitoring, WF3-2003-0024 NUPIC Audit Report, QAG60-GA-03

#### Condition Report/Disposition Requests

CR-WF3-2003-00849, CR-WF3-2003-00925, CR-WF3-2003-01032, CR-WF3-2003-02476, CR-WF3-2003-02886, CR-WF3-2004-00119, CR-WF3-2004-02161, CR-WF3-2004-02470, CR-WF3-2004-02622, CR-WF3-2005-00100, CR-WF3-2005-00333, CR-WF3-2005-00675, CR-WF3-2005-00644, CR-WF3-2005-01061, CR-WF3-2005-01068, CR-WF3-2005-01128, CR-WF3-2005-02796,

# **Procedures**

RP-502	Inspection and Maintenance of Respiratory Protection Equipment, Revision 2
HP-002-603	Breathing Air Filtration Panel, Revision 3
RP-504	Breathing Air, Revision 2
CE-001-004	Periodic Analysis Scheduling Program, Revision 4
CE-003-509	Routine Filter Repalcement and Grab Sampling for PIG Monitors and WRGM, Revision 3
CE-003-515	Gaseous Radiological Waste Release Permit, Revision 6
CE-003-700	General Grab Sampling Techniques, Revision 11
MI-003-360	Containment High Range Safety Channel A or B Area Radiation Monitor Calibration ARMIR5400A or ARMIR5400B, Revision 4
MI-003-457	Steam Generator 1 and 2 Liquid Radiation Monitor Channel Calibration PRMIR0100X, Revision 5
MI-003-371	Fuel Handling Building Ventilation System Emergency Exhaust High Range Noble Gas Radiation Monitor Channel Calibration PRMIR3032, Revision 5
MI-003-382	Plant Stack High Range Noble Gas Radiation Monitor Functional Test PRMIR0110, Revision 3
MI-003-383	Plant Stack High Range Noble Gas Radiation Monitor Channel Calibration PRMIR0110, Revision 6

# Calibration Packages

LS-6500, Liquid Scintillation Counter Calibration, January 5, 2005 G5000, Gross Alpha Counter Calibration, May 21, 2004 Germanium Lithium (GeLi) Detector #4, Gamma Spectroscopy Calibration, December 4, 2002 Maintenance Action Item # 50373969 Maintenance Action Item # 50690565 Maintenance Action Item # 50994716

Liquid Release Permits LB2003-0160, LC2003-0172, LC2003-0174, LB2003-0181, LB2004-0037, LC2004-0049, LB2004-0059, LC2004-0061 Gas Release Permits

GC2003-0004, GC2003-0005, GC2003-0006, GC2003-0007, GC2003-0008, GC2003-0009, GB2003-0033, GB2004-0013

<u>Miscellaneous</u> 2002 and 2003 Annual Radiological Effluent Release Reports

## Section 2PS2: Radioactive Material Processing and Transportation

Audits and Self Assessments

LO-WLO-2004-0001-CA-4	Radioactive Waste Reduction and Transportation
LO-WLO-2004-0136-CA-1	Radioactive Material Processing and Transportation
SQA 2003-00551	Duratek/Chem-Nuclear Systems NUPIC Audit 18545
SQA 2003-00608	Duratek (H210) NUPIC Audits 03-01-GA and 03-02-GA

Condition Report/Disposition Requests

WF3-2003-1092, WF3-2003-3514, WF3-2004-2945, WF3-2004-03259

# Procedures

ENS-RW-101	Radioactive Waste Management, Revision 1
ENS-RW-102	Radioactive Shipping Procedures, Revision 2
ENS-RW-104	Scaling Factors, Revision 0
ENS-RW-105	Process Control Program, Revision 2
HP-002-224	Spent Resin Operations, Revision 4
OP-007-005	Resin Waste Management, Revision 11
RW-001-221	Radwaste Processing Standard, Revision 0
RW-002-200	Collection and Packing of Solid Radioactive Waste, Revision 15
RW-002-210	Radioactive Waste Solidification/Dewatering, Revision 14
RW-002-300	Receipt, Storage, and Loading of Shipping Containers, Revision 14
RW-002-310	Storage of Radioactive Waste, Revision 7

Shipping Paper Work and Manifests

Radioactive Material Shipment Log 2003 - 2005 Radioactive Waste Shipment Log 2003 - 2005 Shipment Numbers: 03-1003, 03-3064, 04-1009, 04-1013, 04-1021, and 05-1001

#### Miscellaneous

2002 and 2003 10 CFR Part 61 sample results and resulting scaling factors ChemTrec Registration (24 hour Emergency Telephone Service) NTP-2007, Radwaste Services Training, Revision 12 Training Records for two Qualified Shippers WLP-RW-HAZMATR, General Training Lesson Plan, Hazardous Materials Requalification Training, Revision 4

# Section 2PS3: Radiological Environmental Monitoring Program (REMP) And Radioactive Material Control Program

Audits and Asses	sments	
QA-6-2003-WF3-1		Quality Assurance Audit Report (Effluent and Environmental Monitoring)
I O-WI O-2004-0137-CA-1		Radioactive Material Control
LO-WLO-2004-00081 CA-01TLD		Processing Annual Assessment
Condition Report/	Disposition Reque	ests
2003-00769, 200	3-01097, 2003-023	396, 2004-00195, 2004-03199, 2004-03920
Procedures		
CE-003-526	Collection and Pr	eparation of REMP Liquid Samples, Revision 1
CE-003-527	Collection of Milk	Samples, Revision 1
CE-003-531	Collection and Press	eparation of REMP Air Samples, Revision 1
CE-003-532	Preparation and I Revision 1	Distribution of REMP Thermoluminescent Dosimeters,
HP-001-152	Radioactive Mate	rial Control, Revision16
MI-003-395 UNT-0050014	Primary Meteorolo Offsite Dose Calo	ogical Tower Instrument Calibration EM IM0100, Revision culation Manual, Revision 7
Calibrations		

Work Order 50993310 01	Secondary Meteorological Tower Monitor Loop
Work Order 50994501 02	Primary Meteorological Tower Monitor Loop

Miscellaneous 2002 and 2003 Annual Radiological Environmental Operating Report

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