

# UNITED STATES NUCLEAR REGULATORY COMMISSION

#### REGION IV 611 RYAN PLAZA DRIVE, SUITE 400 ARLINGTON, TEXAS 76011-4005

March 11, 2004

Harold B. Ray, Executive Vice President San Onofre, Units 2 and 3 Southern California Edison Co. P.O. Box 128, Mail Stop D-3-F San Clemente, CA 92674-0128

SUBJECT: SAN ONOFRE NUCLEAR GENERATING STATION - NRC RADIATION SAFETY TEAM INSPECTION REPORT 05000361/2004006; 05000362/2004006

Dear Mr. Ray:

On January 30, 2004, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at your San Onofre Nuclear Generating Station, Units 2 and 3, facility. The enclosed Radiation Safety Team inspection report documents the inspection findings, which were discussed at the conclusion of the inspection with Messrs. J. Wambold and D. Nunn, and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The team reviewed selected procedures and records, observed activities, and interviewed personnel. Specifically, the team evaluated the inspectible areas within the Radiation Protection Strategic Performance Area that are scheduled for review every two years. These areas are:

- Radiation Monitoring Instrumentation
- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems
- Radioactive Material Processing and Transportation
- Radiological Environmental Monitoring Program and Radioactive Material Control Program

This inspection report documents two self-revealing findings of very low safety significance (Green). However, because of the very low safety significance and because the findings were entered into your corrective action program, the NRC is treating these findings as noncited violations (NCVs) consistent with Section V1.A of the NRC Enforcement Policy. Additionally, one licensee-identified violation was determined to be of very low safety significance and is listed in Section 4OA7 in this report. If you contest the violation's or significance of these NCVs, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001, with copies to the Regional Administrator, U.S. Nuclear Regulatory Commission, Region IV, 611 Ryan Plaza Drive, Suite 400, Arlington, Texas 76011; the Director, Office of Enforcement, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; and the NRC Resident Inspector at the San Onofre Station facility.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <a href="http://www.nrc.gov/reading-rm/adams.html">http://www.nrc.gov/reading-rm/adams.html</a> (the Public Electronic Reading Room). Should you have any questions concerning this inspection, we will be pleased to discuss them with you.

Sincerely,

#### //RA//

Troy W. Pruett, Chief Plant Support Branch Division of Reactor Safety

Dockets: 50-361

50-362

Licenses: NPF-10

NPF-15

#### Enclosure:

NRC Inspection Report 05000361/2004006; 05000362/2004006

#### cc w/enclosure:

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Dr. Raymond Waldo Southern California Edison Company San Onofre Nuclear Generating Station P. O. Box 128 San Clemente, CA 92674-0128 A. Edward Scherer Southern California Edison San Onofre Nuclear Generating Station P.O. Box 128 San Clemente, CA 92674-0128 Southern California Edison Co.

SONGS Site Secretary (SFN1)

Electronic distribution by RIV:
Regional Administrator (BSM1)
DRP Director (ATH)
DRS Director (DDC)
Senior Resident Inspector (CCO1)
Branch Chief, DRP/C (KMK)
Senior Project Engineer, DRP/C (WCW)
Staff Chief, DRP/TSS (PHH)
RITS Coordinator (NBH)
Rebecca Tadesse, OEDO RIV Coordinator (RXT)

ADAMS: ■ Yes	No In	itials: <u>jlh</u>		
■ Publicly Available	Non-Publich	y Available	Sensitive	Non-Sensitive

## DOCUMENT: R:\ SO23\SO2004-06RP-DRC.wpd

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RIV:DRS\PSB	PSB	PSB(TL)	PSB Team Leader	C:PSB
PJElkmann:jlh	LTRicketson	MPShannon	DRCarter	TWPruett
**	/RA/	'RA/ /RA/		/RA/
/ /04	2/ 19 /04	2/ 19 /04	2/20 /04	3/10 /04
DRP/A	C:PSB			
KKennedy	TWPruett			
/RA/	/RA/			

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T=Telephone

E=E-mail

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<sup>\*\*</sup>Not available for concurrence prior to timeliness date

# U.S. NUCLEAR REGULATORY COMMISSION REGION IV

Dockets: 50-361, 50-362

Licenses: NPF-10, NPF-15

Report: 05000361/2004006 and 05000362/2004006

Licensee: Southern California Edison Co.

Facility: San Onofre Nuclear Generating Station, Units 2 and 3

Location: 5000 S. Pacific Coast Hwy.

San Clemente, California

Dates: January 26 through January 30, 2004

Inspectors: D. Carter, Health Physicist-Team Leader

M. Shannon, Senior Health Physicist L. Ricketson, Senior Health Physicist

P. Elkmann, Emergency Preparedness Inspector

Approved By: Troy W. Pruett, Chief

Plant Support Branch

Division of Reactor Projects

#### SUMMARY OF FINDINGS

San Onofre Nuclear Generating Station, Units 2 & 3 NRC Inspection Report 05000361/04-06; 05000362/04-06 Inspection Dates: January 26 through 30, 2004

IR 05000361/2004-06, 05000362/2004-06; 01/26 -30/2004; San Onofre Nuclear Generating Station, Units 2 and 3; Radioactive Material Control Program; Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems

The report covered a one week period of inspection on site by a team of four region-based health physics inspectors. Two findings of very low safety significance (Green) were identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using IMC 0609, "Significance Determination Process," (SDP). Findings for which the SDP does not apply may be "Green" or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

## A. NRC-Identified and Self-Revealing Findings

Cornerstone: Public Radiation Safety

• Green. A self-revealing, non-cited violation of Technical Specification 5.5.2.1 was reviewed by the team because the licensee failed to have all the required effluent monitors channels operable or implement auxiliary sampling requirements as specified in the Offsite Dose Calculation Manual. Specifically, the Unit 3 condenser air ejector monitor (3-7870) was declared inoperable on May 26, 2003, at 5:40 p.m. and auxiliary sampling equipment was initially put in place. However, from 8:00 p.m. on May 27, 2003, until 9:55 a.m. on May 28, 2003, continuous particulate and iodine samples were not obtained via the compensatory sampling device because a chemistry technician did not install the necessary filters. The situation was discovered during the successive, routine filter change out.

The finding was greater than minor because it was associated with cornerstone attributes (radiation monitoring and chemistry technician performance) and affected the associated cornerstone objective (to ensure adequate protection of public health and safety from exposure to radioactive materials released into the public domain). The finding involved an occurrence in the licensee's radiological effluent monitoring program that was contrary to Offsite Dose Calculation Manual requirements; therefore, it was evaluated using the Public Radiation Safety Significance Determination Process. The finding had very low significance because the finding was not a radioactive material control finding, but it was an effluent release program finding. The finding impaired the licensee's ability to assess dose; however, the licensee was able to assess dose to the public through use of an operational noble gas monitor (3RT-7818) in the condenser evacuation system and the dose did not exceed 10 CFR Part 50 Appendix I values (Section 2PS1).

Green. A self-revealing, non-cited violation of Technical Specification 5.5.1 was
reviewed by the team because the licensee failed to follow radiation protection
procedures related to the release of radioactive material from the protected area.
Specifically, on November 6, 2003, the licensee discovered one magenta-painted hose
in a 55-gallon drum which had been stored in the Mesa area (outside the protected
area) since January 1998. The licensee surveyed the hose and determined that it
contained a total fixed activity of 40 nanocuries.

The finding was greater than minor because it is associated with the cornerstone attribute (material release) and it affected the associated cornerstone objective (to ensure adequate protection of public health and safety from exposure to radioactive material released into the public domain). The finding involved an occurrence in the licensee's radioactive material control program that was contrary to licensee procedures. This finding was evaluated as having very low safety significance using the Public Radiation Safety Significance Determination Process because the finding was a radioactive material control issue, was not a transportation issue, public exposure was not greater than 5 millirem, and there were less than 5 occurrences over the past rolling 8 quarters (Section 2PS3).

# B. <u>Licensee Identified Vio</u>lation

A violation of very low safety significance (Green) which was identified by the licensee was reviewed by the team. Corrective actions taken or planned by the licensee have been entered into the licensee's corrective action program. This violation and corrective action tracking number is listed in Section 4OA7.

#### **REPORT DETAILS**

#### 2. RADIATION SAFETY

Cornerstones: Occupational Radiation Safety [OS] and Public Radiation Safety [PS]

2OS3 Radiation Monitoring Instrumentation and Protective Equipment (71121.03)

## a. <u>Inspection Scope</u>

This area was inspected to determine the accuracy and operability of radiation monitoring instruments that are used for the protection of occupational workers and the adequacy of the program to provide self-contained breathing apparatus (SCBA) to workers. The team used the requirements in 10 CFR Part 20 and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- Calibration data for 6 area radiation monitors associated with transient high and very high radiation areas and 12 post-accident monitors used for remote emergency assessment
- Calibration data for 4 portable radiation detection instrumentation, 10 electronic alarming dosimeters, and one continuous air monitor
- Calibration data for 2 whole body counters located at the 70 foot access facility,
   6 radiation detection instruments, and 9 personnel monitors utilized for material release from the radiologically controlled area
- Audit records (No LER's or self-assessments were submitted during the inspection period)
- Eighteen corrective action program reports since the last inspection
- Repetitive deficiencies or significant individual deficiencies which occurred during the inspection period (No repetitive or significant individual deficiencies occurred)
- Calibration expiration and source response check currency on 4 radiation detection instruments staged for use
- The licensee's capability for refilling and transporting SCBA air bottles to and from the control room and operations support center during emergency conditions, status of SCBA staged and ready for use in the plant and associated surveillance records, and personnel qualification and training
- Qualification documentation for onsite personnel designated to perform maintenance on the vendor-designated vital components, and the vital component maintenance records for SCBA units

The inspector completed 9 of the required 9 samples.

# b. <u>Findings</u>

No findings of significance were identified.

## 2PS1 Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems (71122.01)

## a. <u>Inspection Scope</u>

This area was inspected to ensure that the gaseous and liquid effluent processing systems are maintained so that radiological releases are properly mitigated, monitored, and evaluated with respect to public exposure. The team used the requirements in 10 CFR Part 20, 10 CFR Part 50 Appendices A and I, the Offsite Dose Calculation Manual (ODCM), and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- Self assessments, audits, and the most current radiological effluent release reports (No changes to radiation monitor setpoint calculation methodology, anomalous sampling results, or effluent radiological occurrence performance indicator incidents occurred during the inspection period)
- All gaseous and liquid release system component configurations
- Routine sample collection and analysis of radioactive gaseous effluent, two
  radioactive liquid and gaseous effluent release permits, and associated dose
  projections to members of the public
- Abnormal releases (None were identified)
- Significant changes made by the licensee to the ODCM, the liquid or gaseous radioactive waste system design, procedures, or plant operation which occurred during the inspection period (No significant changes were identified)
- Monthly, quarterly, and annual dose calculations included in the effluent release reports
- Three surveillance test results involving air cleaning systems and associated vent flow rates
- Instrument calibration data for five discharge effluent radiation monitors and flow measurement devices, associated effluent radiation monitor alarm setpoint values, and daily counting room instrumentation calibration and quality control checks (No effluent monitoring system modifications occurred during the inspection period)
- Interlaboratory comparison program results

 Audit reports, self-assessments, and 12 corrective action reports performed during the inspection period (No LER's or special reports occurred during the inspection period)

The inspector completed 10 of the required 10 samples.

## b. Findings

Introduction. A Green self-revealing, non-cited violation of the Technical Specification 5.5.2.1 was reviewed by the team. The licensee failed to have all the required effluent monitor channels operable or implement auxiliary sampling equipment as required by the ODCM.

<u>Description</u>. The Unit 3 condenser air ejector monitor (3-7870) was declared inoperable on May 26, 2003, at 5:40 p.m. Auxiliary sampling equipment was put in place. However, from 8:00 p.m. on May 27, 2003, until 9:55 a.m. on May 28, 2003, continuous particulate and iodine samples were not obtained via the compensatory sampling device because a chemistry technician did not install the necessary filters. The situation was discovered during the successive, routine filter change out.

Analysis. The failure to have an operable monitor or implement auxiliary compensatory actions was a performance deficiency. The finding was greater than minor because it was associated with cornerstone attributes (radiation monitoring and chemistry technician performance) and affected the associated cornerstone objective (to ensure adequate protection of public health and safety from exposure to radioactive materials released into the public domain). The finding involved an occurrence in the licensee's radiological effluent monitoring program that was contrary to ODCM requirements, therefore it was evaluated using the Public Radiation Safety Significance Determination Process. The finding had very low significance because the finding was not a radioactive material control finding, but it was an effluent release program finding. The finding impaired the licensee's ability to assess dose; however, the licensee was able to assess dose to the public through use of an operational noble gas monitor (3RT-7818) in the condenser evacuation system and the dose did not exceed 10 CFR Part 50 Appendix I values.

Enforcement. Technical Specification 5.5.2.1 requires that the ODCM shall be established, implemented, and maintained. ODCM Section 4.2.1 requires the radioactive gaseous effluent monitoring instrumentation channels shown in Table 4-3 to be operable. Table 4-3, Sections 2b and 2c, include channels for an iodine sampler and a particulate sampler, respectively. Action 40, states, in part, that with the number of channels operable less than required by the "Minimum Channel Operable" requirement, effluent releases via the effected pathway may continue, provided samples are continuously collected with auxiliary sampling equipment. The licensee violated this requirement when filters were not installed in the auxiliary sampling equipment. However, because the failure to have an operable effluent monitor and auxiliary sampling was of very low safety significance and has been entered into the station's corrective action program as Action

Request 030600245, this violation is being treated as a non-cited violation, consistent with Section VI.A of the NRC Enforcement Policy: NCV 050-362/0406-01, Failure to have all the required effluent monitor channels operable or implement auxiliary sampling equipment.

## 2PS2 Radioactive Material Processing and Transportation (71122.02)

#### a. Inspection Scope

This area was inspected to verify that the licensee's radioactive material processing and transportation program complies with the requirements of 10 CFR Parts 20, 61, and 71 and Department of Transportation regulations contained in 49 CFR Parts 170-189. The team interviewed licensee personnel and reviewed:

- The final safety analysis report radioactive waste system description, recent radiological effluent release reports, and the scope of the licensee's audit program
- Liquid and solid radioactive waste processing systems configurations, the status
  and control of all radioactive waste process equipment that is not operational or
  is abandoned in place, changes made to the radioactive waste processing
  systems since the last inspection (No changes were identified during the
  inspection period), and current processes for transferring radioactive waste resin
  and sludge discharges
- 2001 and 2002 radio-chemical sample analysis results for radioactive waste streams and use of scaling factors and calculations to account for difficult-to-measure radionuclides
- Shipment packaging, surveying, labeling, marking, placarding, vehicle checking, driver instructing, and disposal manifesting (No shipments were performed during the on-site inspection)
- Shipping records for six non-excepted package shipments and licenses
- Audits, self-assessments, and eight corrective action reports performed since the last inspection (No LER's or special reports occurred during the inspection period)

The inspector completed 5 of the required 6 samples. The inspector was unable to observe the preparation and shipment of radioactive material because no shipments were conducted during the on-site inspection period.

#### b. Findings

No findings of significance were identified.

# 2PS3 Radiological Environmental Monitoring Program and Radioactive Material Control Program (71122.03)

# a. <u>Inspection Scope</u>

This area was inspected to ensure that the Radiological Environmental Monitoring Program (REMP) verifies the impact of radioactive effluent releases to the environment and sufficiently validates the integrity of the radioactive gaseous and liquid effluent release program; and that the licensee's surveys and controls are adequate to prevent the inadvertent release of licensed materials into the public domain. The team used the requirements in 10 CFR Part 20, 10 CFR Part 50, Appendix I, the ODCM, and the licensee's procedures required by technical specifications as criteria for determining compliance. The team interviewed licensee personnel and reviewed:

- Annual environmental monitoring reports
- Nine air sampling stations and thermoluminescence dosimeter (TLD) monitoring stations
- Collection and preparation of environmental samples (None were performed during the on-site inspection)
- Operability, calibration, and maintenance records of all meteorological instruments
- Events documented in the Annual Environmental Monitoring Report which involved a missed sample, inoperable sampler, lost TLD, or anomalous measurement (No examples were identified)
- Significant changes made by the licensee to the ODCM as the result of changes to the land census or sampler station modifications since the last inspection (No significant changes were identified)
- Calibration and maintenance records for ten air samplers, composite water samplers, and radiation measurement instrumentation, quality control program, interlaboratory comparison program results, and vendor audits (The licensee did not have composite water samplers)
- All locations where the licensee monitors potentially contaminated material leaving the radiologically controlled area and the methods used for control, survey, and release from these areas
- Radiation monitoring instrumentation used to monitor items released, survey and release criteria of potentially contaminated material, radiation detection sensitivities, procedural guidance, and material release records

 Six audits, 5 self-assessments, and 19 corrective action reports performed since the last inspection (No licensee event reports or special reports were submitted during the inspection period.)

The inspector completed 9 of the required 10 samples. The inspector was unable to observe the collection and preparation of environmental samples because no samples were collected during the on-site inspection period.

# b. <u>Findings</u>

<u>Introduction</u>. A Green self-revealing, non-cited violation was reviewed for the failure to follow radiation protection procedures as required by Technical Specification 5.5.1. related to the release or radioactive material from the protected area.

<u>Description</u>. The team determined that on November 6, 2003, the licensee discovered one magenta-painted hose in a 55-gallon drum which had been stored in the Mesa area (outside the protected) since January 1998. The licensee surveyed the hose and determined that it contained a total fixed activity of 40 nanocuries. In 1998 the hose had been used to replace charcoal in the Units 2/3 Fuel Handling Building Post Accident Cleanup Units. Four 55-gallon drums containing equipment related to this job were released to the Mesa for storage on January 15, 1998. The failure to identify the contaminated hose, and the release of radioactive material from the protected area, is a violation of Technical Specifications.

Analysis. The team determined that release of a radioactive material from the licensee's protected area is a performance deficiency. The finding was greater than minor because it is associated with the cornerstone objective (material release) and it affected the associated cornerstone objective (to ensure adequate protection of public health and safety from exposure to radioactive material released into the public domain). The finding was associated with a violation of NRC requirements. This finding was evaluated as having very low safety significance using the Public Radiation Safety Significance Determination Process because the finding was a radioactive material control issue, was not a transportation issue, public exposure was not greater than 5 millirem, and there were less than 5 occurrences over the past rolling 8 quarters.

Enforcement. Technical Specification 5.5.1 requires that written procedures shall be established and implemented covering those procedures recommended in Regulatory Guide (RG) 1.33, Appendix A, for the control of radioactivity and limiting personnel exposure. Procedure SO123-VII-20.9.2, "Material Release Surveys," Revision 4, Section 6.1.1, required a release criterion of no detectable licensed activity above background. However, the licensee failed to identify and prevent the release of radioactive material from the protected area. Because this failure is of very low safety significance and has been entered into the licensee's corrective action system as Action Request 031100334, this violation is being treated as an NCV, consistent with Section VI.A of the NRC Enforcement Policy: NCV 05000361/0406-02; 05000362/0406-02, Release of a Contaminated Hose from the Protected Area.

## 4. OTHER ACTIVITIES

## 4OA2 Problem Identification and Resolution

## Annual Sample Review

#### a. Inspection Scope

The team evaluated the effectiveness of the licensee's problem identification and resolution process with respect to the following inspection areas:

- Radiation Monitoring Instrumentation (Section 2OS3)
- Radioactive Gaseous and Liquid Effluent Treatment and Monitoring Systems (Section 2PS1)
- Radioactive Material Processing and Transportation (Section 2PS2)
- Radiological Environmental Monitoring Program and Radioactive Material Control Program (Section 2PS3)

## b. Findings and Observations

No findings of significance were identified.

## 4OA6 Management Meetings

#### Exit Meeting Summary

On January 30, 2004, the team presented the inspection results to Messrs. J. Wambold and D. Nunn, and other members of your staff who acknowledged the findings. One proprietary document was reviewed by the team during the inspection but was not copied or retained.

On February 19, 2004, the team telephonically presented the final inspection results to Mr. M. McBrearty who acknowledged the findings.

#### 4OA7 Licensee-Identified Violations

The following finding of very low safety significance was identified by the licensee as a violation of NRC requirements which meet the criteria of Section VI of the NRC Enforcement Policy, NUREG-1600, for being dispositioned as NCV.

\* 10 CFR Part 20.2006(d) states in part, that each licensee involved in the disposal of waste, shall comply with the requirements specified in Section III of Appendix G to 10 CFR Part 20. 10 CFR Part 20 Appendix G, Section III, E, requires, in part, that any shipment for which acknowledgment is not received within 20 days after transfer be investigated by the shipper, and be traced and reported to the NRC

within two weeks of completion of the investigation. In May 2003 during a QA audit the licensee identified 28 shipments of radioactive waste that had not been acknowledged by the burial ground within 20 days after transfer. The late shipments were not investigated, traced or reported to the NRC. The finding was determined to be of very low safety significance because radiation limits were not exceeded, there was no breach of package during transit, there was no Certificate of Compliance finding, the finding was not a low level burial ground nonconformance, and the finding was not a failure to make notifications or provide emergency information.

#### **ATTACHMENT**

#### SUPPLEMENTAL INFORMATION

## **KEY POINTS OF CONTACT**

## Licensee personnel

- J. Chang, Engineer, Maintenance
- D. Dick, Effluent Supervisor, Chemistry
- P. Elliott, Supervisor, Health Physics
- M. Farmer, Supervisor, Health Physics
- A. Gray, Supervisor, Health Physics
- F. Liu, Engineer, Maintenance Engineering
- J. Madigan, Manager, Health Physics
- M. McBrearty, Engineer, Nuclear Regulatory Affairs
- D. McBride, Supervisor, Maintenance
- D. Nunn, Vice President, Engineering & Technical Services
- A. Scherer, Manager, Nuclear Regulatory Affairs
- S. Vaughan, Supervisor, Health Physics
- R. Waldo, Station Manager
- J. Wambold, Vice President, Nuclear Generation

## NRC

- C. Osterholtz, Senior Resident Inspector
- M. Sitek, Resident Inspector

## LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

#### Opened

NONE

## Opened and Closed During this Inspection

05000362/2004006-01 NCV Failure to have all the required effluent monitor channels

operable or implement auxiliary sampling equipment.

05000361,362/2004006-02 NCV Release of a contaminated hose from the protected area.

Previous Items Closed

NONE

Previous Items Discussed

**NONE** 

#### LIST OF DOCUMENTS REVIEWED

## I. Inspection Procedure 71121.03

## **Area Radiation Monitors**

2/3-7838, 2/3-7841, 2/3-7844, 3-7850, 2/3-7852, and 2/3-7899,

#### **Post-Accident Monitors**

2(3)-7804, 2(3)-7807, 2(3)-7819, 2(3)-7822, 2(3)-7823, 2/3-7824, and 2/3-7825

## Calibration and Performance Checks of Portable Instruments

Gamma-60 Portal Monitors: 1A, 5, and 6

IPM8M (Contamination Monitors): 304, 306, 308, 332, 334, and 345

Portable Instruments: Teletector 6112B-22610, Ion Chamber RO2-3416, Bicron

Mirco-R-B549N, and ASP-1, 0979

## Calibration Packages of Material Release Insturments

SAM9A-145, Dated 11/19/03

SAM9A-188. Dated 04/17/03

SAM9A-189, Dated 12/22/03

SAM9A-281, Dated 12/27/02

SAM9A-282, Dated 01/08/03

Eberline ASP-1-1839, 09/29/03

## Audits

SCES-011-03 Calibration Program

SCES-012-03 Health Physics/Radiation Protection Program

## <u>Corrective Action Reports (Action Requests)</u>

020101490, 020300838, 020500049, 020601286, 020701289, 020800335, 020800599, 020800940, 020900502, 020901259, 020901369, 021001011, 021001607, 030101255, 030201687, 030801020, 031100485, and 031101279

# Procedures

SO123-VII-20.14.1	Health Physics Instrumentation Program, Revision 3
SO123-VII-20.14.2.1	Operation of Common Portable Survey Instruments, Revision 3
SO123-VII-20.14.2.2	Performance Testing of Common Portable Survey Instruments,
	Revision 3
SO123-VII-20.14.5.1	Calibration of Common Portable Dose Rate Instruments, Revision 2
SO123-VII-20.14.5.2	Calibration of the Eberline ASP-1 Meter, Revision 2
SO123-VII-20.14.6.2	Nuclear Enterprises SAM9 and SAM10 Small Article Monitors and
	WAM-4 Bag Monitor, Revision 3
SO123-VII-20.14.6.4	NNC Gamma-60 Portal Monitor, Revision 1
SO123-VII-20.20.7.2	Canberra Accuscan Whole Body Counter, Revision 4

## II. <u>Inspection Procedure 71122.01</u>

# Calibration Packages

01101765000-3RY7865A1 01101782000-2RE7828C 01101794000-3RE7870A1 02061382000-2/3RE7813 02101649000-2/3RE7808G

#### Procedures

Fuel Handling Building Post-Accident Handling Cleanup Filter System Operability Test, Revision 7
Control Room Emergency Air Cleanup System Operation and
Operability Test Surveillance, Revision 6 and 7
Generating Effluent Release Permits Using the Vax Computer,
Revision 4
Units 2/3 Effluents Sampling and Analysis, Revision 25
Units 2/3 Liquid Effluent Sample Collection, Revision 15
Radiological Respiratory Protection Program, Revision 4
National Draeger Self-Contained Breathing Apparatus (SCBA),
Revision 4
Bauer UNICUS 17 High Pressure Compressor, Revision 2
Supplied Breathing Air Grade D and Grade L Quality Check,
Revision 3
Respirator Inventory, Control, and Issue, Revision 5

## **Action Requests**

Radioactive Effluents - 020700297, 030300823, 030600245, 030701613, 030901312 Air Cleaning - 020400917, 020800969, 021101348, 030401274 SCBA - 011200873, 030201834, 030700485

## Self-Assessment and Quality Verification

Nupic Joint Audit/Survey of Nucon International (08/11/03 - 08/15/03) Audit No. SCES-007-02, Environmental ODCM (09/27/02)

## Air Cleaning System Surveillance Tests

Control Room Complex Emergency Air Cleanup System (2/3 418 and 2/3 419)
Unit 2 Fuel Handling Building Post-Accident Cleanup Units (2ME 370 and 2ME 371)
Unit 3 Fuel Handling Building Post-Accident Cleanup Units (3ME 370 and 3ME 371)

## <u>Liquid Effluent Release Permits</u>

010 2/3 T-057 035 2/3 T-076

## Gaseous Effluent Release Permits

015 Unit 3 Containment purge

045 Unit 2/3 T-088

## Miscellaneous

2001 Annual Radioactive Effluent Release Report 2002 Annual Radioactive Effluent Release Report Unit 2/3 Offsite Dose Calculation Manual, Revision 38

# III. Inspection Procedure 71122.02

## Radioactive Waste Shipments

02-1032, 02-2002, 03-1335, 03-2303, and 03-1008

#### Licenses

SNM-42, SNM-770, R-73008-H94, 73016-I04, 0023-07, and 5933-36

## **Action Requests**

 $020100501,\,020100710,\,020601495,\,030102049,\,030301546,\,030500040,\,030500591,\,$  and 030800658

## **Division Self Assessments**

Second Quarter 2002, Third Quarter 2002, Fourth Quarter 2002, First Quarter 2003, Second Quarter 2003, and Third Quarter 2003

## Audits

NORAD Audit Report SCES-003-02, Radioactive Material Control

#### Procedures

Solid Radioactive Waste Sampling for Classification and Typification,
Revision 18
Shipment of Radioactive Material, Revision 18
Receipt of Radioactive Material, Revision 11
Radwaste Process Control Program, Revision 7
Transfer of Waste/Radioactive Material to a Processing Container
Dewatering System Operation, Revision11

#### Miscellaneous

2001 Annual Radioactive Effluent Release Report 2002 Annual Radioactive Effluent Release Report SONGS Final Safety Analysis Report

# IV. Inspection Procedure 71122.03

#### **Procedures**

<u>lui es</u>	
SO123-IX-1.1	Radiological Environmental Monitoring Program Training and
	Personnel Qualification Guideline, Revision 7
SO123-IX-1.10	Review, Analysis, and Reporting of Radiological Environmental
	Monitoring Program Data, Revision TCN 4-1
SO123-VII-20.9.2	Material Release Surveys, Revision 4-1
	SO123-IX-1.1 SO123-IX-1.10 SO123-VII-20.9.2

SO123-II-8.12 Surveillance Requirement 10 and 40 Meter Meteorological Instrumentation Channel Calibration, Revision TCN-8-3
SO123-V-8 Meteorological Data Acquisition System, Revision 0
SO23-3-3.21.1 Surveillance Operating Instruction, Attachment 1, Revision TCN 20-2

## Self-Assessments and Quality Verification

Audit SLES-007-02, Environmental ODCM

Audit TES-01-03, Supplier Audit Report - Teledyne Brown Engineering Environmental Services

NUPIC Audit 17944, Duke Engineering and Services, Inc., May 2001

Framatone ANP DE&S Laboratory Analystical Services, Semi-Annual Quality Assurance Status Report, January through July 2002, July through December 2002, January through June 2003

Audit SCE-MBE-1-03, MBC Applied Environmental Sciences, 2001
Audit SCE-MBC-1-02, MBC Applied Environmental Sciences, 2002
Site Support Services Division Performance Report, Third Quarter 2003
Site Support Services Division Performance Report, Fourth Quarter 2003
Directed Assessment of the Material Release Program (AR 011200873-12)
Directed Assessment for Environmental Compliance (AR 021101286)
Heath Physics Division Quality Assessments (First Quarter 2002 through Third Quarter 2003)

NORAD Audit SCES-003-02, Radioactive Material Control, May 2002

#### Environmental air sampling stations

1, 7, 9, 10, 11, 12, 13, 14, and 15 located in Emergency Planning Sector's NW, NW, ESE, WNW, NNW, NW, E, NNW, and SE, respectively

## Thermoluminescent dosimetry stations

10, 12, 14, 40, 59, 64, 65, 66, and 67, located in Emergency Planning Sectors E, WNW, SE, NNW, WNW, ENE, E, ESE, and NW, respectively

# Action Requests

Radiological Environmental Monitoring Program - 020701298, 021100412, 021000593, and 030500646

Release of Radioactive Material - 010901163, 020801154, 030102341, 030600533, 030900637, 031100334, and 040101950

Meteorological Monitoring - 020101331, 020300003, 030800334,030901254, 040100799, 040101459, 040101489, and 040101257

#### Calibration Data

## Radiological Environmental Monitoring Program

Digital Gas Meter Flow Totalizer, Serial Number 5012, Dated June 17, 2003
Digital Gas Meter Flow Totalizer, Serial Number 5013, Dated August 2, 2002
Digital Gas Meter Flow Totalizer, Serial Number 5014, Dated September 18, 2003
Digital Gas Meter Flow Totalizer, Serial Number 5015, Dated July 26, 2002
Digital Gas Meter Flow Totalizer, Serial Number 5016, Dated August 19, 2003

Digital Gas Meter Flow Totalizer, Serial Number 5017, Dated July 1, 2002

Digital Gas Meter Flow Totalizer, Serial Number 5018, Dated November 26, 203

Digital Gas Meter Flow Totalizer, Serial Number 5019, Dated July 2002

Digital Gas Meter Flow Totalizer, Serial Number 5020, Dated October 30, 2003

Digital Gas Meter Flow Totalizer, Serial Number 5142, Dated November 25, 2002

## Meteorological Monitoring

Repetitive Maintenance Number 50000812000, Primary Met Tower Instrumentation Semi-Annual Calibration, July 2003

Repetitive Maintenance Number 50000812001, Backup Met Tower Instrumentation Semi-Annual Calibration, July 2003

## **Miscellaneous**

2001 and 2002 Annual Radioactive Effluent Release Report 2002 and 2003 Land Use Census Results