July 24, 2003

Mr. John L. Skolds, President Exelon Nuclear Exelon Generation Company, LLC Quad Cities Nuclear Power Station 4300 Winfield Road Warrenville, IL 60555

### SUBJECT: QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2 NRC EVALUATIONS OF CHANGES, EXPERIMENTS, OR TESTS AND PERMANENT PLANT MODIFICATIONS INSPECTION REPORT 50-254/03-10(DRS); 50-265/03-10(DRS)

Dear Mr. Skolds:

On July 11, 2003, the U.S. Nuclear Regulatory Commission (NRC) completed a routine baseline inspection at your Quad Cities Nuclear Power Station, Units 1 and 2. The enclosed report documents the inspection findings, which were discussed on July 11, 2003, with Mr. T. Tulon and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and to compliance with the Commission's rules and regulations and with the conditions of your license. Specifically, this inspection focused on the baseline biennial inspections for evaluations of changes, tests, or experiments (10 CFR 50.59) and permanent plant modifications. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

Based on the results of this inspection, no findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be available electronically for public inspection in the NRC Public Document Room or from the Publically Available Records (PARS) component

of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <u>http://www.nrc.gov/reading-rm/adams.html</u> (the Public Electronic Reading Room).

Sincerely,

#### /RA/

Julio F. Lara, Chief Electrical Engineering Branch Division of Reactor Safety

Docket Nos. 50-254; 50-265 License Nos. DPR-29; DPR-30

- Enclosure: Inspection Report 50-254/03-10(DRS); 50-265/03-10(DRS) w/Attachment: Supplemental Information
- Site Vice President Quad Cities Nuclear Power Station cc w/encl: Quad Cities Nuclear Power Station Plant Manager Regulatory Assurance Manager - Quad Cities **Chief Operating Officer** Senior Vice President - Nuclear Services Senior Vice President - Mid-West Regional **Operating Group** Vice President - Mid-West Operations Support Vice President - Licensing and Regulatory Affairs **Director Licensing - Mid-West Regional** Operating Group Manager Licensing - Dresden and Quad Cities Senior Counsel, Nuclear, Mid-West Regional **Operating Group Document Control Desk - Licensing** Vice President - Law and Regulatory Affairs Mid American Energy Company M. Aguilar, Assistant Attorney General Illinois Department of Nuclear Safety State Liaison Officer. State of Illinois State Liaison Officer, State of Iowa Chairman, Illinois Commerce Commission D. Tubbs, Manager of Nuclear MidAmerican Energy Company

of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <u>http://www.nrc.gov/reading-rm/adams.html</u> (the Public Electronic Reading Room).

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J. Skolds

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## U.S. NUCLEAR REGULATORY COMMISSION

## **REGION III**

Docket Nos:	50-254; 50-265
License Nos:	DPR-29; DPR-30
Report No:	50-254/03-10(DRS); 50-265/03-10(DRS)
Licensee:	Exelon Generation Company, LLC
Facility:	Quad Cities Nuclear Power Station, Units 1 and 2
Location:	22710 206th Avenue North Cordova, IL 61242-9778
Dates:	July 7, 2003 through July 11, 2003
Inspectors:	<ul> <li>G. Hausman, Senior Reactor Inspector, Lead</li> <li>H. Walker, Senior Reactor Inspector</li> <li>S. Spiegelman, Mechanical Engineering Contractor</li> </ul>
Approved by:	Julio F. Lara, Chief Electrical Engineering Branch Division of Reactor Safety

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## SUMMARY OF FINDINGS

IR 05000254/2003-010(DRS), 05000265/2003-010(DRS); Exelon Generation Company, LLC; 07/07/03 - 07/11/03; Quad Cities Nuclear Power Station, Units 1 and 2; Routine Baseline Inspection Report.

This report covers a five day period of announced baseline inspection on evaluations of changes, tests, or experiments and permanent plant modifications. The inspection was conducted by Region III inspectors and an engineering contractor. No findings of significance were identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using Inspection Manual Chapter (IMC) 0609, "Significance Determination Process" (SDP). Findings for which the SDP does not apply may be "Green" or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

## A. Inspector-Identified and Self-Revealed Findings

## **Cornerstone: Initiating Events**

No findings of significance were identified.

## **Cornerstone: Mitigating Systems**

No findings of significance were identified.

## **Cornerstone: Barrier Integrity**

No findings of significance were identified.

#### B. Licensee-Identified Violations

No findings of significance were identified.

## **REPORT DETAILS**

## **Summary of Plant Status**

Unit 1 operated at or near full power throughout the inspection period.

Unit 2 operated at or near 85 percent power throughout the inspection period. Unit 2 was limited to 85 percent power pending a presentation to the NRC concerning the root cause(s) for the steam dryer failure.

## 1. REACTOR SAFETY

### Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity

- 1R02 Evaluations of Changes, Tests, or Experiments (71111.02)
- .1 Review of 50.59 Evaluations and Screenings
- a. Inspection Scope

The inspectors reviewed six evaluations performed pursuant to 10 CFR 50.59. The evaluations related to permanent plant modifications, setpoint changes, procedure changes, conditions adverse to quality, and changes to the updated final safety analysis report. The inspectors confirmed that the evaluations were thorough and that prior NRC approval was obtained as appropriate. The inspectors also reviewed sixteen screenings where the licensee had determined that a 10 CFR 50.59 evaluation was not necessary. In regard to the changes reviewed where no 10 CFR 50.59 evaluation was performed, the inspectors verified that the changes did not meet the threshold to require a 10 CFR 50.59 evaluation. These evaluations and screenings were chosen based on risk significance of samples from the different cornerstones.

b. Findings

No findings of significance were identified.

#### 1R17 <u>Permanent Plant Modifications</u> (71111.17)

- .1 <u>Review of Recent Modifications</u>
- a. Inspection Scope

The inspectors reviewed sixteen permanent plant modifications that were installed in the last two years. The modifications were chosen based upon their affecting systems that had high probabilistic risk analysis (PRA) significance in the licensee's Individual Plant Evaluation (IPE) or high maintenance rule safety significance. The inspectors reviewed the modifications to verify that the completed design changes were in accordance with the specified design requirements and the licensing bases and to confirm that the

changes did not affect any systems' safety function. Design and post-modification testing aspects were verified to ensure the functionality of the modification, its associated system, and any support systems. The inspectors also verified that the modifications performed did not place the plant in an increased risk configuration.

b. Findings

No findings of significance were identified.

### 4. OTHER ACTIVITIES (OA)

- 4OA2 Identification and Resolution of Problems (71152)
- .1 Routine Review of Condition Reports
- a. Inspection Scope

The inspectors reviewed a selected sample of condition reports associated with Quad Cities' permanent plant modifications and concerning 10 CFR 50.59 evaluations and screenings. The inspectors reviewed these issues to verify an appropriate threshold for identifying issues and to evaluate the effectiveness of corrective actions. In addition, condition reports written on issues identified during the inspection were reviewed to verify adequate problem identification and incorporation of the problem into the corrective action system. The specific corrective action documents that were sampled and reviewed by the team are listed in the attachment to this report.

b. Findings

No findings of significance were identified.

#### 40A6 Meetings

.1 Exit Meeting

The inspectors presented the inspection results to Mr. T. Tulon and other members of licensee management at the conclusion of the inspection on July 11, 2003. The inspectors asked the licensee whether any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

.2 Interim Exit Meetings

No interim exits were conducted.

## ATTACHMENT: SUPPLEMENTAL INFORMATION

### SUPPLEMENTAL INFORMATION

## **KEY POINTS OF CONTACT**

#### **Licensee**

K. Adlon, Design Engineering

C. Alguire, Design Engineering Supervisor

W. Beck, Regulatory Assurance Manager

P. Behrens, Nuclear Oversight

G. Boerschig, Work Management Director

D. Boyles, Operations

S. Davis, System Engineering

T. Fuhs, Regulatory Assurance

R. Gideon, Engineering Director

R. Heyn, Design Engineering

D. Hieggelke, Nuclear Oversight Manager

B. Porter, Design Engineering Manager

T. Scott, Shift Operations Superintendent

T. Tulon, Site Vice-President

M. Wagner, Regulatory Assurance

T. Wojcik, Engineering Programs Supervisor

D. Wolf, Design Engineering

Nuclear Regulatory Commission

R. Telson, Acting Senior Resident Inspector

M. Kurth, Resident Inspector

# LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

<u>Opened</u>

None.

# <u>Closed</u>

None.

## **Discussed**

None.

## LIST OF DOCUMENTS REVIEWED

The following is a list of documents reviewed during the inspection. Inclusion on this list does not imply that the NRC inspectors reviewed the documents in their entirety but rather that selected sections of portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document on this list does not imply NRC acceptance of the document or any part of it, unless this is stated in the body of the inspection report.

## CALCULATIONS

Document Number	Title or Description	<b>Revision or Date</b>
QC-1ET-E-001	Battery Room Minimum Airflow Requirements	April 7, 1994
QDC-1000-M-1195	RHRSW Pump Room Cooler Supply Line Restricting Orifice Modification	May 17, 2002

## CONDITION REPORTS (CRs) GENERATED DURING THE INSPECTION

<b>Document Number</b>	Title or Description	<b>Revision or Date</b>
00166641	Incorporate Hydraulic Transient Precautions in	July 8, 2003
	QCOP 3900-01	
00166821	Unit Number Not Identified on 50.59 Forms	July 9, 2003
00167098	Evaluate UFSAR Instrument Air to N <sub>2</sub> Setpoint	July 10, 2003
	Statement	
00167054	Inadequate 50.59 for EDG TD7 Timer Modification	July 11, 2003
00167100	Inadequate 50.59 for Recirculation Pump Seal	July 11, 2003
	Injection Modification and Inadequate Procedural	
	Guidance for Seal Injection Pump Isolation	

## CONDITION REPORTS (CRs) GENERATED PRIOR TO INSPECTION

<b>Document Number</b>	Title or Description	<b>Revision or Date</b>
AR 00123341	Incorrect Wiring of TD7 (Re-Crank) Repeat Cycle Timer	September 18, 2002
AR 00130499	Leaking Weld on RBCCW Line 1-3713C-1-1/2"-L to	November 6, 2002
AR 00130499	1A Recirc Pump	
Q1999-03024	U-1A(1B) SBO Starting Air Compressors Oil	September 14, 1999
	Analysis Results	
Q1999-03034	Potential Operator Work-Around	September 14, 1999
Q2001-01312	U-2 EDG Day Tank Level Drop	May 1, 2001
Q2000-01435	1/2 DG Vent Backup N <sub>2</sub> Failed to Meet	April 7, 2000
	QCOS 6600-15 Acceptance Criteria	
Q2000-03280	PS for $N_2$ Backup to U1 Dampers OOT	September 15, 2000
Q2001-01299	Failure of PS 2-5790-01	April 30, 2001
Q2001-02857	Corrective Actions of AT26887 for EDG HVAC Swap Were Incomplete	September 14, 2001

## DRAWINGS

DIAMINOS		
<b>Document Number</b>	Title or Description	<b>Revision or Date</b>
4E-1350A, Sheet 1	Schematic Diagram Engine Control & Generator	AN
	Excitation Standby Diesel Generator 1	
4E-1350A, Sheet 2	Schematic Diagram Engine Control & Generator	AL
	Excitation Standby Diesel Generator 1	
M-15, Sheet 1	Diagram of Reactor Feed Piping	BF & BJ
M-36	Diagram of Core Spray Piping	BA
M-37	Diagram of RHR Service Water Piping	AV
M-46, Sheet 1	Diagram of HPCI Piping	BR & BT
M-70	Diagram of Safe Shutdown Make-Up Pump System	U & V
M-2024, Sheet 2	Turbine Building Ventilation Details - Battery Rooms	С
M-2025	HVAC Plan and Details	A
M-2029	HVAC Duct Layout IRSF	С
M-2030	HVAC Sections and Details IRSF	С

## **EVALUATIONS**

<b>Document Number</b>	Title or Description	<b>Revision or Date</b>
QC-E-2001-014	Provide 1B Service Water Pump With an Autostart	July 3, 2001
	Feature (DCP 9900183)	
QC-E-2001-021	Reactor Recirculation Pump Seal Purge Line	October 18, 2001
	Modification EC331303 (U1); EC331304 (U2);	
	UFSAR-R7-01-18 & -19	
QC-E-2001-024	Revise Small Instrument Line Break Analysis	May 3, 2002
	(UFSAR Change UFSAR-01-R7-014; 6.2.3, 15.6.2)	
QC-E-2002-002	Unit 2 Core Reload Design Q2C17	March 1, 2002
QC-E-2002-003	Powerplex CMSS Upgrade EC336060 & EC336165	May 21, 2002
QC-E-2003-002	Malfunction of the EHC Pressure Control System	May 2, 2003
	(UFSAR and Procedure Change:	
	UFSAR-03-R8-022, QCOA 5650-01)	

# MODIFICATIONS

<b>Document Number</b>	Title or Description	<b>Revision or Date</b>
EC24037	Install Re-Crank System and Replace TD1 on	September 6, 2000
	1/2 EDG (contained Screening SE-00-039; dated	
	06/05/2000)	
EC24038	Install Re-Crank System and Replace TD1 on Unit 1	October 6, 2000
	EDG (contained Screening SE-00-039; dated	
	06/05/2000)	
EC24216	Deflector Plate Installation in the Unit 1 Battery	March 7, 2001
	Room HVAC Heating Ventilation Duct (contained	
	Screening SS-F-01-0058; dated 03/07/2001)	
EC24287	Unit 1 EDG Time Delay Relay Addition (contained	February 6, 2003
	Screening QC-S-2003-0016; dated 02/05/2003)	

# MODIFICATIONS

<b>Document Number</b>	Title or Description	Revision or Date
EC24288	Moisture Elimination for the 1A SBO Diesel Air	August 31, 2001
	Compressor (contained Screening	
	QC-S-2001-0298; dated 08/10/2001)	
EC24289	Moisture Elimination from SBO Air Compressor	November 20, 2001
	(contained Screening QC-S-2001-0475; dated	
E004000	11/26/2001)	1 1 0 1 0 0 0 1
EC24290	Moisture Elimination for the 2A SBO Diesel Air	August 31, 2001
	Compressor (contained Screening	
EC24291	QC-S-2001-0298; dated 08/10/2001)	August 10, 2001
EC24291	Moisture Elimination for the SBO Diesel Air	August 10, 2001
	Compressor (contained Screening QC-S-2001-0298; dated 08/10/2001)	
EC24435	RHRSW Cubicle Cooler 2-5745-C Flow Indicator	July 26, 2002
L024433	Addition And Restricting Orifice Replacement	July 20, 2002
	(contained Screening QC-S-2002-0013; dated	
	05/17/2002)	
EC24461	Feedwater Level Control System Digital Upgrade	September 18, 2001
	(contained Evaluation QC-E-2001-020; dated	
	09/11/2001)	
EC24578	Replace SO 2-5201 with Manual Valve on U2 EDG	August 22, 2001
	(contained Evaluation QC-E-2001-019; dated	
	08/21/2001)	
EC333070	EDG Ventilation Back-Up N <sub>2</sub> PS Setpoint Change	April 19, 2002
	(contained Screening QC-S-2001-0487; dated	
	12/14/2001)	
EC332259	Removal of SO 1-5201 and Installation of Manual	November 10, 2001
	Valve 1-5299-181 (contained Evaluation	
<b>F</b> 0000000	QC-E-2001-019; dated 08/21/2001)	N 1 10 0001
EC332260	Removal of SO 0-5201 from Unit 0 EDG Day Tank	November 13, 2001
	Line (contained Evaluation QC-E-2001-019; dated	
EC337693	08/21/2001) Quad Cities Unit 1 Iso-Phase Bus Duct Cooling	Sontombor 26, 2002
EC337093	Modification (contained Screening QC-S-2002-0349;	September 26, 2002
	dated 10/04/2002)	
EC339723	Repair RBCCW Line 1-3713C-1" at 1A Recirc Pump	November 8, 2002
20000720	Seal Cooler (contained Screening QC-S-2002-0378;	
	dated 11/08/2002)	
EC340578	Install Patch Plate Under RHRSW Pipe Support	January 16, 2003
•	(contained Screening QC-S-2003-0006; dated	, -,
	01/15/2003)	

## PROCEDURES

<b>Document Number</b>	Title or Description	<b>Revision or Date</b>
CC-AA-10	Configuration Control Process Description	2
CC-AA-102	Design Input and Configuration Change Impact	5
	Screening	
CC-AA-103	Configuration Change Control	4
CC-AA-104	Document Change Requests	3
CC-AA-107	Configuration Change Acceptance Testing Criteria	3
CC-AA-110	Field Change Requests	2
CC-AA-112	Temporary Configuration Changes	6
LS-AA-104	Exelon 50.59 Review Process	3
LS-AA-104-1000	Exelon 50.59 Resource Manual	1
LS-AA-104-1001	50.59 Review Coversheet Form	1
LS-AA-104-1002	50.59 Applicability Review Form	1
LS-AA-104-1003	50.59 Screening Form	1
LS-AA-104-1004	50.59 Evaluation Form	2
LS-AA-104-1006	Exelon 50.59 Training and Qualification	2
LS-AA-125	CAP Procedure	4
_S-AA-125-1001	Root Cause Analysis Manual	3
LS-AA-125-1002	Common Cause Analysis Manual	2
LS-AA-125-1003	Apparent Cause Evaluation Manual	2
LS-AA-125-1004	Effectiveness Review Manual	1
LS-AA-125-1005	Coding and Trending Manual	3
LS-AA-125-1006	CAP Process Expectations Manual	3
LS-AA-127	Passport Action Tracking Management Procedure	3
QCAN 901(2)-8 G-7	Diesel Fuel Oil Day Tk Unit 1(2) Level High or Low	7
QCOA 0202-04	Reactor Recirc Pump Trip - Single Pump	19
QCOA 0202-06	Recirculation Pump Seal Failure	16
QCOA 3900-01	Service Water System Failure	9
QCOP 0202-35	Reactor Recirculation Pump Seal Injection System Operation	6
QCOP 1400-06	Injecting Water into the RPV Using the ECCS Keep Fill Pump	6
QCOP 2050-10	IRSF General Operation Procedure	8
QCOP 3900-01	Service Water System Operation	8
QCOP 4100-16	Manually Filling the Diesel Fire Pump Day Tank	5
QCOS 6600-01	DG 1(2) Preparation for Standby Operation	25
QCOS 6600-05	DG Fuel Oil Transfer Pump Flow Rate Test	17
QCOS 6600-09	Filling of DG Fuel Oil Tanks With Installed System Unavailable	6
QOM 2-6600-01	U2 DG Valve Checklist	17

## REFERENCES

REFERENCES		
<b>Document Number</b>	Title or Description	<b>Revision or Date</b>
	Quad Cities Organizational Chart	June 6, 2003
	Hit Team Charter Raw Water Inspection Program	
	Quad Cities Station UFSAR Report	January 2003
	Quad Cities Station Technical Specifications	Amendment 216/210
139325-01	Quad Cities Common Cause Analysis and Plan for	March 17, 2003
	RHRSW Pin Hole Leaks	
AR 75698-04	Engineering HP Near-Miss Clock Reset No. 15	September 19, 2001
AT 00164769	Raw Water Corrosion Monitoring Plan	May 13, 2003
CRN 01-03	Addition of the IRSF to the FHA	July 13, 2001
EC330747	Engineering Change Evaluation for Determine	November 15, 2001
	Cause of Repeated OOTs on PS 1-5790-1	
NUREG-0544	NRC Collection of Abbreviations	January 4, 1900
SVP-03-002	10 CFR 50.59 Summary Report	January 3, 2003

## SCREENINGS

<b>Document Number</b>	Title or Description	<b>Revision or Date</b>
QC-S-2001-0262	IRSF Fire Hazards Analysis (CRN 01-03)	July 26, 2001
QC-S-2001-0314	Provide High Speed Ground Fault Protection to the	August 13, 2001
	UAT's (4kV) X and Y Windings and 4kV Leads to	
	the Auxiliary Power Buses (DCP 9900192/9900193)	
QC-S-2001-0315	Battery Equalizing Charges (QOP 6900-11)	August 13, 2001
QC-S-2001-0444	Reroute of Unit 1 SSMP Discharge Piping (EC331679)	October 29, 2001
QC-S-2001-0478	RHRSW System Operation, RHRSW Operation Using Loop A Cross-Tie Header, RHRSW Operation Using Loop B Cross-Tie Header, Flushing Service Water Side of RHR Heat Exchanger or Discharge Valves (QCOP 1000-04, QCOP 1000-15, QCOP 1000-20, QCOP 1000-27)	November 28, 2001
QC-S-2001-0487	EDG Ventilation Back-up N <sub>2</sub> PS Setpoint Change (Setpoint Change Design Package/ EC333070/3330171/333072)	December 14, 2001
QC-S-2002-0013	RHRSW Cubicle Cooler Flow Indicator Addition And Restricting Orifice Replacement (ECs 23424, 23425, 24427, 24428, 24429, 24435, 24436, 24438)	May 17, 2002
QC-S-2002-0044	RPV Control, Primary Containment Control, Secondary Containment Control, Radioactive Release Control, RPV Blowdown, Steam Cooling, RPV Flooding, and RPV Control (ATWS) (QGA 100, 200, 300, 400, 500-1, 500-2, %00-4, & 101)	January 31, 2002
QC-S-2002-0186	Install Temporary Plug in the Watertight Door in the 1D RHRSW Vault (TCCP EC336935)	May 2, 2002
QC-S-2002-0214	Service Air Header Isolation Valve to U2 SBLC Sparge Air (EC337242)	May 30, 2002

Attachment

## SCREENINGS

Title or Description	<b>Revision or Date</b>
CS Discharge Header High/Low Pressure, CS	June 25, 2002
0	
	August 27, 2002
Keep Fill Pump (QCOP 1400-06)	
	September 24, 2002
Timers (DCR/EC338928	
Revise M-71 Sheet 1 and Passport to Show	March 27, 2003
Abandoned Instrument Air Valves in the Unit 2	
<b>0</b> 1 0	April 19, 2003
Sequence (QCOP 1000-05 (TIC))	
Installation of a Pipe Cap on the Discharge Pipe and	May 8, 2003
(TCC EC342689)	
	CS Discharge Header High/Low Pressure, CS Header A Valve Leakage, CS Header B Valve Leakage (QCAN 901(2)-3 B-16, QCAN 901(2)-3 G-5, QCAN 901(2)-3 H-5) Injecting Water Into The RPV Using The ECCS Keep Fill Pump (QCOP 1400-06) Revise EDG Start Circuit Schematics to Show Proper Configuration for the TD-7 Repeat Cycle Timers (DCR/EC338928 Revise M-71 Sheet 1 and Passport to Show Abandoned Instrument Air Valves in the Unit 2 RHRSW Vaults (DCR EC340328) Revision to Shutdown Cooling Pump Starting Sequence (QCOP 1000-05 (TIC))

## VALIDATIONS

<b>Document Number</b>	<u>Title or Description</u>	<b>Revision or Date</b>
QC-V-2001-0016	IPEEE ASM Battery Equalizing Charges	August 13, 2001
	(QCOP 6900-09)	
QC-V-2001-0040	Replace EDGs 1 and 2 Speed Sensing Panels with	December 14, 2001
	Updated Design (Modifications/EC24422 and	
	EC24423)	
QC-V-2002-0021	Unit 1 Feedwater Level Control System Digital	May 14, 2002
	Upgrade (QC-S-2001-0334)	
QC-V-2002-0022	Unit 1 Feedwater Level Control System Digital	May 14, 2002
	Upgrade (QC-E-2002-0020)	

# LIST OF ACRONYMS USED

ADAMS AR ASM ATWS CAP CFR CMSS CR CS DCP DCR DG DPR DRS EC ECCS EDG EHC FHA gov HPCI html http HVAC IMC IPE IPEEE IR IRSF LLC NRC NRR NUREG OA OOT PARS PRA PS	Agency-Wide Document Access and Management System Action Request Air Start Motors Anticipated Transient Without Scram Corrective Action Program Code of Federal Regulations Core Monitoring Software System Condition Report Core Spray Design Change Package Design Change Package Design Change Request Diesel Generator Demonstration Power Reactor Division of Reactor Safety Emgineering Change Emergency Core Cooling System Emergency Diesel Generator Electro-Hydraulic Control Fire Hazards Analysis Goverment Human Performance High Pressure Coolant Injection Hypertext Markup Language Hypertext Transfer Protocol Heating, Ventilation and Air Conditioning Inspection Manual Chapter Individual Plant Examination of External Events for Severe Accident Vulnerabilities Inspection Report Individual Plant Examination of External Events for Severe Accident Vulnerabilities Inspection Report Individual Plant Examination of External Events for Severe Accident Vulnerabilities Inspection Report Individual Plant Examination of External Events for Severe Accident Vulnerabilities Inspection Report Individual Plant Examination of External Events for Severe Accident Vulnerabilities Inspection Report Individual Plant Examination of External Events for Severe Accident Vulnerabilities Inspection Report Individual Plant Examination of External Events for Severe Accident Vulnerabilities Inspection Report Individual Plant Examination of External Events for Severe Accident Vulnerabilities Inspection Report Designation Office of Nuclear Reactor Regulation NRC Technical Report Designation Other Activities Out-of-Tolerance Publically Available Records System Probabilistic Risk Analysis Pressure Switch
PARS	Publically Available Records System
PRA	Probabilistic Risk Analysis
QC	Quad Cities
RBCCW	Reactor Building Closed Cooling Water
RHR	Residual Heat Removal
RHRSW	Residual Heat Removal Service Water

## LIST OF ACRONYMS USED

RPV	Reactor Pressure Vessel
SBLC	Standby Liquid Control
SBO	Station Black Out
SDP	Significance Determination Process
SSMP	Safe Shutdown Makeup Pump
UAT	Unit Auxiliary Transformer
UFSAR	Updated Final Safety Analysis Report
V	Validation or Volt
wpd	WordPerfect Document
www	World Wide Web