February 6, 2002

Mr. M. Reddemann Site Vice President Kewaunee and Point Beach Nuclear Plants Nuclear Management Company, LLC 6610 Nuclear Road Two Rivers, WI 54241

SUBJECT: POINT BEACH NUCLEAR PLANT

NRC INSPECTION REPORT 50-266/02-002(DRS); 50-301/02-002(DRS)

Dear Mr. Reddemann:

On January 18, 2002, the NRC completed an inspection at your Point Beach Nuclear Plant. The enclosed report documents the inspection findings which were discussed on January 18, 2002, with Mr. Taylor and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel. Specifically, this inspection focused on the baseline annual inspection for evaluations of changes, tests, or experiments (10 CFR 50.59) and the baseline biennial inspection for permanent plant modifications.

No findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

Sincerely,

/RA/

Ronald N. Gardner, Chief Electrical Engineering Branch Division of Reactor Safety

Docket Nos. 50-266; 50-301 License Nos. DPR-24; DPR-27

Enclosure: Inspection Report 50-266/02-002(DRS); 50-301/02-002(DRS)

See Attached Distribution

cc w/encl:

R. Grigg, President and Chief Operating Officer, WEPCo

R. Anderson, Executive Vice President and Chief Nuclear Officer

T. Webb, Licensing Manager

D. Weaver, Nuclear Asset Manager

T. Taylor, Plant Manager A. Cayia, Site Director

J. O'Neill, Jr., Shaw, Pittman,

Potts & Trowbridge

K. Duveneck, Town Chairman

Town of Two Creeks

D. Graham, Director

Bureau of Field Operations A. Bie, Chairperson, Wisconsin Public Service Commission S. Jenkins, Electric Division

Wisconsin Public Service Commission

State Liaison Officer

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See Attached Distribution

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NAME	GHausman:jb		RLanksbury	RGardner		
DATE	02/05/02		02/06/02	02/06/02		

cc w/encl:

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R. Anderson, Executive Vice President

and Chief Nuclear Officer T. Webb, Licensing Manager

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State Liaison Officer

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U. S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos: 50-266; 50-301 License Nos: DPR-24; DPR-27

Report No: 50-266/02-002(DRS); 50-301/02-002(DRS)

Licensee: Nuclear Management Company, LLC

Facility: Point Beach Nuclear Plant

Location: 6610 Nuclear Road

Two Rivers, WI 54241

Dates: January 14 through 18, 2002

Inspectors: Robert C. Daley, Reactor Inspector

George M. Hausman, Senior Reactor Inspector (Lead)

Kenneth G. O'Brien, Senior Reactor Inspector

Darrell L. Schrum, Reactor Inspector

Observer: Nicholas A. Valos, Reactor Inspector (Trainee)

Approved by: Ronald N. Gardner, Chief

Electrical Engineering Branch Division of Reactor Safety

SUMMARY OF FINDINGS

50-266/02-002(DRS); 50-301/02-002(DRS), on 01/14-18/2002, Nuclear Management Company, LLC, Point Beach Nuclear Plant. Evaluations of Changes, Tests, or Experiments and Permanent Plant Modifications Report.

The report covers a five day announced inspection. The inspection was conducted by four Region III based reactor inspectors and one observer in training. No findings of significance were identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using IMC 0609 "Significance Determination Process" (SDP). The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described at its Reactor Oversight Process website at http://www.nrc.gov/NRR/OVERSIGHT/index.html. Findings for which the SDP does not apply are indicated by "No Color" or by the severity level of the applicable violations.

A. <u>Inspector Identified Findings</u>

Cornerstones: Initiating Events, Mitigating Systems, Barrier Integrity

No findings of significance were identified.

Report Details

Summary of Plant Status

Units 1 and 2 were operated at or near 100 percent power throughout the inspection period.

1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity

1R02 Evaluations of Changes, Tests, or Experiments (71111.02)

.1 Review of 10 CFR 50.59 Evaluations and Screenings for Changes, Tests, or Experiments

a. <u>Inspection Scope</u>

The inspectors reviewed ten evaluations performed pursuant to 10 CFR 50.59. The evaluations related to permanent plant modifications, setpoint changes, procedure changes, conditions adverse to quality, and changes to the updated final safety analysis report. The inspectors reviewed the evaluations to verify that the evaluations were thorough and that prior NRC approval was obtained as appropriate. The inspectors also reviewed fifteen screenings where the licensee had determined that a 10 CFR 50.59 evaluation was not necessary. In regard to the changes reviewed where no 10 CFR 50.59 evaluation was performed, the inspectors verified that the changes did not meet the threshold to require a 10 CFR 50.59 evaluation. These evaluations and screenings were chosen based on risk significance of samples from the different cornerstones.

b. Findings

No findings of significance were identified.

1R17 Permanent Plant Modifications (71111.17)

.1 Review of Recent Permanent Plant Modifications

a. Inspection Scope

The inspectors reviewed ten permanent plant modifications that were installed in the last two years. The modifications were chosen based upon their affecting systems that had high probabilistic risk analysis (PRA) significance in the licensee's Individual Plant Evaluation (IPE) or high maintenance rule safety significance. The inspectors reviewed the modifications to verify that the completed design changes were in accordance with the specified design requirements and the licensing bases and to confirm that the changes did not affect any systems' safety function. Design and post-modification testing aspects were reviewed to ensure the functionality of the modification, its

associated system, and any support systems. The inspectors also reviewed the modifications performed to verify the modification did not place the plant in an increased risk configuration.

b. <u>Findings</u>

No findings of significance were identified.

4. OTHER ACTIVITIES (OA)

4OA2 Identification and Resolution of Problems

a. <u>Inspection Scope</u>

The inspectors reviewed a selected sample of corrective action documents associated with the licensee's permanent plant modifications and concerning 10 CFR 50.59 evaluations and screenings to verify that the licensee had an appropriate threshold for identifying issues. The inspectors evaluated the effectiveness of the corrective actions for the identified issues.

b. Findings

No findings of significance were identified.

4OA6 Meetings

Exit Meeting

The inspectors presented the inspection results to Mr. Taylor and other members of licensee management at the conclusion of the inspection on January 18, 2002. The licensee acknowledged the findings presented. No proprietary information was identified.

KEY POINTS OF CONTACT

Nuclear Management Company, LLC

- J. Anderson, Production Planning Manager
- L. Armstrong, Design Engineering Manager
- D. Black, Nuclear Safety Engineer
- D. Carter, Seismic Qualification Engineer
- A. Cayia, Site Operations Director
- D. Dobson, IST Engineer
- C. Drescher, Modification Engineer
- J. Ertman, NMC Fleet Fire Protection Eng
- F. Flentje, Licensing Specialist
- N. Hoefert, Programs Engineering Manager
- J. Hanna, System Engineer
- R. Hornak, Supervising Engine
- R. Hannemann, Supervising Engineer
- D. Hettick, Performance Improvement Manager
- K. Holt, Supervising Engineer
- V. Kaminskas, Maintenance Manager
- T. Kendall, Supervising Engineer
- A. Little, System Engineer
- J. Marean, Engineering Drafting Supervisor
- J. MacNamara, Supervising Engineer
- R. Mende, KPB Engineering Director
- D. Pederson, System Engineer
- L. Peterson, Engineering Processes Manager
- K. Peveler, Nuclear Oversight Manager
- R. Repshas, KPB Site Services Manager
- J. Roberts, MOV Engineer
- M. Rosseau, Supervising Engineer
- D. Schoon, Operations Manager
- T. Taylor, Plant Manager
- S. Thomas, Radiation Protection Manager
- T. Vandenbosch, AOP Coordinator

NRC

P. Krohn, Senior Resident Inspector

LIST OF ACRONYMS USED

ADAMS Agencywide Documents Access and Management System

AFW Auxiliary Feedwater

ANSI American National Standards Institute

AOP Abnormal Operating Procedure
ATWS Anticipated Transient Without Scram

CCW Component Cooling Water CFR Code of Federal Regulations

dc Direct Current

DPR Demonstration Power Reactor
DRS Division of Reactor Safety
ECR Engineering Change Request
EDG Emergency Diesel Generator
EOP Emergency Operating Procedure

EQ Environmental Qualification

ESFAS Engineered Safety Features Actuation System

F Fahrenheit

FSAR Final Safety Analysis Report

FW Feedwater
GL Generic Letter
gov Government

html Hypertext Markup Language http Hypertext Transfer Protocol

HVAC Heating Ventilation and Air Conditioning

HX Heat Exchanger IA Instrument Air

IMC Inspection Manual Chapter IPE Individual Plant Evaluation

IR Inspection Report IST In-service Testing

KPB Kewaunee and Point Beach LLC Limited Liability Company

LOV Loss of Voltage
MCC Motor Control Center
MFW Main Feedwater
MOV Motor Operated Valve

NEI Nuclear Energy Institute

NMC Nuclear Management Company

NRC United States Nuclear Regulatory Commission

NRR Office of Nuclear Reactor Regulation

OA Other Activities

PARS Publicly Available Records
PBNP Point Beach Nuclear Plant
PRA Probabilistic Risk Analysis

QA Quality Assurance QC Quality Control

LIST OF ACRONYMS USED

RG Regulatory Guide

RHR Residual Heat Removal
RPS Reactor Protection System
RCP Reactor Coolant Pump
RCS Reactor Coolant System
RWST Refueling Water Storage Ta

RWST Refueling Water Storage Tank

SD Shutdown

SDP Significance Determination Process

SE Safety Evaluation

SER Safety Evaluation Report

SFP Spent Fuel Pool SI Safety Injection

SSFI Safety Systems Functional Inspection

SW Service Water

TDAFW Turbine Driven Auxiliary Feedwater TRM Technical Requirements Manual

TS Technical Specification

USAR Updated Safety Analysis Report

USI Unresolved Safety Issue

V Volt

WE Wisconsin Electric wpd WordPerfect Document

www World Wide Web

LIST OF DOCUMENTS REVIEWED

The following is a list of licensee documents reviewed during the inspection, including documents prepared by others for the licensee. Inclusion on this list does not imply that NRC inspectors reviewed the documents in their entirety, but rather that selected sections or portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document in this list does not imply NRC acceptance of the document, unless specifically stated in the inspection report.

CALCULATIONS

Number	<u>Description</u>	Rev/Date
692301-2.2	Effect of Loss of Ventilation on Critical Plant Areas at the PBNP Following a Station Blackout	2
99-0031	Structural Qualification of D-07 and D-08 Seismic Brace	0
99-0117	Evaluate Temporary 8 Inch SW Discharge from CCW HX	1
Bechtel	Primary Auxiliary Building Ventilation Calculation	February 16, 1970
# 6118		
N93-060	WE - PBNP Battery D305 DC System Calculation	1
N94-056	SFP HX 013A/B - SW Flow vs Temperature Requirement	2
N94-130	4160V and 480V Safeguards Buses LOV Relay Setpoints	1
N-94-158	Verification of Required AFW Pump Differential Head for Accident Flow Rate	November 15, 1994
M-09334.212- AF.1	Turbine Driven AFW Low Pressure Capability	July 10, 1997
M-09334.425- AF.1	Evaluation of Impact of Check Valve AF-100/AF-101 Replacement	0
WE 0005-10	Determination of Time Available to Isolate the RWST	0
WE 300060	Piping System Qualification Report, Service Water Piping	1
WE 300068	Piping System Qualification Report - Service Water Piping, Return from HX-13A and HX-13B	0

CONDITION REPORTS (CRs) GENERATED DURING INSPECTION

Number	<u>Description</u>	Rev/Date
CAP001871	QA Calculation Cannot Be Found in Controlled Records	January 15, 2002
CAP001877	FSAR Discrepancy on Definition of RG 1.97 Category 2	January 16, 2002
CAP001886	Incomplete 10 CFR 50.59 Screening Answer for Changes to Relief Valve CC-736A/B	January 16, 2002
CAP001897	Letdown Isolation Valve Appendix R Operation During Degraded Voltage Conditions	January 17, 2002
CAP001900	Incomplete Corrective Actions Regarding Classification of RC-427	January 17, 2002
CAP001901	FSAR Chapter 9 Changes for CFC Service Water Pressure Not Also Made in Chapter 6	January 17, 2002
CAP001910	Safety Evaluation 2000-0014 Incomplete	January 17, 2002

CONDITION REPORTS (CRs) GENERATED DURING INSPECTION

Number	<u>Description</u>	Rev/Date
CAP001915	Questioning Corrective Action for CR00-2859 on	January 18, 2002
	Screening Procedure Temp Change	
CAP001916	Incorrect Answer to Tech Spec Change Question in	January 18, 2002
	10 CFR 50.59 Evaluation	
CAP003518	IT-21/22 Needs Procedural Tie	January 16, 2002

CONDITION REPORTS (CRs) GENERATED PRIOR TO INSPECTION

Number	<u>Description</u>	Rev/Date
96-1631	Unanticipated Alarm on G-03 During Start	June 28, 1997
97-0174	Incorrect SW System Design Pressure Stated in DG-M-03	January 20, 1997
97-1089	Momentary Air Bank Pressure Low Alarm Received When G03 EDG Started for Performance of TS-83	April 5, 1997
99-1618	Concern ID'd with SW System Operating Pressure During Design Basis Accident with Only 1 Unit in Operation	September 24, 1999
00-0128	P32 Thermocouple Installation Not Documented	January 11, 2000
00-0204	Inappropriate Use of Pre-Screening Criteria	January 20, 2000
00-1155	Less Than Adequate 10 CFR 50.59's	April 7, 2000
00-2841	Abnormal Operating Procedure 6E Concerns	September 22, 2000
00-2859	Installation of Temporary Gauge for AFW Pump Causes Potential to Violate Technical Specifications	September 27, 2000
01-1221	Procedures Incorrectly Exempted from 10 CFR 50.59 Screening	April 11, 2001
01-0403	Inadequate Safety Screening	February 8, 2001

DRAWINGS

Number	<u>Description</u>	Rev/Date
Enertech	Nozzle Check Valve, 3", Class 600 DRV-Z	3
PD99532		
EMD8413730	Schematic Diagram Diesel Generator G01 Start No. 1	December 11, 2001
Sh. 3	Circuitry	
EMD8413730	Schematic Diagram Diesel Generator G01 Start No. 2	December 11, 2001
Sh. 4	Circuitry	
EMD8413730	Schematic Diagram Diesel Generator G01 Annunciator	November 13, 2001
Sh. 6	Part-2	
MFWK	Feed Water System	38
00000138	•	
MAFK	Auxiliary Feed Water System	69
00000169	·	

ENGINEERING CHANGE REQUESTS (ECRs)

<u>Number</u>	Description	Rev/Date
1999-0186	MR 98-024*B	October 26, 1999

ENGINEERING CHANGE REQUESTS (ECRs)

Number	Description	Rev/Date
2000-0016	MR 98-024*B	March 16, 2000
2000-0020	MR 98-024*E	March 28, 2000
2000-0023	MR 98-024*B	September 18, 2000
2000-0024	MR 98-024*B	September 14, 2000
2000-0025	MR 98-024*B	September 11, 2000
2000-0026	MR 98-024*B	September 18, 2000
2000-0030	MR 98-024*E	April 4, 2000
2000-0032	MR 98-024*B	September 18, 2000
2000-0033	MR 98-024*B	September 18, 2000
2000-0034	MR 98-024*E	May 30, 2000
2000-0036	MR 98-024*E	August 22, 2000
2000-0038	MR 98-024*E	August 22, 2000
2000-0039	MR 98-024*E	April 11, 2000
2000-0040	MR 98-024*E	May 31, 2000
2000-0041	MR 98-024*E	April 5, 2000
2000-0042	MR 98-024*E	May 3, 2000
2000-0043	MR 98-024*E	April 6, 2000
2000-0044	MR 98-024*B	September 14, 2000
2000-0046	MR 98-024*E	April 7, 2000
2000-0047	MR 98-024*E	April 11, 2000
2000-0049	MR 98-024*E	April 8, 2000
2000-0050	MR 98-024*E	April 10, 2000
2000-0052	MR 98-024*E	April 10, 2000
2000-0054	MR 98-024*E	April 12, 2000
2000-0056	MR 98-024*E	April 26, 2000
2000-0060	MR 98-024*E	April 20, 2000
2000-0061	MR 98-024*E	April 26, 2000
2000-0063	MR 98-024*E	May 23, 2000
2000-0064	MR 98-024*E	April 28, 2000
2000-0066	MR 98-024*E	May 3, 2000
2000-0078	MR 98-024*E	June 1, 2000
2000-0085	MR 98-024*E	June 14, 2000
2000-0093	MR 98-024*E	June 14, 2000
2000-0094	MR 98-024*E	June 14, 2000
2000-0101	MR 98-024*E	June 19, 2000
2000-0108	MR 98-024*E	June 29, 2000
2000-0109	MR 98-024*E	July 4, 2000
2000-0112	MR 98-024*E	July 10, 2000
2000-0117	MR 98-024*E	July 8, 2000
2000-0128	MR 98-024*B	September 18, 2000
2000-0135	MR 98-024*B	September 11, 2000
2000-0137	MR 98-024*B	September 11, 2000

ENGINEERING CHANGE REQUESTS (ECRs)

<u>Number</u>	Description	Rev/Date
2000-0141	MR 98-024*B	September 13, 2000
2000-0156	MR 98-024*E	August 11, 2000
2000-0157	MR 98-024*B	October 3, 2000
2000-0158	MR 98-024*B	September 18, 2000
2000-0161	MR 98-024*B	September 18, 2000
2000-0165	MR 98-024*B	September 14, 2000
2000-0174	MR 98-024*B	October 4, 2000
2000-0204	MR 98-024*E	October 26, 2000
2001-0017	MR 98-024*E	February 14, 2001
2001-0027	MR 98-024*E	February 27, 2001

MODIFICATIONS:

Number	Description	Rev/Date
MR 94-048	Seismic Modifications of 1A-05, 2A-05, D-07, and D-08	February 10, 1999
MR 97-129*A	U1 Replace Check Valves AF-100 and AF-101 With a New Style	May 27, 1999
MR 98-024*B	U0 Spent Fuel Pool Cooling Service Water Alternate Supply/ Return Lines	April 28, 1999
MR 98-024*E	U0 Service Water From Component Cooling Water Heat Exchanger Return Header Piping/Isolation	January 10, 2000
MR 98-047	U1 Pressurizer Level Controller Upgrade	April 13, 1998
MR 99-005*C	Instrument Air Containment Isolation Valve Trim Package Replacement	June 17, 1999
MR 99-0055	Instrument Air Containment Isolation Valve Logic Change	February 17, 2000
MR 99-029*A	AFW Pump P-38A Minimum Flow Re-circulation Line Orifice	January 28, 2000
MR 00-051	RWST (T-013) Vent Upgrade - Unit 1	June 12, 2000
MR 00-070	Elimination of Nuisance Low Starting Air Pressure Alarms During EDG Start of G01	May 21, 2001

PROCEDURES

<u>Number</u>	<u>Description</u>	Rev/Date
AOP-6E	Alternate Boration/Loss of Shutdown Margin	8 & 9
AOP-6F	Low Concentration Water Pockets in RCS	2
AOP-6H	Quadrant Power Tilt	0
AOP-27	Recovery from Inadvertent Containment Isolation (Unit 2)	0
CSP-H.1	U2 Response to Loss of Secondary Heat Sink	22
CSP-S.1	Response to Nuclear Power Generator ATWS	22
EOP-0.1	Reactor Trip Response - Unit 2	25
EOP-1.1	Safety Injection Termination (Unit 2)	28
EOP-1.3	Transfer to Containment Sump Re-circulation Unit 2	26

PROCEDURES

Number	Description	Rev/Date
IT-08A	Cold Start of TDAFW Pump and Valve Test (Quarterly)	25
	Unit 1	
IT-10	Test of Electrically-Driven AFWPs and Valves (Quarterly)	43
IT-10	Temporary Change 2000-0463	39
IT-110	Instrument Air Valves (Quarterly) Unit 1	16
IT-115	Instrument Air Valves (Quarterly) Unit 2	15
IT-305B	Steam Generator B Main Feed Line Check Valves	7
	(Cold SD) Unit 2	
IT-350	SI Check Valves (Cold Shutdown)	2
IWP96-071*	EDG G01 Governor Replacement Post-Modification	0
A2	Testing	
NP1.1.2	Procedure and Administrative Controls	4
NP1.1.3	Procedure Preparation Review and Approval	14
NP1.1.4	Use and Adherence of Procedures and Work Plans	6
NP5.1.8	10CFR50.59/72.48 Applicability, Screening and Evaluation	1
NP7.2.1	Plant Modifications	8
NP10.3.1	Authorization of Changes, Tests, and Experiments	14
OM3.7	AOP and EOP Procedure Sets Use and Adherence	10
ORT47	IA Supply Unit 1	17
ORT47	IA Supply Unit 2	14
ORT48	IA Supply Unit 1	16
ORT48	IA Supply Unit 2	12
1ICP02.003-	RPS Logic Train A Monthly Surveillance Test	11 & 12
A-1		
	RPS Logic Train B Monthly Surveillance Test	12 & 13
B-1		
1SOP-CC-001 Component Cooling System 6 & 7		

REFERENCES

Number	Description	Rev/Date
EWR 97-159	Determine Replacement Check Valve Design for	December 16, 1997
DBD-02	AF-100/101 CCW System Design Basis Document	0
DBD-02 DBD-29	Auxiliary Building and Control Building HVAC	0
FSAR	Chapters 6 and 9	June 2001
Letter	NRC SER on Resolution of USI A-46 at PBNP	July 7, 1998
Letter	Conformance to RG 1.97 for PBNP	July 11, 1986
Memo	Hannemann to O'Brien, Follow-Up on Plant Walkdown of PAB Ventilation System,	January 17, 2002
2000-026	Manager's Supervisory Staff Meeting Minutes	May 2, 2000
NPM 99-0864	Seismic Evaluation of Auxiliary Bldg HVAC System	July 29, 1999

REFERENCES

Number	<u>Description</u>	Rev/Date
NPM 99-1099	Downgrading Seismic Classification of FW Regulating &	September 28,
	Bypass Valves	1999
PBF-2031	Auxiliary Building Shift Log	43
PBF-2031	Daily Logsheet	56
RG 1.97	Instrumentation For Light-Water-Cooled Nuclear Power Plants To Assess Plant And Environs Conditions During And Following An Accident - Revision 2	December 1, 1980
SPEED 2000-131	Set Point, Drawing Number, and Capacity Change for CC-736A Spare Parts Equivalency Document	November 8, 2000

50.59 EVALUATIONS(SEs)

7.1101to(020)	
<u>Description</u>	Rev/Date
Instrument Air Containment Isolation Valve Trim Pack Replacement	March 22, 2000
Downgrade the Seismic Design Classification of Auxiliary Building Ventilation and MFW Equipment That Isolates the Supply of MFW to the Steam Generators in FSAR	February 16, 2000
Chapter 8, DG Systems FSAR Changes	0
FSAR Phase II Review and Upgrade Project, Section 9.6 SW	August 30, 2000
Refueling Water Storage Tank (RWST) Seismic Upgrade	April 4, 2000
MR 98-024*V - Modify SI Logic for Non-Essential SW Load Isolation Valves	May 3, 2000
AFW Pump P-38A and P-38B Minimum Flow Recirculating Line Flow Orifice Replacement	May 10, 2000
MR 97-014*E, Transfer dc Supplies for Inverters 1DY-01 and 2DY-02, Install Panel D-26 and Transfer Loads to D-26	June 20, 2000
Changes to SW System Operating Procedures and FSAR	August 9, 2000
Revise Procedure for Spurious Operation of SI 851 A/B Valves	September 21, 2000
CCW System Closed Loop Inside Containment	0
MR 98-024*J - Unit 1 Containment Fan Cooler and Fan Motor Cooler Replacement	March 14, 2001
MR 00-103, SW Upgrades to EDG, G02	June 6, 2001
TRM 3.5.1/TSR 3.5.1.3 & TSR 3.5.1.3 Bases (Draft Rev. C)	October 16, 2001
Filling RHR Common Suction From RWST With P-33	April 3, 2001
	Description Instrument Air Containment Isolation Valve Trim Pack Replacement Downgrade the Seismic Design Classification of Auxiliary Building Ventilation and MFW Equipment That Isolates the Supply of MFW to the Steam Generators in FSAR Chapter 8, DG Systems FSAR Changes FSAR Phase II Review and Upgrade Project, Section 9.6 SW Refueling Water Storage Tank (RWST) Seismic Upgrade MR 98-024*V - Modify SI Logic for Non-Essential SW Load Isolation Valves AFW Pump P-38A and P-38B Minimum Flow Recirculating Line Flow Orifice Replacement MR 97-014*E, Transfer dc Supplies for Inverters 1DY-01 and 2DY-02, Install Panel D-26 and Transfer Loads to D-26 Changes to SW System Operating Procedures and FSAR Revise Procedure for Spurious Operation of SI 851 A/B Valves CCW System Closed Loop Inside Containment MR 98-024*J - Unit 1 Containment Fan Cooler and Fan Motor Cooler Replacement MR 00-103, SW Upgrades to EDG, G02 TRM 3.5.1/TSR 3.5.1.3 & TSR 3.5.1.3 Bases (Draft Rev. C)

50.59 SCREENS(SCRs)

<u>Number</u>	<u>Description</u>	Rev/Date
1999-0430	Replace 3" AFW Check Valves 1AF-0100 and 1AF-0101	May 28, 1999

50.59 SCREENS(SCRs)

Number	Description	Rev/Date
2000-0023	RMP 9200-6, Switchyard Battery D-800, D-801, and D-807 Capacity Test	0
2000-0062	Changes to PBNP 6/99 FSAR Section 8.6 120Vac Vital Instrument Power (Y)	January 28, 2000
2000-0072	Changes to PBNP June 99 FSAR Section 7.2, RPS, 7.3, ESFAS, and 7.6.1, Nuclear Instrumentation System	February 3, 2000
2000-0100	1-SOP-CC-001, Component Cooling System, Revision 2, 2-SOP-CC-001, Component Cooling System, Revision 3, PBF 2031, Auxiliary Building Shift Log, Revision 43	February 9, 2000
2000-0207	Revise TLB-59 (Tank Level Book, Diesel Generator G-03/G-04 Day Tank T-176A/B)	March 6, 2000
2000-0401	Temporary Ventilation for Main Feed Pump Motors	May 8, 2000
2000-0695	Replacement Valve and Setpoint Change for Unit 1 & 2 CC-736A/B	September 7, 2000
2000-0783	50.59 Screening, AF Vent & Drain Temporary Gauges	September 14, 2000
2000-0975	Relief Valve Capacity Change for Unit 2 CC-736A	October 28, 2000
2001-0053	Temporary Changes to EDG G-03 Temperature and Alarm Functions	January 19, 2001
2001-0067	Downgrade CCW Heat Exchanger SW Relief Valves From Safety Related to Augmented Quality	January 31, 2001
2001-0084	Service Water System Malfunction	January 26, 2001
2001-0141	Revise RMP 9367, Containment Coatings to Include Lenience on Dry Film Thickness Range for Carboguard 890	February 16, 2001
2001-0205	AOP 27, Recovery from Inadvertent Containment Isolation	March 6, 2001
2001-0208	ORT 69, Revision 21, CCW To and From 1P-1B Refueling Shutdown Unit 1	March 6, 2001
2001–0216	Filling RHR Common Suction From RWST With P-33	April 3, 2001

LIST OF INFORMATION REQUESTED

INITIAL DOCUMENT REQUEST

(Sent to Licensee on December 18, 2001)

Inspection Report: 50-266/02-002(DRS); 50-301/02-002(DRS)

Inspection Dates: January 14 -18, 2002

Inspection Procedures: IP 71111.02, "Evaluations of Changes, Tests, or Experiments"

IP 71111.17, "Permanent Plant Modifications"

Lead Inspector: George M. Hausman, (630) 829-9743

I. Information Needed for In-Office Preparation Week

The following information is needed by January 7, 2002, or sooner, to facilitate the selection of specific items that will be reviewed during the on-site inspection week. The team will select specific items from the information requested below and submit a list to your staff by January 10, 2002. We request that the specific items selected from the lists be available and ready for review on the first day of inspection. If you have any questions regarding this information, please call the team leader as soon as possible. All lists requested should cover a two year span. All information should be sent electronically (as a PDF or WordPerfect/Word document) if at all possible.

- a. <u>List</u> of permanent plant modifications to risk significant systems, structures, or components. For the purpose of this inspection, permanent plant modifications include permanent: (i) plant changes, (ii) design changes, (iii) set point changes, (iv) procedure changes, (v) equivalency evaluations, (vi) suitability analyses, (vii) calculations, and (viii) commercial grade dedications.
- **b.** <u>List</u> of all 10 CFR 50.59 completed evaluations involving: (i) changes to facility (modifications), (ii) procedure revisions, (iii) tests or non-routine operating configurations, (iv) changes to the UFSAR, and (v) calculations.
- <u>List</u> of all 10 CFR 50.59 screenings for changes that have been screened out as not requiring a full evaluation involving: (i) changes to facility (modifications), (ii) procedure revisions, (iii) tests or non-routine operating configurations, (iv) changes to the UFSAR, and (v) calculations.
- **d.** <u>List</u> of all 10 CFR 50.59 validations or applicability reviews done in lieu of a full 10 CFR 50.59 evaluation.
- **e.** <u>List</u> of corrective action documents (open and closed) that address the plant permanent modification process and 10 CFR 50.59 issues or concerns.

LIST OF INFORMATION REQUESTED

- **f.** <u>List</u> of special tests or experiments and non-routine operating configurations.
- **g.** Copies of modification procedure(s), post-modification testing procedure(s) and 10 CFR 50.59 procedures.
- **h.** Name(s) and phone numbers for the regulatory and technical contacts.
- i. Latest Organizational Chart.

II. Information to be Available on First Day of Inspection (January 14, 2002)

- a. Copies of all selected items need to be ready for the inspectors when they arrive on-site. Selected items will be relayed to the NRC coordinator/contact person by January 10th.
- **b.** The following information needs to be <u>available</u> to the team once they arrive on-site. Copies of these documents do not need to be solely available to the team as long as the inspectors have easy and unrestrained access to them.
 - Updated Final Safety Analysis Report
 - Technical Specifications
 - Latest IPE/PRA Report
 - Vendor Manuals
 - The Latest 10 CFR 50.59 FSAR Update Submittal
- c. Please ensure that all supporting documents for the selected items have been located and are readily retrievable as the team will likely be requesting these documents during the inspections. Examples of supporting documents are:
 - Calculations supporting the selected modifications or 10 CFR 50.59 evaluations
 - Post-modification tests for the selected modifications
 - Copies of other evaluations or validations referred to by a modification or 10 CFR 50.59 evaluation.
 - Drawings supporting the modifications.
 - Procedures affected by the modifications.

LIST OF INFORMATION REQUESTED

UFSAR change papers generated due to a 10 CFR 50.59 evaluation.

III. <u>Information to Be Provided Throughout the Inspection</u>

- **a.** Copies of any Condition Report generated as a result of the team's questions or queries during this inspection.
- **b.** Copies of the list of questions submitted by the team members and the status/resolution of the information requested (provided daily during the inspection to each team member).