December 22, 2000

Mr. M. Reddemann Site Vice President Kewaunee and Point Beach Nuclear Plants Wisconsin Electric Power Company 6610 Nuclear Road Two Rivers, WI 54241

SUBJECT: POINT BEACH NUCLEAR PLANT - NRC INSPECTION REPORT

50-266/00-16(DRS); 50-301/00-16(DRS)

Dear Mr. Reddemann:

On December 8, 2000, the NRC completed a baseline inspection at your Point Beach Nuclear Plant. The results of this inspection were discussed on December 8, 2000, with you and members of your staff. The enclosed report presents the results of that inspection.

The inspection was an examination of activities conducted under your license as they relate to emergency preparedness and to compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas the inspection consisted of a selective examination of procedures and representative records, observations of activities, and interviews with personnel. Specifically, this inspection focused on performance during your biennial emergency preparedness exercise and your staff's capability to self-assess your participants' performance. In addition, we reviewed your staff's determinations of performance indicators for the Emergency Preparedness Cornerstone. We also reviewed records related to the June 2000 relocation of the alternate emergency operations facility.

Based on the results of this inspection, no findings of significance were identified.

In accordance with 10 CFR Part 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

We will gladly discuss any question you have concerning this inspection.

Sincerely,

/RA/

Gary L. Shear, Chief Plant Support Branch Division of Reactor Safety

Docket Nos. 50-266; 50-301 License Nos. DPR-24; DPR-27

Enclosure: Inspection Report 50-266/00-16(DRS);

50-301/00-16(DRS)

cc w/encl: R. Grigg, President and Chief

Operating Officer, WEPCo

M. Wadley, Chief Nuclear Officer, NMC

J. Gadzala, Licensing Manager D. Weaver, Nuclear Asset Manager

F. Cayia, Plant Manager J. O'Neill, Jr., Shaw, Pittman,

Potts & Trowbridge

K. Duveneck, Town Chairman

Town of Two Creeks
B. Burks, P.E., Director
Bureau of Field Operations
A. Bie, Chairperson, Wisconsin
Public Service Commission
S. Jenkins, Electric Division

Wisconsin Public Service Commission

State Liaison Officer

W. Curtis, FEMA, Region V

We will gladly discuss any question you have concerning this inspection.

Sincerely,

/RA/

Gary L. Shear, Chief Plant Support Branch Division of Reactor Safety

Docket Nos. 50-266; 301 License Nos. DPR-24; DPR-27

Enclosure: Inspection Report 50-266/00-16(DRS); 50-301/00-16(DRS)

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DATE	12/22 /00	12/22 /00	12/22/00	

U.S. NUCLEAR REGULATORY COMMISSION REGION III

Docket Nos: 50-266; 50-301 License Nos: DPR-24; DPR-27

Report No: 50-266/00-16(DRS); 50-301/00-16(DRS)

Licensee: Nuclear Management Company, LLC

Facility: Point Beach Nuclear Plant, Units 1 and 2

Location: 6610 Nuclear Road

Two Rivers, WI 54241

Dates: December 4-8, 2000

Inspectors: T. Ploski, Senior Emergency Preparedness Analyst

J. Foster, Emergency Response Coordinator

R. Powell, Resident Inspector

Approved by: Gary L. Shear, Chief, Plant Support Branch

Division of Reactor Safety

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety

Radiation Safety

Safeguards

- Initiating Events
- Mitigating Systems
- Barrier Integrity
- Emergency Preparedness
- Occupational
 - Public
- Physical Protection

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: http://www.nrc.gov/NRR/OVERSIGHT/index.html.

SUMMARY OF FINDINGS

IR 05000266-00-16(DRS), IR 05000301-00-16(DRS), on 12/4-8/2000, Nuclear Management Company, Point Beach Nuclear Plant, Units 1 & 2. Emergency Preparedness.

The report covers a one week period of announced inspection by two regional emergency preparedness inspectors and a resident inspector. This inspection focused on the Reactor Safety, Emergency Preparedness Cornerstone, and included the following: evaluation of licensee staff's capability to assess licensee participants' performance during the biennial emergency preparedness exercise; review of the three emergency preparedness performance indicators; and review of records related to the June 2000 relocation of the alternate emergency operations facility.

REACTOR SAFETY

Cornerstone: Emergency Preparedness

 No findings of significance were identified during this inspection (Section 1EP1, Section 4OA1, and Section 4OA4).

Report Details

1. REACTOR SAFETY

Cornerstone: Emergency Preparedness (EP)

1EP1 Drill, Exercise, and Actual Events

a. <u>Inspection Scope</u>

The inspectors reviewed the 2000 exercise's objectives and scenario to ensure that the exercise would acceptably test major elements of the licensee's emergency plan. The inspectors verified that the simulated problems provided an acceptable framework to support demonstration of the licensee's capabilities to implement its emergency plan. The inspectors also reviewed records of a practice drill conducted in October 2000, in order to determine whether the associated scenario was sufficiently different from the scenario used in the December 5 exercise.

The inspectors evaluated the licensee's exercise performance, focusing on the risk-significant activities of emergency classification, notification, and protective action decision making, as well as implementation of accident mitigation strategies in the following emergency response facilities:

- Control Room Simulator (CRS)
- Technical Support Center (TSC)
- Operations Support Center (OSC)
- Emergency Operations Facility (EOF)

The inspectors also assessed the licensee's recognition of abnormal plant conditions, transfer of responsibilities between facilities, internal communications, interface with offsite officials, readiness of emergency facilities and related equipment, and overall implementation of the licensee's emergency plan.

The inspectors attended post-exercise critiques in the CRS, TSC and EOF to evaluate the licensee's initial self-assessment of its exercise performance. The inspectors later met with the licensee's emergency preparedness staff to obtain the licensee's refined assessments of its exercise participants' and controllers' performances. These self-assessments were then compared with the inspectors' independent observations and related assessments.

b. <u>Findings</u>

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA1 Performance Indicator (PI) Verification

a. Inspection Scope

The inspectors reviewed records related to each of the three EP indicators, Alert and Notification System (ANS), Emergency Response Organization (ERO) Drill Participation, and Drill and Exercise Performance (DEP), to verify the accuracy and completeness of data submitted through September 2000. The inspectors also reviewed procedural guidance related to the gathering and assessment of PI-related information. Records related to the raw data for each indicator were evaluated.

b. Findings

No findings of significance were identified.

4OA4 Alternate EOF (AEOF) Relocation

a. Inspection Scope

The inspector reviewed training records and procedure revisions related to the June 2000 relocation of the licensee's AEOF from Milwaukee, Wisconsin to the Kewaunee Nuclear Power Plant's EOF in Green Bay, Wisconsin, which was closer to the plant than the discontinued AEOF in Milwaukee.

b. Findings

No findings of significance were identified.

4OA6 Management Meetings

Exit Meeting Summary

The inspectors presented the inspection results to Mr. M. Reddemann and other members of licensee management and staff on December 8, 2000. The licensee acknowledged the information presented and did not identify any as being proprietary.

PARTIAL LIST OF PERSONS CONTACTED

<u>Licensee</u>

- A. Cayia, Plant Manager
- V. Kaminskas, Maintenance Manager
- D. Gehrke, Organizational Assessment Supervisor
- R. Hayden, EP Supervisor
- B. O'Grady, Operations Manager
- M. Reddemann, Site Vice President
- R. Repshas, Site Services Manager

NRC

R. Powell, Resident Inspector

	ITEMS OPENED, CLOSED, AND DISCUSSED
<u>Opened</u>	
None	
Closed	
None	
Discussed	
None	

LIST OF ACRONYMS USED

AEOF Alternate Emergency Operations Facility

ANS Alert and Notification System
CFR Code of Federal Regulations
CRS Control Room Simulator

DEP Drill and Exercise Performance
DRS Division of Reactor Safety
EOF Emergency Operations Facility
EP Emergency Preparedness

EPIP Emergency Preparedness Implementing Procedure EPMP Emergency Preparedness Maintenance Procedure

ERO Emergency Response Organization

FEMA Federal Emergency Management Agency

IR Inspection Report

JPIC Joint Public Information Center

NEI Nuclear Energy Institute

NRC Nuclear Regulatory Commission

OA Other Activities

OSC Operations Support Facility
PI Performance Indicator
TSC Technical Support Center

INSPECTION PROCEDURES USED

71114 Reactor Safety-Emergency Preparedness

71114.01 Exercise Evaluation

71151 Performance Indicator Verification

LIST OF DOCUMENTS REVIEWED

Miscellaneous

"Point Beach Power Plant Emergency Plan"

Administrative Procedure NP 5.2.16, "NRC Performance Indicators," Revision 1

Periodic Test Results for ANS Equipment in Manitowoc County October 1999 - September 2000

"Point Beach Nuclear Plant ERO Drill Participation October 1997 - September 2000"

Exercise Scenario Manual for the EP Exercise conducted on December 5, 2000

Memorandum from the EP Supervisor to the Site Vice President, dated October 2000: "Drill versus Exercise Similarities/Liability"

Records of EP Drills conducted on August 5, 1999; December 14, 1999; April 20, 2000; July 6, 2000; and August 3, 2000

Records of Accident Assessment Training conducted in November and December 1999

Training Handout titled, "Point Beach AEOF Orientation Day - June 30, 2000"

Letter, dated August 1, 2000, from L. Borchart of Wisconsin Emergency Management to the EP Supervisor titled, "Alternate EOF Relocation"

Report S-P-00-03, "Assessment of the Point Beach Plant's Emergency Planning Zone Sirens" Report RCE 00-076, "ANS Siren Failures"

Supplemental IR 50-266/00-12(DRS) and 50-301/00-12(DRS)

NEI 99-02, Revision 0, "Regulatory Assessment Performance Indicator Guideline"

Condition Reports

00-1412; 00-2554; 00-2812

Procedures

- EPIP 1.1, Revision 35, "Course of Actions"
- EPIP 1.2, Revision 33, "Emergency Classification"
- EPIP 1.3, Revision 27, "Dose Assessment and Protective Action Recommendations"
- EPIP 2.1, Revision 19, "Notifications ERO, State, Counties, and NRC"
- EPIP 4.1, Revision 29, "TSC Activation and Evacuation"
- EPIP 4.2, Revision 14, "OSC Activation and Evacuation"
- EPIP 4.3, Revision 24, "EOF Activation and Evacuation"
- EPIP 4.7, Revision 1, "Offsite Radiation Protection Facility Activation and Evacuation"
- EPIP 5.1, Revision 13, "Personnel Emergency Dose Authorization"
- EPIP 6.1, Revision 18, "Assembly and Accountability, Release and Evacuation of Personnel"
- EPIP 10.1, Revision 20, "Emergency Reentry"
- EPIP 12.1, Revision 7, "Emergency De-escalation, Termination, or Recovery Operations"
- EPIP 12.2, Revision 12, "Recovery Implementation"
- EPMP 1.3, Revision 13, "Routine Inventory of TSC, EOF, AEOF, JPIC, and OSC EP Supplies"