

UNITED STATES NUCLEAR REGULATORY COMMISSION

REGION II SAM NUNN ATLANTA FEDERAL CENTER 61 FORSYTH STREET, SW, SUITE 23T85 ATLANTA, GEORGIA 30303-8931

May 5, 2006

Carolina Power & Light Company
ATTN: Mr. C. J. Gannon, Jr.
Vice President - Harris Plant
Shearon Harris Nuclear Power Plant
P. O. Box 165, Mail Code: Zone 1
New Hill, NC 27562-0165

SUBJECT: SHEARON HARRIS NUCLEAR POWER PLANT- NRC EMERGENCY

PREPAREDNESS INSPECTION REPORT 50-400/2006008

Dear Mr. Gannon:

On April 3-7, 2006, the US Nuclear Regulatory Commission (NRC) completed an inspection at your Harris Plant. The enclosed inspection report documents the inspection findings, which were discussed on April 3-7, 2006, with Mr. Edward Wills and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

Based on the results of this inspection no findings of significance were identified. However, a licensee identified violation which was determined to be of very low significance is listed in the report. The NRC is treating the violation as a non-cited violation (NCV) consistent with Section VI.A of the NRC Enforcement Policy because of the very low significance of the violation and because it is centered into your corrective action program. If you contest this NCV, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the Nuclear Regulatory Commission, ATTN.: Document Control Desk, Washington DC 20555-0001; with copies to the Regional Administrator Region II; the Director, Office of Enforcement, United States Nuclear Regulatory Commission, Washington, DC 20555-0001; and the NRC Resident Inspector at Shearon Harris Nuclear Power Plant.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be available electronically for public inspection in the NRC Public Document Room (PDR) or from the Publically Available Records (PARS) component of NRC's document system (ADAMS). To the extent possible, your response should not include any personal privacy, proprietary, or safeguards information so that it can be

placed in the PDR and PARS without redaction. ADAMS is accessible from the NRC Website at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Sincerely,

/RA/

Brian R. Bonser, Chief Plant Support Branch 2 Division of Reactor Safety

Docket Nos.: 50-400 License Nos: NPF-83

Enclosure: Inspection Report 05000400/2006008

w/Attachment: Supplemental Information

cc w/encl:

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U. S. NUCLEAR REGULATORY COMMISSION

REGION II

Docket Nos.: 50-400

License Nos.: NPF-63

Report No: 05000400/2006008

Licensee: Progress Energy

Facility: Shearon Harris Nuclear Power Plant

Location: 5421 Shearon Harris Road

New Hill, NC 27562-9998

Dates: April 3-7, 2006

Inspectors: Lee Miller, Senior Emergency Preparedness Inspector

(Section 1EP2, 1EP3, 1EP4, 1EP5, 4OA1)

Approved by: Brian R. Bonser, Chief

Plant Support Branch 2 Division of Reactor Safety

SUMMARY OF FINDINGS

IR No. 05000400/2006008; 04/03/2006 - 04/07/2006; Harris Nuclear Power Plant; Emergency Preparedness

The report covered a one week announced inspection be a regional senior emergency preparedness inspector. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using Inspection Manual Chapter (IMC) 0609, "Significance Determination Process" (SDP). Findings for which the SDP does not apply may be Green or be assigned a severity level after the NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUNREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

A. NRC-Identified and Self-Revealing Findings

Cornerstone: Emergency Preparedness

No findings of significance were identified.

B. Licensee-Identified Violations.

Violations of very low safety significance, which were identified by the licensee have been reviewed by the inspectors. Corrective actions taken or planned by the licensee have been entered into the licensee's corrective action program. These violations and corrective actions are listed in Section 4OA7 of this report.

REPORT DETAILS

REACTOR SAFETY

Cornerstone: Emergency Preparedness

1EP2 Alert and Notification System Testing

a. Inspection Scope

The inspectors evaluated the adequacy of licensee methods for testing the alert and notification system in accordance with NRC Inspection Procedure 71114, Attachment 02, "Alert and Notification System (ANS) Testing". The applicable planning standard 10 CFR Part 50.47(b)(5) and its related 10 CFR Part 50, Appendix E, Section IV.D requirements were used as reference criteria. The criteria contained in NUREG-0654, "Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants," Revision 1, was also used as references.

The inspectors reviewed various documents which are listed in the Attachment to this report.

b. Findings

No findings of significance were identified.

1EP3 Emergency Response Organization (ERO) Augmentation

a. Inspection Scope

The inspectors reviewed the ERO augmentation staffing requirements and the process for notifying the ERO to ensure the readiness of key staff for responding to an event and timely facility activation. The results of the 03/14/2006 unannounced off-hours augmentation drill was reviewed. The inspectors conducted a review of the backup notification systems. The qualification records of key position ERO personnel was reviewed to ensure all ERO qualifications were current. A sample of problems identified from augmentation drills or system tests performed since the last inspection were reviewed to assess the effectiveness of corrective actions.

The inspection was conducted in accordance with NRC Inspection Procedure 71114, Attachment 03, "Emergency Response Organization (ERO) Augmentation Testing." The applicable planning standard, 10 CFR 50.47(b)(2) and its related 10 CFR 50, Appendix E requirements were used as reference criteria.

The inspectors reviewed various documents which are listed in the Attachment to this report.

b. Findings

No findings of significance were identified.

1EP4 Emergency Action Level (EAL) and Emergency Plan Changes

a. <u>Inspection Scope</u>

The inspectors evaluated the associated 10 CFR 50.54(q) reviews associated with non-administrative emergency plan, implementing procedures and EAL changes. The revision 50 of the Emergency Plan covered the period from 03/22/2005 to 04/07/2006.

The inspection was conducted in accordance with NRC Inspection Procedure 71114, Attachment 01, "Emergency Action Level and Emergency Plan Changes." The applicable planning standard, 10 CFR 50.47(b)(4) and its related 10 CFR 50, Appendix E requirements were used as reference criteria. The criteria contained in NUMARC/NESP-007, "Methodology for Development of Emergency Action Levels", Revision 2, Regulatory Guide 1.101 were also used as references.

The inspectors reviewed various documents which are listed in the Attachment to this report.

b. Findings

No findings of significance were identified.

1EP5 Correction of Emergency Preparedness Weaknesses and Deficiencies

a. Inspection Scope

The inspectors reviewed the corrective actions identified through the EP program to determine the significance of the issues and to determine if repeat problems were occurring. The facility's self-assessments and audits were reviewed to assess the licensee's ability to be self-critical, thus avoiding complacency and degradation of their EP program. In addition, inspectors review licensee's self-assessments and audits to assess the completeness and effectiveness of all EP-related corrective actions.

The inspection was conducted in accordance with NRC Inspection Procedure 71114, Attachment 05, "Correction of Emergency Preparedness Weaknesses and Deficiencies." The applicable planning standard, 10 CFR 50.47(b)(14) and its related 10 CFR 50, Appendix E requirements were used as reference criteria.

The inspectors reviewed various documents which are listed in the Attachment to this report.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA1 Performance Indicator (PI) Verification

a. Inspection Scope

The inspectors reviewed the licensee's procedure for developing the data for the Emergency Preparedness PIs which are: (1) Drill and Exercise Performance (DEP); (2) ERO Drill Participation; and (3) ANS Reliability. The inspectors examined data reported to the NRC for the period 01/01/2005 to 12/31/2005. Procedural guidance for reporting PI information and records used by the licensee to identify potential PI occurrences were also reviewed. The inspectors verified the accuracy of the PI for ERO drill and exercise performance through review of a sample of drill and event records. The inspectors reviewed selected training records to verify the accuracy of the PI for ERO drill participation for personnel assigned to key positions in the ERO. The inspectors verified the accuracy of the PI for alert and notification system reliability through review of a sample of the licensee's records of periodic system tests.

The inspection was conducted in accordance with NRC Inspection Procedure 71151, "Performance Indicator Verification." The applicable planning standard, 10 CFR 50.9 and NEI 99-02, Revision 3, "Regulatory Assessment Performance Indicator Guidelines," were used as reference criteria. This inspection activity represents three samples on an annual basis.

The inspectors reviewed various documents which are listed in the Attachment to this report.

b. Findings

No findings of significance were identified.

40A6 Meetings, including Exit

On April 7, 2006, the inspector presented the inspection results to Mr. E. Willis, Manager - Operations, and other members of his staff acknowledged the findings. The team lead stated that additional information would be accepted and reviewed. The inspectors confirmed that proprietary information was not provided or taken during the inspection.

40A7 Licensee-Identified Violations

The following violation of very low safety significance (Green) was identified by the licensee and is a violation of NRC requirements which meet the criteria of Section VI of the NRC Enforcement Policy, NUREG-1600, for being dispositioned as an NCV.

10CFR50.47(b)(5) requires in part that the public alert and notification system meets the design requirements of FEMA-REP-10 or is compliant with FEMA approved Alert and Notification System (ANS) design report and supporting FEMA approval letter. Contrary to this, on January 24, 2006, during a review of the Harris Plant tone alert radio database, it was discovered that 92 addresses in the 5 mile radius of the Harris Nuclear

Plant did not have tone alert radios. This was identified in the licensee's Corrective Action Program, Action Request Number: 182074, tone alert radios not in all residences as required by the E Plan. This finding is of very low safety significance because the licensee demonstrated that all of the 92 addresses could be notified with backup route alerting within 45 minutes and the licensee took prompt action to deliver tone alert radios to the 92 homes. The last radio was delivered on February 1, 2006.

ATTACHMENT: Supplemental Information

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

<u>Licensee personnel</u>

None

- E. Wills, Manager Operations
- P. Fulford, Manager Nuclear Assessment
 D. Corlett, Supervisor Licensing/Regulatory Programs
 T. Pilo, Emergency Preparedness Supervisor
- M. Janus, Lead Assessor Nuclear Assessment
- J. Dills, Maintenance Superintendent

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

	LIST OF ITEMS O	PENED, CLOSI	ED, AND DISC	USSEL
<u>Openned</u>				
None				
Openned and Close	<u>ed</u>			
None				
Closed				

A-1 Attachment

LIST OF DOCUMENTS REVIEWED

Section 1EP2: Alert and Notification System Testing

Records and Data

Reviewed Maintenance data sheets and post-maintenance testing, January - December, 2005

Miscellaneous

CP&L Harris Emergency Warning System Operating Manual, October 10, 2001 CP&L Harris Touch Screen Operator's Manual, November 5, 2001 Siren Notification System Study Report and Recommendations, December 1986 Observed bi-weekly silent test of sirens Documentation for Siren testing and maintenance, January 2005 - December 2005

Section 1EP3: Emergency Response Organization (ERO) Augmentation

Procedures

EPM-602, Testing of Dialogic System, 151801, Action Request 151174, Action Request

Records and Data

1/4/06, ERO Roster 2/15/05, 0401 unannounced call out drill 2/23/05, 0348 unannounced call out drill 5/10/05, 2000 unannounced pager test 8/30/05, 1900 unannounced pager test 12/6/05, 1900 unannounced pager test

Section 1EP4: Emergency Action Level (EAL) and Emergency Plan Changes

Procedures

REG-NGGC-0010, 10 CFR 50.59 and Selected Regulatory Reviews, Rev. 8 PLP-201, Emergency Plan, Rev. 50 PRO-NGGC-0204, Procedure Review and Approval, Rev. 8 EP-EAL, Emergency Action Level Guidelines, Rev.2

Plans and Changes packages

A-2 Attachment

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05-0142, EPM-420, Emergency Equipment Inventory, Rev. 5 05-0048, EPM-100, EP Program Administration, Rev. 7 04-0245, EPM-200, ERO Training Program, Rev. 5 05-0419, EPM-200, ERO Training Program, Rev. 6 177805, EPM-200, ERO Training Program, Rev. 8 05-0231, EPM-210, EP Drill and Exercise Program, Rev. 9 04-1219, EPM-211, EP Scenario Development Guidelines, Rev. 4 05-0041, EPM-400, Public Notifications, Rev. 9 05-0430, EPM-602, Testing of Dialogic SystemRev. 1 05-0195, PEP-250, Activation and Operation of JIC, Rev. 11 04-1167, PEP-270, Activation and Operation of EOF, Rev. 13 159330, PEP-310, Notifications, Rev. 19 05-0181, PEP-330, Radiological Consequences, Rev. 7 05-0141, PEP-340, EP Drill and Exercise Program, Rev. 12 170895, PLP-201, Emergency Plan, Rev. 50
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Section 1EP5:Correction of Emergency Preparedness Weaknesses and Deficiencies

Procedures

EPM-100, EP Program Administration, Rev. 7 EPM-200 ERO Training Program, Rev. 8 EPM-210 EP Drill and Exercise Program, Rev. 10 ADM-NGGC-0201, Nuclear Task Management, Rev. 1 CAP-NGGC-0200, Corrective Action Program, Rev. 16

Records and Data

Memo- Self Evaluation Expectations, March 23, 2006

EP File#4.4, January 27, 2005, Full Scale EP Drill & Augmentation Drills Critique Report, 2/25/2005

Harris Task Force meeting, 12/2005

EP File#4.4, April 5, 2005, FEMA/NRC Graded Exercise Critique Report, 4/25/05

EP File#4.4, July 14, 2005, 05-03 Training Drill Critique Report, 8/11/05

EP File#4.4, September 13, 2005, 05-04 Drill Critique Report, 9/27/05

EP File#4.4, November 10, 2005, 05-05 Drill Critique Report, 03/10/06, Rev. 1

H-EP-05-02 HNAS-05-091, Harris Emergency Preparedness Assessment, 12/7/2005

A-3 Attachment

Miscellaneous

Atomic Safety and Licensing Board Panel (ASLBP) No. 82-472-03, Shearon Harris ANS issues; and ASAB-852, Appeal of Shearon Harris ANS issues

Section 40A1: Performance Indicator (PI) Verification

Procedures

REG-NGGC-0009, NRC Performance Indicators and Monthly Operating Report Data, Rev. 5 PLP-115, NRC Performance Indicator (PI) Program Procedure, Rev. 0

Records and Data

Documentation (scenarioes, line event notification forms/critique reports) for ERO drills and Operations simulator evaluations, January 2005 - December 2005 Selected training records of drill/exercise participation by ERO personnel for 2005 DSS-006, Licensed Operator Requalification Simulator Examination Scenario, Rev. 13 DSS-015, Licensed Operator Requalification Simulator Examination Scenario, Rev. 11 DSS-034, Licensed Operator Requalification Simulator Examination Scenario, Rev. 2

A4 Attachment

LIST OF ACRONYMS

ANS Alert and Notification System
DEP Drill/Exercise Performance
DIE Decrease in Effectiveness
EAL Emergency Action Level

ERO Emergency Response Organization

NCV Non-Cited Violation
NEI Nuclear Energy Institute

NRC Nuclear Regulatory Commission

PI Performance Indicator PS Planning Standard

RSPS Risk Significant Planning Standard SDP Significance Determination Process

A-5 Attachment

placed in the PDR and PARS without redaction. ADAMS is accessible from the NRC Website at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Sincerely, /RA/ Brian R. Bonser, Chief Plant Support Branch 2 Division of Reactor Safety

Docket Nos.: 50-400 License Nos: NPF-83

Enclosure: Inspection Report 05000400/2006008

w/Attachment: Supplemental Information

cc w/encl:

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(cc w/encl cont'd - See page 3)

□ PUBLICLY AVAILABLE □ NON-PUBLICLY AVAILABLE □ SENSITIVE □ NON-SENSITIVE

ADAMS:

Yes ACCESSION NUMBER:

OFFICE	RII:DRS	RII:DRP	RII:DRS				
SIGNATURE	RA	RA	RA				
NAME	MILLER	FREDRICKSON	BONSER				
DATE	5/5/2006	5/5/2006	5/5/2006	5/ /2006	5/ /2006	5/ /2006	5/ /2006
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO